TEXAS UTILITIES GENERATING COMPANY

July 28, 1978

TXX-2367

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
611 Ryan Plaza Dr., Suite 1000

Docket Nos. 50-445

COMANCHE PEAK STEAM ELECTRIC STATION
1981-83 2300 MW INSTALLATION
MISCLASSIFICATION OF SAFETY-RELATED EQUIPMENT
FILE NO. 10110

Dear Mr. Seidle:

Arlington, Texas 76011

A J GARY

This will supplement our letter of July 18, 1978 logged TXX-2860, wherein we committed to advise you by August 31, 1978 of the reportability of the interface breakdown, which resulted in the misclassification of safety-related equipment foundations by our architect engineer.

The engineering safety implication analysis has been completed. We have concluded that this item is not reportable under the provisions of 10 CFR 50.55(e).

The identified problems are being dispositioned in accordance with our established procedures.

Very truly yours,

50-446

RJG:dla

cc: Director, Office of Inspection and Enforcement (15)

Director, Office of Management Information and Program Control (1) U. S. Nuclear Regulatory Commission Washington, D. C. 20555

FOIA-87-508

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8709240284 870921 PDR FDIA WILLIAM#87-508 PDR

2001 BRY AN TOWER - DALLAS, TEXAS 75201

R J GARY

November 1, 1978 TXX-2900

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, Texas 76011

Docket Nos. 50-445 50-446

COMANCHE PEAK STEAM ELECTRIC STATION
1981-83 2300 MW INSTALLATION
SERVICE WATER PUMP SEISMIC RESTRAINT
INTERIM RESPONSE
FILE NO. 10110

Dear Mr. Seidle:

On October 6, 1978 we notified you by telephone that a question on the design of the Seismic Restraint for the Service Water Pump might be potentially reportable under 10 CFR 50.55(e). The engineer has conducted an evaluation of the design and has notified us that had the deficiency gone undetected it would not have adversely affected the safety of the plant. However, we are still evaluating his response, and will report to you our final determination by November 17, 1978.

Very truly yours,

R. J. Gary

RJG:dla

oc: Director, Office of Inspection and Enforcement (15)

Director, Office of Management Information and Program Control (1) U. S. Nuclear Regulatory Commission Washington, D. C. 20555

FOIA-87-508

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## TEXAS UTILITIES GENERATING CONFERNY

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October 18, 1978 TXX-2890

A LANG.

Mr. K. V. Seyfeit, Director J. S. Nuclear Regulatory Commission Region IV 511 Ryan Plaza Drive, Satte 1000 Arlington, Texas 76011

Docket Nos 50-445

COMANCHE PEAK STEAM ELECTRIC STATION 1981-83 2300 MW INSTALLATION COMANCHE PEAK CADMELDING FILE NO. 10210

Dear Mr. Sayfrit:

On October 4, 1978, we verbally informed your Mr. R. G. Taylor CPS/S
Resident Inspector, of a single Tailure of an installed cadveld sleave
on No. 18 reinforcing steel for the exterior wall of the Unit No. 1
containment structure that we were evaluating to a calcentially" opportable
deficiency within the definition of 10 CFF 50.55(e). We have completed
an in-depth investigation of exposed cadweld spines performed by the
individual craftsman involved and others working to the same procedures
and have concluded that the incident was an isolated case. Additionally,
engineering analyses clearly indicate that had the single incident gone
undetected, the matter would not have been reportable under 10 CFR 50.55(e).
Records supporting these determinations are available for your inspector's
review at the CPSES site.

However, as discussed with you on October 11, 1978, we have recognized, as the result of our evaluation, the need for corrective action in the management of our overall cadweld program. The following items have been or will be implemented to correct these management deficiencies:

- Construction procedures and being revised and will be fully implemented by Outcher 20, 1978.
- Gadwelders and their forence will be retrained to the revised procedures by October 20, 1778, with sephasis placed on craftsmen inspecting their own work.
- 3. Each fore an will place his identifying mark on every splice made by his crew and will furnish to the rebar superintendent a daily list of splices made by his crew. This action will remain in effect until three consecutive months elapse with no defective calwelds.
- 4. All affected codweld inspectors were retrained by October 10, 1978.

