

TEXAS UTILITIES GENERATING COMPANY

2001 BRYAN TOWER - DALLAS, TEXAS 75201

July 28, 1978
TXX-2867

R. J. GARY
EXECUTIVE VICE PRESIDENT
AND GENERAL MANAGER

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, Texas 76011

Docket Nos. 50-445
50-446

COMANCHE PEAK STEAM ELECTRIC STATION
1981-83 2300 MW INSTALLATION
MISCLASSIFICATION OF SAFETY-RELATED EQUIPMENT
FILE NO. 10110

Dear Mr. Seidle:

This will supplement our letter of July 18, 1978 logged TXX-2860, wherein we committed to advise you by August 31, 1978 of the reportability of the interface breakdown, which resulted in the misclassification of safety-related equipment foundations by our architect engineer.

The engineering safety implication analysis has been completed. We have concluded that this item is not reportable under the provisions of 10 CFR 50.55(e).

The identified problems are being dispositioned in accordance with our established procedures.

Very truly yours,

R. J. Gary
R. J. Gary

RJG:dla

cc: Director, Office of Inspection and Enforcement (15)
Director, Office of Management Information and Program Control (1)
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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PDR FOIA
WILLIAM 87-508 PDR

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TEXAS UTILITIES GENERATING COMPANY

2001 BRYAN TOWER - DALLAS, TEXAS 75201

November 1, 1978
TXX-2900

R J GARY
EXECUTIVE VICE PRESIDENT
AND GENERAL MANAGER

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, Texas 76011

Docket Nos. 50-445
50-446

COMANCHE PEAK STEAM ELECTRIC STATION
1981-83 2300 MW INSTALLATION
SERVICE WATER PUMP SEISMIC RESTRAINT
INTERIM RESPONSE
FILE NO. 10110

Dear Mr. Seidle:

On October 6, 1978 we notified you by telephone that a question on the design of the Seismic Restraint for the Service Water Pump might be potentially reportable under 10 CFR 50.55(e). The engineer has conducted an evaluation of the design and has notified us that had the deficiency gone undetected it would not have adversely affected the safety of the plant. However, we are still evaluating his response, and will report to you our final determination by November 17, 1978.

Very truly yours,

R. J. Gary
R. J. Gary

RJG:dla

cc: Director, Office of Inspection and Enforcement (15)
Director, Office of Management Information and Program Control (1)
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

FOIA-87-508

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TXX-2896

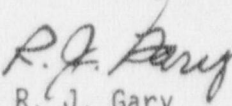
Page 2

Comanche Peak Cadwelding

5. Each cadweld inspector whose inspections were determined to be suspect during our investigation has been counseled at length by the TUGCO site QA Supervisor. This counseling emphasized that anything less than 100% will not be tolerated.
6. Senior TUGCO QA/QC staff personnel will monitor cadweld inspection on a daily basis until adequate confidence level in our inspection effort is restored, then periodically to maintain this level of confidence.

If you have any further questions, please advise.

Very truly yours,


R. J. Gary

RJG:d1a

cc: L. F. Fikar

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TEXAS UTILITIES GENERATING COMPANY

2001 BRYAN TOWER - DALLAS, TEXAS 75201

R J GARY
EXECUTIVE VICE PRESIDENT
AND GENERAL MANAGER

September 18, 1980
TXX-3195

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection & Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, Texas 76012

Docket Nos. 50-445
50-446

COMANCHE PEAK STEAM ELECTRIC STATION
1981-83 2300 MW INSTALLATION
CLASS V PIPING SUPPORTS
FILE NO: 10110

Dear Mr. Seidle:

In accordance with 10 CFR 50.55(e) we are submitting the attached report of actions taken to correct discrepancies involving installation and inspection requirements for Class V non-nuclear safety-related piping supports. We previously made a verbal report to Mr. R. G. Taylor on November 20, 1979 and submitted interim reports logged TXX-3127 and TXX-3162 dated April 21, 1980 and July 14, 1980, respectively.

