ADVISORY COMMITTEE ON REACTOR SAFEGUARDS

NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

November 20, 1975

M. Bender, Chairman C. F. Braun Subconmittee

SCHEDULE FOR ARCHITECT-ENGINEER BALANCE OF PLANT (C. F. BRAUN) SUBCOMMITTEE MEETING, DECEMBER 3, 1975

The Architect-Engineer Balance of Plant Subcommittee will meet on December 3, 1975 in Room 1062 at 1717 H Street N.W., Washington, DC to review the application by C. F. Braun and Company for preliminary design approval for the Braun Standard Safety Analysis Report (Braun SAR). The tentative schedule for Wednesday, December 3 is:

2:00 p.m. Executive Session (Closed)

. 2:30 p.m. Meeting with C. F. Braun & Company and the Nuclear Regulatory Commission Staff (Open)

4:30 p.m. Caucus (Closed)

5:00 p.m. Summation and Adjourn (Open)

Topics to be discussed include:

- Responsibilities of C. F. Braun, the nuclear steam system supplier, and the client utility for the integration of the standard plant elements.
- 2. Use of Braun SAR for multi-unit sites, with respect to the effect of turbine orientation.
- 3. Measures to cope with high energy pipe failures outside of containment.
- 4. The integration of on-site and off-site power for Braum SAR and the balance of plant.
- Fire protection integration for the turbine-generator island, the balance of plant, and the nuclear steam supply system.
- 6. The extent of C. F. Braun & Company participation in the overall plant quality assurance program.
- Coordination of the instrumentation sensors in the turbine-generator island with the displays in the balance of plant or nuclear steam supply system.

ATTACHMENT A

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- 8. Bounding seismic design criteria and the nature of the implicit seismic design margins.
- 9. Correlation of containment action features with the pressure/temperature of the feedwater/steam system control requirements for the turbine-generator and nuclear steam system.
- 10. Variations in feedwater and condensate systems necessary to accommodate three different turbire-generator vendors.
- 11. Criteria for steam valve sealing.
- 12. "As low as reasonably achievable" criteria for shielding, including source term quantification.
- 13. Ultimate heat sink interfaces with the nuclear steam supply system and the balance of plant.
- 14. Industrial security interfaces with the nuclear steam supply system and the balance of plant.

Topics may be added to or deleted from the above list as appropriate. Attendance by the following ACRS members is anticipated.

Mr. Bender

Dr. Okrent

Dr. Siess

cc: ACRS Members

B. Rusche

S. H. Hanauer

D. F. Knuth

H. J. C. Kouts

S. Varga

B. Grier

F. Schroeder

J. Stolz

R. Martin

ACRS SUBCOMMITTEE MEETING WASHINGTON, D C DECEMBER 3, 1975

My name is William A Jack. I am Vice-President of C F Braun & Co and Manager of its Power Division.

C F Braun & Co, founded in 1909, specializes in the planning, project management, engineering and construction of process and power plants throughout the world. Our annual business approximates \$350 million, of which over ninety percent involves engineering and construction of large projects. Our pertinent nuclear power work includes contracts with General Electric Company for the preparation of their Standard Reactor Island Design using the design criteria in GESSAR. The resulting design is being used for TVA's four Hartsville units and their two Phipps Bend units. Along with this effort, Braun has undertaken at its own expense and without contract or subsidy, the preparation of a matching Standard Turbine Island Design based upon the design criteria in Braun SAR.

It is our firm belief that maximum standardization of nuclear power plant design and construction is a significant answer to escalating plant costs and elongating schedules. Also, we believe such standardization could be the largest single contributor to reducing our nation's independence from foreign oil supply. Our Turbine Island design, as expressed in the Braun SAR, in conjunction with

ATTACHMENT B

the Reactor Island design identified in the 238 GESSAR, constitutes
the maximum practical standardization of a nuclear generating complex.
Only the site or client related features are excluded from
standardization. We are convinced, therefore, that our current
efforts in this field of standardization will contribute significantly
to the betterment and stability of the utility industry.

