ATTACHMENT 2

PROPOSED REVISED PAGES

1. SITE

1.1 LOCATION

The La Crosse Boiling Water Reactor (LACBWR) is located on the east bank of the Mississippi River, approximately 19 miles south of the city of La Crosse, Wisconsin, and 1 mile south of the village of Genoa, Wisconsin.

1.2 EXCLUSION AND RESTRICTED AREAS

1.2.1 The distance from the centerline of the reactor building to the boundary of the exclusion area, as defined in 10 CFR 100, shall be at least 1109 feet. Any changes in occupancy of the exclusion area which lead to residential uses shall be noticed to the NRC.

1.3 PRINCIPAL ACTIVITIES

The principal activities carried on within the exclusion area shall be the possession of the reactor and associated power-generating equipment, and the operation of conventional steam-electric plants and electrical transmission switching and distribution centers, all of which shall be located within the site boundary.

2. DESIGN AND PERFORMANCE REQUIREMENTS

2.1 REACTOR BUILDING

2.1.1 Containment Vessel

2.1.1.1 The containment vessel shall be capable of containing an internal pressure of 52 psig at 280°F, and it shall be capable of withstanding an external-over-internal pressure of 0.5 psi.

2.1.1.2 The leakage rate from the containment vessel shall correspond to a leakage rate that, per 24-hour period, does not exceed 0.1 percent by weight of a steam-air mixture at 273°F, 28.5 psig, and a steam-to-initial-air ratio of 2.2.

SURVEILLANCE REQUIREMENT - (Continued)

ASME Boiler and Pressure Vessel Code and applicable Addenda Terminology for inservice inspection and testing activities	Required frequencies for performing inservice inspection and testing activities
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Every 9 months	At least once per 276 days
Yearly or annually	At least once per 366 days

c. The provisions of Specification 3.0.7 are applicable to the above required frequencies for performing inservice inspection and testing activities.

d. Performance of the above inservice inspection and testing activities shall be in addition to other specified Surveillance Requirements.

e. Nothing in the ASME Boiler and Pressure Vessel Code shall be construed to supersede the requirements of any Technical Specification.

f. Deleted.

3.0.11 Deleted.

3.0.10 This specification ensures that inservice inspection of ASME Code Class 1,2, an 3 components and inservice testing of ASME Code Class 1,2, and 3 pumps and valves will be performed in accordance with a periodically updated version of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda as required by 10 CFR 50, Section 50.55a. Relief from any of the above requirements has been provided in writing by the Commission and is not a part of these Technical Specifications.

This specification includes a clarification of the frequencies of performing the inservice inspection and testing activities required by Section XI of the ASME Boiler and Pressure Vessel Code and applicable addenda. This clarification is provided to ensure consistency in surveillance intervals throughout these Technical Specifications and to remove any ambiguities relative to the frequencies for performing the required inservice inspection and testing activities.

Under the terms of this specification, the more restrictive requirements of the Technical Specifications take precedence over the ASME Boiler and Pressure Vessel Code and applicable Addenda. For example, the requirements of Specification 3.0.9 to perform surveillance activities prior to entry into an OPERATIONAL CONDITION or other specified applicability condition takes precedence over the ASME Boiler and Pressure Vessel Code provision which allows pumps to be tested up to one week after return to normal operation. and for example, the Technical Specification definition of OPERABLE does not grant a grace period before a device that is not capable of performing its specified function is declared inoperable and takes precedence over the ASME Boiler and Pressure Vessel Provision which allows a valve to be incapable of performing its specified function for up to 24 hours before being declared inoperable.

3.0.11 Deleted.

4.1 GENERAL

4.1.3 Deleted.

4.1.4 Whenever the reactor contains one or more fuel elements, any operations from points outside the control room of equipment which may affect the reactor shall be conducted under the direction, or with the knowledge, of the control room operator.

4.1.5 If the plant is operational during a tornado warning, the Shift Supervisor on duty shall keep informed of the actual tornado activity which may approach the plant. In the event that reports indicate an imminent tornado strike at or near the LACBWR plant, the Shift Supervisor shall reduce reactor power to a level which permits prompt reduction of power generation to station load. However, the Shift Supervisor shall be instructed to discontinue plant operation if, in his judgment, this action is required to ensure plant safety.

