

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Hope Creek Generating Station										DOCKET NUMBER (2) 0 5 0 0 0 3 5 4 1 OF 0 3										PAGE (3) 1 OF 3	
TITLE (4) Invalid Loss Of Coolant Accident (LOCA) Signal Isolation When Performing Test Due To Leaking Instrument Valve																					
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)							
														0 5 0 0 0							
0 9	2 0	8 7	8 7	0 4	2	0 0	1 0	2 0	8 7					0 5 0 0 0							
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																			
4		20.402(b)					20.406(e)					<input checked="" type="checkbox"/> 50.73(a)(2)(iv)					73.71(b)				
POWER LEVEL (10)		20.406(a)(1)(i)					50.36(a)(1)					50.73(a)(2)(v)					73.71(e)				
0 0 0 0		20.406(a)(1)(ii)					50.36(a)(2)					50.73(a)(2)(vii)					OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
		20.406(a)(1)(iii)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(A)									
		20.406(a)(1)(iv)					50.73(a)(2)(iii)					50.73(a)(2)(viii)(B)									
		20.406(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(ix)									
LICENSEE CONTACT FOR THIS LER (12)																					
NAME R.B. Cowles, Lead Engineer - Technical										TELEPHONE NUMBER 6 0 9 3 3 9 - 1 5 2 6 4											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPDOS											
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR					
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO									

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A Channel "A" Loss of Coolant Accident (LOCA) signal resulted in the isolation of various Channel "A" components. The invalid LOCA signal occurred during an I&C department sensor calibration of a reactor vessel reference leg pressure transmitter. While pressurizing the isolated transmitter from a test source, the associated instrument isolation valve leaked, resulting in a pressure spike on the reference leg. The pressure spike tripped the level transmitter which provides the Channel "A" LOCA signal. Subsequent investigation determined that the leaking instrument valve was the root cause of this incident. Corrective actions consisted of repairing the leaking valve and re-performing the subject sensor calibration.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Hope Creek Generating Station	0 5 0 0 0 3 5 4 8 7	—	0 4 2	—	0 0	0 2	OF 0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor (BWR/4)  
Reactor Vessel Instrumentation (EIIS Designation: JB)

IDENTIFICATION OF OCCURRENCE

Invalid Loss Of Coolant Accident (LOCA) Signal Isolation When  
Performing Test Due To Leaking Instrument Valve

Event Date: 09/20/87

Event Time: 0505

This LER was initiated by Incident Report No. 87-146

CONDITIONS PRIOR TO OCCURRENCE

Plant in OPERATIONAL CONDITION 4 (Cold Shutdown), Reactor  
Power 0%. Scheduled maintenance outage in progress.

DESCRIPTION OF OCCURRENCE

On September 20, 1987 at 0505, an isolation of various systems and components occurred during the performance of an I&C sensor calibration procedure on a reactor vessel reference leg pressure transmitter. A Channel "A" LOCA signal caused the initiation of "A" Filtration, Recirculation, and Ventilation System (FRVS) vent unit, "A" Service Water Pump, and provided Channel "A" inputs to the Primary Containment Isolation System and the LOCA sequencer. After determining the LOCA signal was invalid, operators returned all systems/components to a normal condition.

APPARENT CAUSE OF OCCURRENCE

Leaking instrument isolation valve at the pressure transmitter on which I&C department was performing the sensor calibration.

ANALYSIS OF OCCURRENCE

Technical Specifications require that the calibration and operability of reactor vessel pressure transmitter BB-PT-N090J be verified at least once every 18 months. An I&C department sensor calibration procedure (IC-SC.BB-019) is utilized to satisfy this requirement.

On 9/20/87, an I&C technician was beginning to perform IC-SC.BB-019, and had shut the instrument isolation valve for BB-PT-N090J. The procedure calls for pressurizing the transmitter with a test source and then performing various adjustments.

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ANALYSIS OF OCCURRENCE CONT'D

When the technician began pressurizing the transmitter, the above described isolations/actuators occurred. Investigation subsequent to the event determined that the instrument isolation valve for BB-PT-N090J was not seating properly and had leaked back through to the vessel pressure/level reference leg. When the technician attempted to pressurize the transmitter, the pressure was propagated from the test source through to the reference leg, and the resulting actuators/isolations occurred due to the vessel level 2 (-38") level transmitter receiving a spurious low level signal.

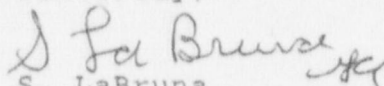
This event had no potential impact on plant safety, as all systems performed as expected, even though the initiating signal was invalid. Additionally, this sensor calibration is normally performed during shutdown conditions.

This incident was researched for similar previous occurrences at Hope Creek (instrument valves leaking through causing actuators/isolations), and none were noted.

CORRECTIVE ACTIONS

The instrument isolation valve for the subject pressure transmitter was repaired and the sensor calibration procedure was re-performed satisfactorily.

Sincerely,



S. LaBruna  
General Manager-  
Hope Creek Operations

RBC/  
SORC Mtg. 87-153





Public Service Electric and Gas Company P.O. Box L Hancocks Bridge, New Jersey 08038  
Hope Creek Operations

October 20, 1987

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

USNRC-DS  
1987 OCT 23 A 10:32

Dear Sir:

HOPE CREEK GENERATING STATION  
DOCKET NO. 50-354  
UNIT NO. 1  
LICENSEE EVENT REPORT 87-042-00

This Licensee Event Report is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(iv).

Sincerely,

S. LaBruna  
General Manager -  
Hope Creek Operations

RBC/

Attachment  
SORC Mtg. 87-153

C Distribution

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