

September 15, 1987

Docket No.: 50-322

Mr. John D. Leonard, Jr.  
Vice President - Nuclear Operations  
Long Island Lighting Company  
Shoreham Nuclear Power Station  
P. O. Box 618, North Country Road  
Wading River, New York 11792

Dear Mr. Leonard:

SUBJECT: FIRST TEN-YEAR INTERVAL INSERVICE INSPECTION PROGRAM

RE: SHOREHAM NUCLEAR POWER STATION, UNIT 1

We are reviewing the Revision 3 of the subject program for compliance with 10 CFR 50.55a. We have determined that additional information is required for us to complete the review. Enclosed is a list of the information required (See Section 2 of the enclosure). Your response is requested by November 7, 1987.

Sincerely,

/s/

Ronnie Lo, Project Manager  
Project Directorate I-2  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
As stated

cc w/enclosure:  
See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script, appearing to read "Ronnie Lo".

Ronnie Lo, Project Manager  
Project Directorate I-2  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
As stated

cc w/enclosure:  
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Mr. John D. Leonard, Jr.  
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## Enclosure

LONG ISLAND LIGHTING COMPANY  
SHOREHAM NUCLEAR POWER STATION UNIT 1  
DOCKET NUMBER 50-322

MATERIALS ENGINEERING BRANCH  
DIVISION OF ENGINEERING AND SYSTEMS TECHNOLOGY

### Request for Additional Information - First 10-Year Interval Inservice Inspection Program Plan

#### 1. Scope/Status of Review

Throughout the service life of a water-cooled nuclear power facility, 10 CFR 50.55a(g)(4) requires that components (including supports) which are classified as American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Class 1, Class 2, and Class 3 meet the requirements, except design and access provisions and preservice examination requirements, set forth in the ASME Code Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," to the extent practical within the limitations of design, geometry, and materials of construction of the components. This section of the regulations also requires that inservice examinations of components and system pressure tests conducted during the initial 120-month inspection interval shall comply with the requirements in the latest edition and addenda of the Code incorporated by reference in 10 CFR 50.55a(b) on the date 12 months prior to the date of issuance of the operating license, subject to the limitations and modifications listed therein. The components (including supports) may meet requirements set forth in subsequent editions and addenda of this Code which are incorporated by reference in 10 CFR 50.55a(b) subject to the limitations and modifications listed therein. The Licensee, Long Island Lighting Company, has prepared the Inservice Inspection (ISI) Program Plan to meet the requirements of the 1980 Edition, Winter 1981 Addenda (80W81) of the ASME Code Section XI except that the extent and frequency of examination for Code Class 1 and 2 piping welds has been determined by the 1974 Edition through Summer 1975 Addenda (74S75).

As required by 10 CFR 50.55a(g)(5), if the licensee determines that

certain Code examination requirements are impractical and relief is requested, the licensee shall submit information to the Nuclear Regulatory Commission (NRC) to support that determination.

The staff has reviewed the available information in the Shoreham Nuclear Power Station Unit 1 First 10-Year Interval Inservice Inspection Program Plan, Revision 3, and the Support and Snubber Inservice Examination and Testing Program Plan, Revision 1, both submitted May 17, 1985 and the requests for relief from the ASME Code Section XI requirements which the Licensee has determined to be impractical.

2. Additional Information Required

Based on the above review, the staff has concluded that the following information and/or clarification is required in order to complete the review of the Inservice Inspection Program Plan:

- A. Verify that Revision 3 is the latest revision of the Shoreham Nuclear Power Station Unit 1 First 10-Year Interval ISI Program Plan. If it is not, the Licensee should submit the latest revision for staff review.
- B. Paragraph 10 CFR 50.55a(b)(2)(iv) requires that ASME Code Class 2 piping welds in the Residual Heat Removal (RHR), Emergency Core Cooling (ECC), and Containment Heat Removal (CHR) systems shall be examined. These systems should not be completely exempted from inservice volumetric examination based on Section XI exclusion criteria contained in IWC-1220. Later editions and addenda of the Code require volumetric examination of Class 2 piping welds greater than or equal to 3/8 inch nominal wall thickness for piping greater than 4 inch nominal pipe size. The staff has previously determined that a 7.5% augmented volumetric sample constitutes an acceptable resolution at similar plants. The staff notes that the Licensee is performing volumetric examination on only 4.5% of the Class 2 piping welds in the RHR system and is not performing volumetric examination on any of the Class 2 piping welds in the Core Spray

system. Verify that volumetric examination will be performed on at least a 7.5% sample of the Class 2 piping welds in the ECC, RHR, and CHR systems and that the ISI Program Plan will be revised to include the 7.5% augmented volumetric sample.

- C. Exceptions to the Section XI ASME Code requirements should be submitted as requests for relief. In the Containment Penetration Flued Heads section of the ISI Program Plan, the Licensee states: "Because of the configuration of the Containment Penetration Flued Heads, the Section XI required surface examination cannot be done. In lieu of the required examinations a surface examination will be done of the outer weld and a leak test will be done of the annulus between the inner and outer weld." Since the Licensee has determined that certain ASME Code Section XI requirements are impractical, a request for relief from those requirements is required for staff review.
- D. The staff notes that there are welds listed in the ISI Program Plan for which the Licensee had previously requested relief during preservice inspection (PSI) but has not requested relief for ISI (e.g. Reactor Pressure Vessel welds). Requests for relief are not automatically granted for subsequent inspection intervals. A complete submittal of the requests for relief must be made for each 10-year inspection interval for the staff to review the requests for relief against the updated Code requirements. Verify that there are no additional relief requests applicable to the first 10-year inspection interval, other than Relief Requests 8613-R1 and 8618-R1. If additional relief requests are required, the Licensee should submit them for staff review.