

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME: 01 F P C R P 3 | LICENSE NUMBER: 0 0 - 0 0 0 0 - 0 0 | LICENSE TYPE: 4 1 1 1 1 | EVENT TYPE: 0 3

CATEGORY: 01 CONT | REPORT TYPE: - - | REPORT SOURCE: L L | DOCKET NUMBER: 0 5 0 - 0 3 0 2 | EVENT DATE: 0 8 1 0 7 7 | REPORT DATE: 0 0 3 0 0 7

EVENT DESCRIPTION

02 | In Mode 1 operation, it was discovered that the Primary Containment Average Tempera-
03 | ture exceeded 130 degrees F. contrary to Technical Specification 3.6.1.5. Redundancy
04 | NA. Second occurrence of this event reported by LER 77-41. SWV-152 was opened to
05 | allow cooling water to return to industrial cooling system and reactor building
06 | average temperature decreased to less than 130 degrees F. within 30 minutes. LER 77-99

SYSTEM CODE: 07 10 | CAUSE CODE: 11 | COMPONENT CODE: 12 13 14 15 | PRIME COMPONENT SUPPLIER: 43 | COMPONENT MANUFACTURER: 44 45 46 47 | VIOLATION: 48

CAUSE DESCRIPTION

08 | Industrial Cooling Water Supply Valve, SWV-152, was closed. Procedural inadequacy
09 | in that opening of SWV-152 was not required upon completion of SP-344. SP-344 is
10 | being revised to verify that either the Nuclear Services Closed Cycle Cooling
11 | System (SW) or the Industrial Cooling System (CI) is aligned to supply cooling for the primary Containment.

FACILITY STATUS: 11 E | % POWER: 10 6 5 | OTHER STATUS: 12 13 14 15 16 17 | METHOD OF DISCOVERY: 44 45 46 47 | DISCOVERY DESCRIPTION: 48

FORM OF ACTIVITY RELEASED: 12 Z | CONTENT OF RELEASE: 10 11 12 13 14 15 16 17 | AMOUNT OF ACTIVITY: 44 45 46 47 | LOCATION OF RELEASE: 48

PERSONNEL EXPOSURES

NUMBER: 13 0 0 0 | TYPE: 12 2 | DESCRIPTION: 13 14 15 16 17 18 19 20

PERSONNEL INJURIES

NUMBER: 14 0 0 0 | DESCRIPTION: 11 12 13 14 15 16 17 18 19 20

OFFSITE CONSEQUENCES

15 | NA

LOSS OR DAMAGE TO FACILITY

TYPE: 16 Z | DESCRIPTION: 10 11 12 13 14 15 16 17 18 19 20

PUBLICITY

17 | NA

ADDITIONAL FACTORS

18 | NA

8707090361 870630
PDR ADOCK 05000302
S PDR

IE22 1/1

SEE ATTACHED SUPPLEMENTARY INFORMATION.

19 | NA

NAME: W. P. Stewart

PHONE: 813/866-4159

SUPPLEMENTARY INFORMATION

REPORT NO: 50-302/77-99-1
FACILITY: Crystal River Unit #3
REPORT DATE: June 30, 1987
OCCURRENCE DATE: August 10, 1977

IDENTIFICATION OF OCCURRENCE:

Primary containment average air temperature exceeded 130 degrees F. contrary to Technical Specification 3.6.1.5.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1 Power Operation.

DESCRIPTION OF OCCURRENCE:

At 1600, it was discovered that the Reactor Building average air temperature was 139 degrees F. Investigation revealed that while performing the Nuclear Services Cooling System Operability Surveillance Procedure, SP-344, the industrial cooling water supply valve SWV-152 was closed. Upon discovery of rise in Reactor Building temperature, SWV-152 was opened allowing cooling water to return to the industrial cooling water to return to the industrial cooling system, and the Reactor Building average temperature decreased to less than 130 degrees F. within 30 minutes.

DESIGNATION OF APPARENT CAUSE:

The cause of this event was due to a procedural inadequacy, in that upon completion SP-344 did not require opening of SWV-152.

ANALYSIS OF OCCURENCE:

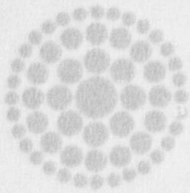
The initial temperature condition as assumed in LOCA accident analysis was exceeded. However, no hazard to plant or public was presented by this occurrence.

CORRECTIVE ACTION:

A procedure change has been initiated to ensure that upon completion of SP-344A and SP-344B cooling water supply and return valves for the Reactor Building Cooling Fans are open. These valves may be from either Nuclear Services Closed Cycle Cooling System or the Industrial Cooling System.

FAILURE DATA:

This is the second occurrence of this event as reported by LER 77-41 dated May 17, 1977.



**Florida
Power**
CORPORATION

June 30, 1987
3F0687-21

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Licensee Event Report No. 77-099-01

Dear Sir:

Enclosed is Licensee Event Report (LER) No. 77-099-01 which is submitted in accordance with 10 CFR 50.73.

Should there be any questions, please contact this office.

Sincerely,

E. C. Simpson
Director, Nuclear
Operations Site Support

WLR:mag

Enclosure

xc: Dr. J. Nelson Grace
Regional Administrator, Region II

Mr. T. F. Stetka
Senior Resident Inspector

IE22
11