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TXX-2896 Page 2 Comanche Peak Cadwelding 5. Each cadweld inspector whose inspections were determined to be suspect during our investigation has been counseled at length by the TUGCO site QA Supervisor. This counseling emphasized that anything less than 100% will not be tolerated. 6. Senior TUGCO QA/QC staff personnel will monitor cadweld inspection on a daily basis until adequate confidence level in our inspection effort is restored, then periodically to maintain this level of confidence. If you have any further questions, please advise. Very truly yours, RJG:dla cc: L. F. Fikar

TEXAS UTILITIES GENERATING COMPANY

2001 BRYAN TOWER - DALLAS, TEXAS 76201

September 18, 1980

TXX-3195

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection & Enforcement
611 Ryan Plaza Dr., Suite 1000

Docket Nos. 50-445

COMANCHE PEAK STEAM ELECTRIC STATION 1981-83 2300 MW INSTALLATION CLASS V PIPING SUPPORTS FILE NO: 10110

Dear Mr. Seidle:

Arlington, Texas 76012

In accordance with 10 CFR 50.55(e) we are submitting the attached report of actions taken to correct discrepancies involving installation and inspection requirements for Class V non-nuclear safety-related piping supports. We previously made a verbal report to Mr. R. G. Taylor on November 20, 1979 and submitted interim reports logged TXX-3127 and TXX-3162 dated April 21, 1980 and July 14, 1980, respectively.

Supporting documentation is available at the job-site for your inspector's review.

If we can provide any additional information, please advise.

Very truly yours,

50-446

- 1000

RJG:dk Attachment

cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforcement - (15 copies) c/o Distribution Services Branch, DDC, ADM. U. S. Nuclear Regulatory Commission

Washington, D. C. 20555

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#### CLASS V PIPING SUPPORTS

### Description of the Deficiency

Various sections of the FSAR establish the pertinent requirements for the design fabrication, installation, and inspection of seismic supports for Class V piping systems. The applicable project specification 2323-MS-46B does not clearly define the applicable quality requirements for the supports. Consequently, there is insufficient positive assurance that the FSAR commitments have been satisfied.

### Safety Implications

As a result of the stated deficiency, there is no documented assurance that the supports would satisfy the design conditions and withstand the postulated seismic loadings. Therefore, this deficiency could possibly reduce the functioning of a Seismic Category I system or component to an unacceptable safety level.

### Corrective Actions

To address the stated deficiency, the following corrective actions have been completed:

- A) FSAR changes have been instituted to more clearly delineate the requirements for Seismic Category II piping supports for Class V piping systems.
- B) Changes to project specification 2323-MS-46B have been issued to detail and clarify the quality requirements and pertinent provisions of 10 CFR 50, Appendix B, to be applied during design, fabrication and installation and inspection.
- C) Implementing construction and inspection instructions have been prepared and are being used for ongoing work activities. Previously installed hangers are being reworked as necessary and inspected on a system basis.

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TEXAS UTILITIES GENERATING COMPANY

BOOL BRYAN TOWER SALLAR, TEXAN 78201

R. J. GARY BRECUTIVE VICE PAREIDENT BRO GENERAL MOVINGEN October 10, 1980 TXX-3217

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection & Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, Texas 76012

Docket Nos. 50-445 50-446

COMANCHE PEAK STEAT ELECTRIC STATION
1981-83 2300 MT INSTALLATION
UPPER INTERNALS ROTO-LOCK INSERT
FILE NC: 10110

Dear Mr. Seidle:

On September 13, 1980, we verbally informed your Mr. Ray Hall that we were evaluating a deficiency with the Roto-lock insert locking tab as a potentially reportable deficiency within the definitions of 10 CFR 50.55(e).

We have completed our investigation and have concluded that the matter is not reportable under 10 CFR 50.5! (e). Records supporting this determination are available for your Inspector's review at the CPSES site.

Very truly yours.

R. G. Cary

RJG:dk

cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforce ent - (15 copies) c/o Distribution Services Bran n, DDC. ADM. U. S. Nuclear Regulatory Commission Washington, D. C. 20555

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2001 BRYAN TOWER - DALLAS, TEXAS 76201

R J GARY

October 29, 1980 TXX-3229

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection & Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, Texas 76012

Docket Nos. 50-445 50-446

COMANCHE PEAK STEAM ELECTRIC STATION PLASITE 7122 COATING FILE NO: 10110

Dear Mr. Seidle:

On September 18, 1980, we verbally informed your Mr. R. G. Taylor that we were evaluating a condition involving the Plasite 7122 lining in the Service Water System Piping as a potentially reportable deficiency within the definitions of 10 CFR 50.55(e). We previously submitted an interim status report logged TXX-3218, dated October 13, 1980.

