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February 25, 1972

ACRS Members

## CONTAINMENT PRESSURE CALCULATION CAPABILITY

Attached are four items for background information relative to the subject of an independent capability by the Regulatory Staff to calculate containment pressure. This matter is scheduled for discussion at the March 1972 ACRS Meeting.

> M. C. Gaske Assistant to Executive Secretary

Attachments:

- 1. REG:RSR-72 2. DRD&T Ltr.,/2/3/71
- 3. REC:RSR-74, 1/31/72
- 4. AJN Ltr., 1/23/71 w/trip report

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## UNITED STATES ATOMIC ENERGY COMMISSION WASHINGTON. D.C. 20545

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> U.S. ATOMIC ENERGY COMM. ADVISORY COMMITTEE ON REACTOR SAFEGUARDS

Milton Shaw, Director, Division of Reactor Development and Technology

CONTAINMENT RESPONSE ANALYTICAL MODEL DEVELOPMENT - REG:RSR-72

The regulatory staff has immediate need for increased analytical capabilities regarding containment pressure-temperature response analysis. At this time, the staff is limited to making independent analyses of the peak pressores following a LOCA in dry PWR containments and BWR pressure suppression containments.

In order to adequately evaluate recent applications for certain commercial nuclear power plants, we require additional analytical capabilities for containment response analyses in the following specific areas, starting with the most urgently needed items.

- 1. Modification of CONTEMPT-PS to more correctly handle the long term pressure-temperature transients in a containment following a LOCA.
- 2. Development of an analytical code to calculate pressure build-up in reactor cavities and other enclosures.
- Development of a code to calculate the pressure-temperature transients in ice condenser containments.
- Modification of CONTEMPT-PS to include bypass flow between the drywell and wetwell.
- Modification of CONTEMPT-PS to handle pressure transients in the air annulus in dual containments.

As the above needs are evaluated, you may determine that some of these tasks can best be performed under the ANC Regulatory Technical Assistance Program even though the tasks are of a code development nature. Should this be the

Milton Shaw

case, we would have no objections since, as was pointed out in REG:RSR-69, the technical assistance needs for containment response analysis are closely coupled to analytical development efforts.

We would be happy to discuss these needs with your staff and provide additional specific details as may be required.

Edson G. Case, Director Division of Reactor Standards

cc: Members of Steering Committee on Reactor Safety Research G. M. Kavanagh

- C. K. Beck
- M. B. Biles
- P. A. Morris
- H. D. Bruner
- A. J. Pressesky
- W. H. Beach, OCM

- G. O. Bright, WRSPO
- R. F. Fraley, ACRS (18)

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