Supporting documentation is available at the job-site for your inspector's review.

If we can provide any additional information, please advise.

Very truly yours,

R. J. Gary
R. J. Gary

RJS:dk
Attachment
cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforcement - (15 copies)
c/o Distribution Services Branch, DDC, ADM.
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

FOIA-87-508

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CLASS V PIPING SUPPORTS

Description of the Deficiency

Various sections of the FSAR establish the pertinent requirements for the design fabrication, installation, and inspection of seismic supports for Class V piping systems. The applicable project specification 2323-MS-46B does not clearly define the applicable quality requirements for the supports. Consequently, there is insufficient positive assurance that the FSAR commitments have been satisfied.

Safety Implications

As a result of the stated deficiency, there is no documented assurance that the supports would satisfy the design conditions and withstand the postulated seismic loadings. Therefore, this deficiency could possibly reduce the functioning of a Seismic Category I system or component to an unacceptable safety level.

Corrective Actions

To address the stated deficiency, the following corrective actions have been completed:

- A) FSAR changes have been instituted to more clearly delineate the requirements for Seismic Category II piping supports for Class V piping systems.
- B) Changes to project specification 2323-MS-46B have been issued to detail and clarify the quality requirements and pertinent provisions of 10 CFR 50, Appendix B, to be applied during design, fabrication and installation and inspection.
- C) Implementing construction and inspection instructions have been prepared and are being used for ongoing work activities. Previously installed hangers are being reworked as necessary and inspected on a system basis.

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TEXAS UTILITIES GENERATING COMPANY

8001 BRYAN TOWER, DALLAS, TEXAS 75201

R. J. GARY
EXECUTIVE VICE PRESIDENT
AND GENERAL MANAGER

October 10, 1980
TXX-3217

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection & Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, Texas 76012

Docket Nos. 50-445
50-446

COMANCHE PEAK STEAM ELECTRIC STATION
1981-83 2300 MW INSTALLATION
UPPER INTERNALS KOTO-LOCK INSERT
FILE NO: 10110

Dear Mr. Seidle:

On September 13, 1980, we verbally informed your Mr. Ray Hall that we were evaluating a deficiency with the Koto-lock insert locking tab as a potentially reportable deficiency within the definitions of 10 CFR 50.55(e).

We have completed our investigation and have concluded that the matter is not reportable under 10 CFR 50.55(e). Records supporting this determination are available for your Inspector's review at the CPSES site.

Very truly yours,

R. J. Gary
R. J. Gary

RJG:dk

cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforcement - (15 copies)
c/o Distribution Services Branch, DDC, ADM.
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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TEXAS UTILITIES GENERATING COMPANY

2001 BRYAN TOWER - DALLAS, TEXAS 75201

R. J. GARY
EXECUTIVE VICE PRESIDENT
AND GENERAL MANAGER

October 29, 1980
TXX-3229

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection & Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, Texas 76012

Docket Nos. 50-445
50-446

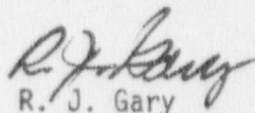
COMANCHE PEAK STEAM ELECTRIC STATION
PLASITE 7122 COATING
FILE NO: 10110

Dear Mr. Seidle:

On September 18, 1980, we verbally informed your Mr. R. G. Taylor that we were evaluating a condition involving the Plasite 7122 lining in the Service Water System Piping as a potentially reportable deficiency within the definitions of 10 CFR 50.55(e). We previously submitted an interim status report logged TXX-3218, dated October 13, 1980.

We have completed our investigation and have concluded that the matter is not reportable under 10 CFR 50.55(e). Documentation supporting this determination is available for your Inspector's review at the CPSES site.

