

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Hope Creek Generating Station DOCKET NUMBER (2) 050003541 OF 04 PAGE (3)

TITLE (4) Failure to Perform a Reactor Level Instrumentation Surveillance Within the Required Period - Personnel Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
08	18	87	87	038	00	09	17	87			05000
<p>THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)</p>											

OPERATING MODE (8) 1	20.402(b)	20.406(a)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 100	20.406(a)(1)(i)	50.38(a)(1)	50.73(a)(2)(v)	73.71(e)
	20.406(a)(1)(ii)	50.38(a)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
	20.406(a)(1)(iii)	X 50.73(a)(2)(ii)	50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)  
NAME: A. M. Ervin, Lead Engineer - Technical  
TELEPHONE NUMBER: 61093139-15239  
AREA CODE: 6109

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)  
YES (if yes, complete EXPECTED SUBMISSION DATE)  NO  X  
EXPECTED SUBMISSION DATE (15)

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 18, 1987 at 1520 hours the plant was in OPERATIONAL CONDITION 1 (Power Operation) at 100% power generating 1072 MWe. At this time it was determined that a surveillance of the reactor level instrumentation required by the plant Technical Specifications was overdue. The systems which are initiated by the level instrumentation were declared INOPERABLE and the applicable Technical Specification Action Statements were entered. The missed surveillance was completed satisfactorily at 1640 hours and the Technical Specification Action Statements were cleared. The root causes of this occurrence are (1) the failure to issue the Work Order for the surveillance to the department responsible for performing the surveillance test in a timely manner and (2) the failure to cross check work packages against the weekly look ahead surveillances list - personnel errors. Corrective actions include counselling of the individuals responsible for the personnel errors.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor (BWR/4)  
Nuclear Boiler and Reactor Recirc System (EISS Designator:AB)

IDENTIFICATION OF OCCURRENCE

Failure to Perform a Reactor Level Instrumentation Surveillance  
Within the Required Period - Personnel Error  
Event Date: August 18, 1987  
Event Time 1520 Hours  
This LER was initiated by Incident Report No. 87-125

CONDITIONS PRIOR TO OCCURRENCE

The Plant was in OPERATIONAL CONDITION 1 (Power Operation) at 100% power generating 1072 MWe.

DESCRIPTION OF OCCURRENCE

On August 18, 1987 at 1520 hours it was determined that a surveillance of the Reactor level instrumentation which was due on August 6, 1987 and overdue on August 16, 1987 had not been performed. Since the surveillance was overdue, the systems which are activated by the Reactor level instrumentation were declared inoperable in accordance with the Technical Specifications. These systems were "B" Core Spray (CS), "B" Reactor Heat Removal (RHR), Automatic Depressurization System (ADS), "B" Emergency Diesel Generator (EDG), Reactor Core Isolation Cooling (RCIC), Filtration Recirculation Ventilation System (FRVS) and Torus Water Cleanup (TWCU). The most limiting Technical Specification Action Statement was 3.5.1.d for ADS. This Action Statement requires that whenever two or more of the ADS valves are inoperable, the plant shall be in HOT SHUTDOWN within 12 hours and reduce steam dome pressure to ≤100 psig within the next 24 hours. The surveillance was completed satisfactory at 1640 hours on the discovery date and the Action Statements were cleared.

APPARENT CAUSE OF OCCURRENCE

The root causes of this occurrence are (1) the failure to issue the Work Order for the surveillance to the department responsible for performing the surveillance test in a timely manner and (2) the failure to cross check Work Order (WO) packages against the weekly look ahead surveillances list - personnel errors.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

ANALYSIS OF OCCURRENCE

A report was generated by the Hope Creek WO system which listed the Reactor Level instrumentation surveillance, however the WO package was not delivered to the department performing the surveillance test in time for the work to be done before the surveillance test was overdue. The Supervisor responsible for the surveillance test normally cross checks the weekly list of upcoming surveillances against the WO packages he receives to ensure that no surveillance tests are missed. This cross check was omitted for the week during which the missed surveillance was scheduled because of the increased work load following a plant trip recovery. When the missed surveillance was detected, the Control Room was immediately notified.

A previous occurrence involving a missed Technical Specification surveillance test was reported in LER 86-008 (April 24, 1986). At the time of this occurrence initial fuel loading was in progress concurrent with control rod scram time and friction testing. Fuel loading was suspended and all control rods were inserted several times for maintenance on the refueling bridge. Technical Specifications require that the Standby Liquid Control System (SLCS) be demonstrated OPERABLE whenever any control rod is withdrawn while in OPERATIONAL CONDITION 5. It was discovered that the 24 hour SLCS surveillances were not current (i.e. the surveillances had not been performed in the preceding 24 hours) when testing was resumed after a suspension of testing activities. The root cause of this Technical Specification violation was a personnel error in not performing the required surveillance during a suspension of testing.

The Hope Creek Technical Specifications require that a CHANNEL FUNCTIONAL TEST of the Reactor Vessel Level instrumentation be performed at least every 31 days (with a grace period of 25% of the test interval). When the required surveillance is not performed within a 31 day period, the Technical Specifications require that the affected systems be declared INOPERABLE and the plant be in HOT SHUTDOWN within 12 hours. The missed surveillance was performed satisfactorily, restoring all affected systems to OPERABLE condition before the 12 hour grace period had expired. Compliance with the plant Technical Specifications ensures that the plant is operated in a safe manner. For this reason, the health and safety of the public was not compromised by this occurrence.

This report is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(i).

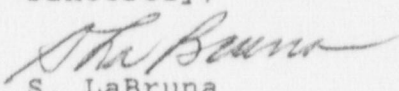
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CORRECTIVE ACTIONS

- (1) The person responsible for providing the WO for the missed surveillance to the performing department has been counseled on the importance of ensuring that all WO packages are delivered to the performing department before surveillances are scheduled to be performed
- (2) The supervisor who failed to cross check the work ahead list against the WO packages received has been counseled on the importance of ensuring that all scheduled WO's are performed before their overdue date.
- (3) The managers of all Hope Creek departments which perform Technical Specification surveillances will review this occurrence with their staff and supervisors by October 15, 1987. This review will include instructions on their responsibilities to ensure that sufficient cross-checking of completed surveillances against the work-ahead lists is performed.

Sincerely,  
  
 S. LaBruna  
 General Manager -  
 Hope Creek Operations

AME:

SORC Mtg. 87-128



Public Service Electric and Gas Company P.O. Box L Hancocks Bridge, New Jersey 08038  
Hope Creek Operations

September 16, 1987

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

HOPE CREEK GENERATING STATION  
DOCKET NO. 50-354  
UNIT NO. 1  
LICENSEE EVENT REPORT 87-038-00

This Licensee Event Report is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(i).

Sincerely,

A handwritten signature in cursive script, appearing to read "S. LaBruna".

S. LaBruna  
General Manager -  
Hope Creek Operations

AME:

Attachment  
SORC Mtg. 87-128

C Distribution