NAC Form 308 (9-83)		LICE	INSE	E EVE	T RE	PORT	(LER)	U.S. NUC AP EX	CLEAR REGULAT	0RY COMMISSION 0. 3150-0104			
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NRC Form 388 (9-83)

NRC Form 386A (9-83) LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OMB NO. 3150-0104 EXPIRES. 8/31/85											ION									
FACILITY NAME (1)		DOCKET NUMBER (2)					T	LER NUMBER (6)						1	PAGE (3)					
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Event Date: 09/22/87 Event Time: 0840 This LER was initiated	l by	In	cid	eı	nt	Rep	001	rt	No		87	-14	17							
CONDITIONS PRIOR TO OC	CURI	REN	CE																	

Plant in OPERATIONAL CONDITION 4 (Cold Shutdown), Reactor Power 0%. Scheduled maintenance outage in progress.

DESCRIPTION OF OCCURRENCE

On September 22, 1987 at 0840 an NSSSS Channel "D" isolation of "B" RHR loop occurred when a fuse was blown on a portion of the Channel "D" isolation logic. Because the isolation caused the "A" and "B" RHR loop common suction valve to close, a loss of shutdown cooling resulted. During the course of this event, reactor coolant inventory temperature was maintained via the Reactor Water Cleanup system, with Low Pressure Ccolant Injection in standby as an alternate means of decay heat removal, if necessary. At 0922, "B" RHR loop was returned to service and shutdown cooling was restored.

APPARENT CAUSE OF OCCURRENCE

Inadequate access to the Steam Leak Detection (SLD) panels for testing and installation of test equipment.

ANALYSIS OF OCCURRENCE

Investigation subsequent to the RHR isolation determined that the Channel "D" isolation logic fuse was blown during performance of an I&C surveillance procedure when test equipment leads inadvertently came in contact with the ground bus inside the SLD panel in which the test was being performed. The circumstances surrounding this incident are similar to past incidents (reference: LER's 86-057, 86-089, 87-003 and 87-010) in that lack of access to monitoring/calibration points inside the SLD panels directly contributes to M&TE leads inadvertently contacting the panel ground bus. LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OM8 NO 3150-0104 EXPIRES 8/31/85

PACILITY KAME (1)	DOCKET NUMBER (2)	LER NUMBER (S)	PAGE (3)			
		YEAR SEQUENTIAL REVISION NUMBER				
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ANALYSIS OF OCCURRENCE CONT'D

When a lead is grounded, the fuse in the NSSSS logic card associated with the temperature module being tested will blow, resulting in an isolation signal to equipment in the area being monitored by the temperature module.

The potential safety impact associated with this incident was minimal. The reactor was in a cold shutdown condition at the time that RHR isolated, and temperatures were maintained with RWCU for the short duration of this event. Had RWCU not been able to maintain reactor coolant temperature, LPCI was in standby and available for decay heat removal.

CORRECTIVE ACTIONS

In response to the previously identified occurrences, a major design change package had been initiated to modify the internal architecture of the SLD panels to allow better access for testing. This design change will be implemented during and prior to completion of the first refueling outage. Other administrative corrective actions identified in the previous LER's will remain in effect until this design change is implemented.

Sincerely, terna S. Owl

S. LaBruna General Manager -Hope Creek Operations

RBC/

SORC Mtg. 87-154

AC Form 386A



Public Service Electric and Gas Company P.O. Box L. Hancocks Bridge, New Jersey 08038 Hope Creek Operations



October 20, 1987

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Dear Sir:

HOPE CREEK GENERATING STATION DOCKET NO. 50-354 UNIT NO. 1 LICENSEE EVENT REPORT 87-003-00

This Licensee Event Report is being submitted pursuant to the requirements of 10 CFR50.73(a)(2)(v).

Sincerely,

da

S. LaBruna General Manager -Hope Creek Operations /

RBC/

Attachment SORC Mtg. 87-154

C Distribution



The Energy People