4.1.6 If the plant is in CONDITION 1, 2, or 3 and the Mississippi River level adjacent to the plant reaches 639.2' and is predicted to exceed 640', commence reactor shutdown and be in CONDITION 4 prior to the river level exceeding 640'.

4.2 OPERATING LIMITS

4.2.1 Reactor Building

4.2.1.1 CONTAINMENT INTEGRITY shall be maintained in Conditions 1, 2, 3 and during:

a. CORE ALTERATIONS,

b. handling of irradiated fuel, or

c. there is fuel in the reactor and any control rod is withdrawn.

4.2.1.2 Gasketed closures and ventilation system closures which have been subjected to maintenance, repair or other operations which might affect their performance shall, before any subsequent operation for which containment integrity is required, be tested for leak tightness using the soap-bubble technique (or other method of equivalent sensitivity). This test shall be performed using a pressure differential no less than 0.5 psi and the results shall be used as a guide in evaluating leakage.

REACTOR COOLANT SYSTEM

STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

4.2.2.23 The structural integrity of the ASME Code Class 1, 2 and 3 components shall be maintained in accordance with Specification 5.2.2.23.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, 4 and 5

ACTION:

a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDT considerations.

b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 212°F.

c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s)to within its limit or isolate the affected component(s) from service.

d. The provisions of Specification 3.0.4 are not applicable.

e. The provisions of Specification 3.0.3 are not applicable in OPERATIONAL CONDITION 5.

SURVEILLANCE REQUIREMENTS

5.2.2.23 The structural integrity of ASME Code Class 1, 2 and 3 components utilized in connection with fuel storage shall be demonstrated per the requirements of Specification 3.0.10.

The applicable surveillance requirements are set forth in Section XI of the ASME Boiler and Pressure Vessel Code, 1974 Edition, Summer 1975 Addenda, except where specific written relief has been granted from the Nuclear Regulatory Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).

4/5.2.2.23 STRUCTURAL INTEGRITY

The inspection and testing program conforms to the requirements of 10 CFR 50, Section 50.55a(g). To the extent practical, the inspection and testing of components utilized in connection with fuel storage classified into Class 1, 2, and 3 will conform to the requirements for ASME Code Class 1, 2, and 3 components contained in Section XI of the ASME Boiler and Pressure Vessel Code, "Inservice Inspection of Nuclear Reactor Coolant Systems," 1974 Edition, Summer 1975 Addenda.

Using Regulatory Guide 1.26, Revision 3, "Quality Group Classifications and Standards for Water, Steam, and Radioactive-Waste-Containing Components of Nuclear Power Plants" as a guide, LACBWR components have been classified in "Classes." This classification serves as the basis for determining which ASME Code Class inspection and testing requirements are applicable to a given component. 10 CFR 50, Section 50.55a(g) requires components which are part of the reactor coolant pressure boundary and their supports to meet the inservice inspection and testing requirements applicable to components classified as ASME Code Class 1. Other safety-related components must meet the inservice inspection and testing requirements applicable to components classified as ASME Code Class 2 or 3.

The inservice inspection and testing program must be updated at 120-month intervals in accordance with 10 CFR 50, Section 50.55a(b)(2). A description of the updated programs should be submitted to the NRC for review at least 3 months before the start of each interval.

The inservice inspection and testing program must, to the extent practical, comply with the requirements in editions and addenda to the ASME Code that are "in effect" more than twelve months before the start of the period covered by the updated program. The term "in effect" means both having been published by the ASME, and having been referenced in paragraph (b) of 10 CFR 50, Section 50.55a. If a code required inspection or test is impractical, requests for deviations are submitted to the Commission in accordance with 10 CFR 50, Section 50.55a(g)(6)(i). Deviation requests should, if possible, be submitted to the NRC for review at least 90 days before the start of each period, however, deviations identified during an inspection period may be requested.

4.2.4 Reactor Core

4.2.4.1 The reactor shall not be operated.