Very truly yours,


R. J. Gary

RJG:dk

cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforcement - (15 copies)
c/o Distribution Services Branch, DDC, ADM.
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

FOIA-87-508

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TEXAS UTILITIES GENERATING COMPANY

2001 BRYAN TOWER · DALLAS, TEXAS 75201

R. J. GARY
EXECUTIVE VICE PRESIDENT
AND GENERAL MANAGER

October 15, 1980
TXX-3219

Mr. W. C. Seidle, Chief
Reactor Construction & Engineering
Support Branch
U. S. Nuclear Regulatory Commission
Office of Inspection & Enforcement
611 Ryan Plaza Dr., Suite 1000
Arlington, Texas 76012

Docket Nos. 50-445
50-446


COMANCHE PEAK STEAM ELECTRIC STATION
1981-83 2300 MW INSTALLATION
THREAD ENGAGEMENT OF SWAY STRUTS
FILE NO: 10110

Dear Mr. Seidle:

On September 19, 1980, we verbally informed your Mr. R. G. Taylor that we were evaluating a condition involving sway strut units which do not have threads upset as a potentially reportable deficiency within the definitions of 10 CFR 50.55(e).

We have completed our investigation and have concluded that the matter is not reportable under 10 CFR 50.55(e). Records supporting this determination are available for your Inspector's review at the CPSES site.

Very truly yours,


R. J. Gary

RJG:dk

cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforcement - (15 copies)
c/o Distribution Services Branch, DDC, ADM.
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

FOIA-87-508

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TEXAS UTILITIES GENERATING COMPANY

2001 RYAN TOWER DRIVE, ARLINGTON, TEXAS 76010-9990

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R. J. GARY
EXECUTIVE VICE PRESIDENT
AND GENERAL MANAGER

November 30, 1982
TXX-3594

Mr. G. L. Madsen, Chief
Reactor Projects Branch 1
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76012

Docket Nos.: 50-445
50-446

COMANCHE PEAK STEAM ELECTRIC STATION
OVER-TORQUING OF SAFETY RELIEF VALVES
FILE NO. 10 10

Dear Mr. Madsen:

On September 10, 1982, we verbally informed your Mr. R. G. Taylor of a deficiency regarding the over-torquing of safety relief valves. We submitted an interim report logged TXX-3577 on October 6, 1982.

We have completed our investigation and concluded that the matter is not reportable under 10 CFR50.55(e). Records supporting this determination are available for your inspector's review at the CPSES site.

Very truly yours,

R. J. Gary
R. J. Gary

RJG:eeq

cc: NRC REGION IV - (0 + 1 copy)

Director, Inspection & Enforcement (15 copies)
U. S. Nuclear Regulatory Commission
Washington, DC 20555

FOIA-87-508

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cc: R. J. Gary (2)
B. R. Clements
D. N. Chapman
R. L. Ramsey
J. C. Kuykendall
J. T. Merritt
D. Frankum
H. C. Schmidt
B. S. Dacko
R. G. Tolson
R. G. Taylor
F. H. Shants

TEXAS UTILITIES GENERATING COMPANY

2001 BRYAN TOWER DALLAS, TEXAS 75201-3050

R. J. GARY
EXECUTIVE VICE PRESIDENT
AND GENERAL MANAGER

FOIA-87-508

October 29, 1982
TXX-3589

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Mr. G. L. Madsen, Chief
Reactor Project Branch 1
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76012

Docket Nos. 50-445
50-446

COMANCHE PEAK STEAM ELECTRIC STATION
AUXILIARY BUILDING CONCRETE FLOOR SLABS
FILE NO.: 10110

Dear Mr. Madsen:

On October 20, 1982, we verbally informed your Mr. R. G. Taylor of a deficiency regarding embedded rotofoam and plywood in the Auxiliary Building concrete floor slabs.

We have completed our investigation and concluded that the matter is not reportable under 10CFR50.55(e). Records supporting this determination are available for your Inspector's review at the CPSES site.

Very truly yours,

[Handwritten signature]
for R. J. Gary

RJG:eaq

cc: NRC Region IV - (0 + 1 copy)

Director, Inspection & Enforcement (15 copies)
U. S. Nuclear Regulatory Commission
Washington, DC 20555

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