Docket Nos. 50-275 and 50-323

Pacific Gas and Electric Company ATTN: Mr. John C. Morrissey Vice President and General Counsel 77 Beale Street San Francisco, California 94106

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Gentlemen:

The purpose of this letter is to call your attention to the list of items which remain outstanding in the safety review of the application for operating licenses for Diablo Canyon Units 1 and 2. A list of these items and their status is given in the enclosure to this letter and in Section 22 of Supplement Number 3 to the Safety Evaluation Report. As indicated in the enclosure, the resolution of a number of these items cannot be completed until additional information has been received from you. Although we realize that the majority of your efforts in recent months have been spent in the areas of geology and seismology, we want to emphasize that all of the outstanding items listed in the enclosure must be satisfactorily resolved before the issuance of operating licenses for the Diablo Canyon Units.

Please inform us within 7 days after receipt of this letter of your schedule for supplying the additional information that is required for all of the outstanding items listed in the enclosure. Do not hesitate to contact us if you have any questions regarding the information that has been requested.

Sincerely,

Original Staned by O. D. Parr

Olan D. Parr, Chief Light Water Reactors Project Branch 1-3 Division of Reactor Licensing NA3

Enclosure: Outstanding Items in the Diablo Canyon Safety Review

CC: See page 2

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DATE 9/26/75

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ENCLOSURE

OUTSTANDING ITEMS IN THE DIABLO CANYON SAFETY REVIEW

1. Effects of Tsunamis Caused by Near-Shore Generators

Our evaluation of this item has not been completed; however, we presently do not believe that we will require any additional information.

2. Earthquake Potential of the Hosgri Fault

We have not yet received all of the additional information which was requested in our letter to you dated February 12, 1975.

3. Pipe Break Outside Containment

Additional information on this item was requested in our letter to you dated August 7, 1975.

4. Seismic Qualification of Category I Instrumentation and Electrical Equipment

Part of the required information regarding qualification of balance-of-plant equipment was requested in our letter to you dated August 7, 1975. The remainder of this information is being requested from Westinghouse on a generic basis.

6. Thermal and Hydraulic Design

We will require some additional information concerning fuel rod bowing after the first fuel cycle before our evaluation of this item can be completed. This matter is being handled on a generic basis with Westinghouse.

8. Single Failures in the Containment Sump Isolation Valves

Our evaluation of this item has not been completed. This item is related to the single failure analysis that we have requested in conjunction with the ECCS analysis (See item 9 below).

9. Emergency Core Cooling System Analysis

We have not received all of the additional information which was requested in our letters to you dated April 3, 1975, June 13, 1975, and July 9, 1975. In addition your final ECCS analysis for Unit 2 has not been received.

10. Physical Separation in the Process Analog System

Additional information on this item was requested in our letter to you dated September 4, 1975.

11. ATWS

Our evaluation of this item has not been completed. No additional information is required at this time.

12. Environmental Qualification of Category I Instrumentation and Electrical Equipment

Part of this information regarding qualification of balance-ofplant equipment was requested in our letter to you dated August 7, 1975. The remainder of this information is being requested from Westinghouse on a generic basis.

16. Feedwater Hammer

Our evaluation of this item has not been completed. No additional information is required at this time.

17. Tornado Missile Protection

We have not yet received all of the information which was requested in our letter to you dated July 11, 1975.

In addition, as we have discussed with your representatives the following information will also be required.

- (a) A discussion of areas other than the 4.16 kV switchgear rooms where the emergency power busses are vulnerable to damage from tornado missiles.
- (b) The reasons for the difference between the tornado missile resisting capabilities you have listed for the containment structure and the containment access hatches. (See Table 3.3-2 of the FSAR).
- (c) Additional discussion of the availability of reactor coolant makeup water following postulated damage to the refueling water storage tank from tornado missiles.

18. Fire Protection

Additional information on this item was requested in our letter to you dated August 7, 1975.

19. Adequacy of the Reactor Vessel Supports

Our evaluation of this item has not been completed. No additional information is required at this time.