We have completed our investigation and have concluded that the matter is not reportable under 10 CFR 50.55(e). Documentation supporting this determination is available for your Inspector's review at the CPSES site.

Very truly yours,

Ry Lang

RJG:dk

cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforcement - (15 copies) c/o Distribution Services Branch, DDC, ADM. U. S. Nuclear Regulatory Commission Washington, D. C. 20555

FOI A-87-508

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2001 BRYAN TOWER - DALLAS, TEXAS 75201

R. J. GARY

October 15, 1980 TXX-3219

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection & Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, lexas 76012

Docket Nos. 50-445 50-446

COMANCHE PEAK STEAM ELECTRIC STATION 1981-83 2300 MW INSTALLATION THREAD ENGAGEMENT OF SWAY STRUTS FILE NO: 10110

Dear Mr. Seidle:

On September 19, 1980, we verbally informed your Mr. R. G. Taylor that we were evaluating a condition involving sway strut units which do not have threads upset as a potentially reportable deficiency within the definitions of 10 CFR 50.55(e).

We have completed our investigation and have concluded that the matter is not reportable under 10 CFR 50.55(e). Records supporting this determination are available for your Inspector's review at the CPSES site.

Very truly yours,

R. J. Gary

RJG:dk

cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforcement - (15 copies) c/o Distribution Services Branch, DDC, ADM. U. S. Nuclear Regulatory Commission Washington, D. C. 20555

FOIA-87-508

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0 -11 28/50/60 TEXAS UTILITIES GEN RAIING COMPA 2001 DRIAN TOWER O'LL IN TENAR PAROLINA November 30, 1982 TXX-3594 Mr. G. L. Madsen, Chief Reactor Projects Branch 1 U. S. Nuclear Regulatory Commission Office of Inspection and Enforcement 511 Ryan Plaza Drive, Suite 1000 Docket Nos.: 50-445 Arlington, TX 76012 50-446 COMANCHE PEAK STEAM ELEITRIC STATION OVER-TORQUING OF SAFETY RELIEF VALVES FILE NO. 10 10

Dear Mr. Madsen:

R. J. GARY AND BENERAL MANAGER

> On September 10, 1982, we verbally informed your Mr. R. G. Taylor of a deficiency regarding the over-torquing of safety rel ef valves. We submitted an interim report logged TXX-3577 on October 6, 1982.

We have completed our investigation and concluded that the matter is not reportable under 10 CFR50.55(e). Records supporting this determination are available for your inspector's review at the CPSES site.

Very truly yours,

RJG: eag

cc: NRC REGION IV - (0 + 1 copy)

Director, Inspection & Enforcement (15 copies) U. S. Nuclear Regulatory Commission Washington, DC 20555

FOIA-81-508

cc: R. J. Gary (2)

2 11.2

B. R. Clements
D. N. Chapman
R. L. Ramsey
J. C. Kuykendall
J. T. Merritt

D. Frankum
H. C. Schmidt
B. S. Dacko
R. G. Tolson
R. G. Taylor
F. B. Shants

2001 BRYAN TOWER DALLAS TEX AS 75201-3050

EXECUTIVE VICTE PRESIDENT

# FOIA-87-508

October 29, 1982 TXX-3589

3 1982

Mr. G. L. Madsen, Chief Reactor Project Branch 1 U. S. Nuclear Regulatory Commission Office of Inspection and Enforcement 611 Ryan Plaza Drive, Suite 1000 Arlington, TX 76012

Docket Nos. 50-445

50-446

COMANCHE PEAK STEAM ELECTRIC STATION AUXILIARY BUILDING CONCRETE FLOOR SLABS FILE NO.: 10110

Dear Mr. Madsen:

On October 20, 1982, we verbally informed your Mr. R. G. Taylor of a deficiency regarding embedded rotafoam and plywood in the Auxiliary Building concrete floor slabs.

We have completed our investigation and concluded that the matter is not reportable under 10CFR50.55(e). Records supporting this determination are available for your Inspector's review at the CPSES site.

Very truly yours,

for R. J. Gary

RJG: eaq

cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforcement (15 copies) U. S. Nuclear Regulatory Commission Washington, DC 20555

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