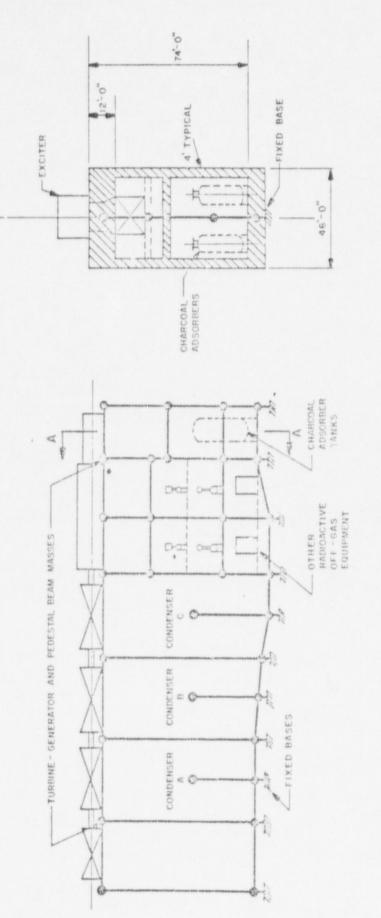
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No amount of standardization effort on our part, however, will be effective without assurance that the approved designs will be usable for a reasonable length of time without the requirement of re-review. We urge, therefore, that swift and early resolution of the PDA validity span be given top priority. We believe that three years is the minimum practical time of such validity. Surely, any time less than two years will completely eliminate the necessary incentive for industry's acceptance of standardization.

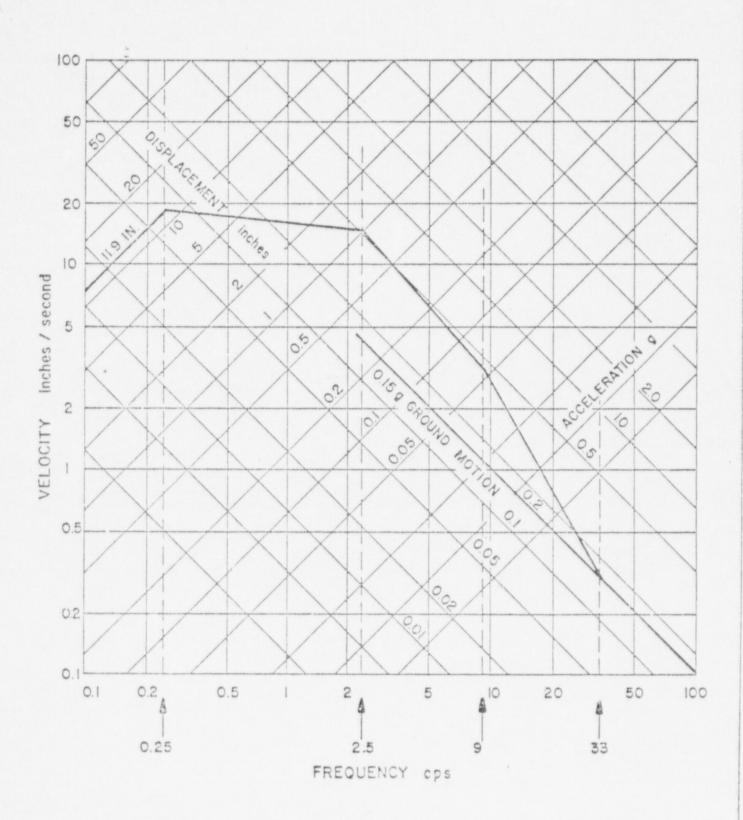
During the preparation of Braun SAR, extensive discussions took place between Braun and major equipment suppliers. From these discussions, we gained assurance that our design incorporates proven systems and equipment to the maximum extent. We achieved our design goal to not incorporate untried systems or equipment. And we believe that our design approach provides for selection of equipment, apparatus, systems and materials through competitive bidding.

In the course of the development of the design we had the benefit of review and comments from GE, TVA, Yankee Atomic, and Niagara Mohawk. We received valuable, up-to-date operating and maintenance information which are factored into our concept. And following our application we have continued to broaden our utility contracts. We see some indications of utility acceptance of the standard plant design approach. Granting a PDA for Braun SAR with its attendant guarantee of freedom from ratcheting is a key ingredient for positive acceptance.

Now, Mr William Shepherd, our Braun SAR Project Manager, will present the principal features and application of the Braun Standard Plant.



TURBINE GENERATOR
PEDESTAL MODEL
AMENDHENT 3
May 30, 1975



Viewgraph 56

FIGURE 3.2-2
HORIZONTAL DESIGN RESPONSE
SPECTRA FOR OFF-GAS
SYSTEM ENCLOSURE.

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