



Log # TXX-6723  
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Ref: 10CFR50.55(e)

William G. Council  
Executive Vice President

September 14, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

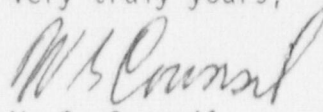
SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
COMPONENT COOLING WATER (CCW) ISOLATION FOLLOWING REACTOR  
COOLANT PUMP (RCP) THERMAL BARRIER RUPTURE  
SDAR: CP-87-29 (INTERIM REPORT)

Gentlemen:

On June 29, 1987 we verbally notified your Mr. Shannon Phillips of a deficiency involving instrumentation and controls utilized for isolation of Component Cooling Water (CCW) to the Reactor Coolant Pump (RCP) thermal barrier heat exchanger. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report logged was TXX-6608, dated July 29, 1987.

Our evaluation of the effects of this condition is continuing. The next report on this issue will be submitted no later than November 20, 1987.

Very truly yours,

  
W. G. Council

CBC/mlh

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

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