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SVP-98-100

March 20, 1998

U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Document Control Desk

Subject: Quad Cities Nuclear Power Station Units 1 and 2
Summary Report of Changes, Tests and Experiments Completed
NRC Docket Numbers 50-254 and 50-265

Reference: (a) L. W. Pearce letter (SVP-97-251) to USNRC dated October 31, 1997

The Engineering Summary Report dated October 31, 1997 (Reference (a)), contained an error applicable to Safety Evaluation SE-97-024. This safety evaluation was conducted to implement an interim alternate shutdown method by the addition of two isolation valves. One valve is on the Residual Heat Removal Drywell Spray Line and the other is on the fire header in the Reactor Building. The October 31, 1997 Summary Report incorrectly reported that a reduction to the Margin of Safety was associated with Safety Evaluation SE-97-024 when in fact, there is no reduction to the Margin of Safety associated with this Safety Evaluation.

A corrected page to Safety Evaluation Summary SE-97-0924 is enclosed.

If you have any questions concerning this letter, please contact Mr. Charles Peterson, Regulatory Affairs Manager, at (309) 654-2241, extension 3609.

Respectfully,

David A. Sager
Site Vice President
Quad Cities Station

Attachment A: SE-97-024 DCP 9700026 (Unit 1), DCP 9700038 (Unit 2)
Interim Alternate Shutdown Method

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DESCRIPTION:

The change is to provide an Interim Alternate Shutdown Method (IASM) as a means of cooling the reactor core in the event of a loss of ECCS and feedwater systems and the 125 VDC system due to a severe Turbine Building fire. This alternate shutdown system consists of adding an isolation valve on each of the RHR and FP systems, a fire hose to be connected between the two valves, and a portable 125 VDC supply for the Automatic Depressurization Systems (ADS) valves to depressurize the reactor. The method uses the fire pumps to inject river water into the reactor through the RHR system via the fire hose. The only permanent changes to the plant are the addition of two isolation valves. One valve will be on the RHR drywell spray line and the other will be on the fire header in the Reactor Building. All other equipment required for the IASM will be pre-staged in an appropriate location. Connection of this pre-staged equipment to implement the IASM will be covered by the 50.59 evaluation for the new procedure QCOP 4100-17, "Reactor Core Cooling Using Fire Pumps Via Interim Safe Shutdown Hose Line."

SAFETY EVALUATION SUMMARY:

1. The change described above has been analyzed to determine each accident or anticipated transient described in the UFSAR where any of the following is true:
 - The change alters the initial conditions used in the UFSAR analysis.
 - The changed structure, system or component is explicitly or implicitly assumed to function during or after the accident.
 - Operation or failure of the changed structure, system or component could lead to the accident.

The accidents which meet these criteria are listed below:

Fire	SAR SECTION: 9.5.1
Anticipated Transient Without Scram	SAR SECTION: 15.8
Decrease in Reactor Coolant Inventory	SAR SECTION: 15.6
Decrease in Heat Removal from Reactor Coolant	SAR SECTION: 15.2

For these accidents, it has been determined that the change described above will not increase the probability of an occurrence or the consequence of the accident, or malfunction of equipment important to safety as previously evaluated in the SAR.

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2. The possibility for an accident or malfunction of a different type than any previously evaluated in the UFSAR is not created because the installation of this modification will not adversely impact the RHR and FP systems. The new isolation valves will maintain the mechanical and structural integrity of the piping so that the RHR and FP systems can perform their design functions. This system is designated to provide cooling water to the reactor core in the event of a loss of all ECCs and feedwater systems. This modification is added to existing systems and is manually operated and controlled procedurally per QCOP 4100-17. There are no new accidents or malfunctions postulated by the addition of this modification.

3. The margin of safety, as defined in the basis for any Technical Specification, is not reduced because Section 3/4.7.A requires that leakage rate testing be performed on all primary containment valves. The new valves on the RHR systems will be included in the surveillance to ensure the margin of safety is maintained.