RAR-91-28

July 2, 1991

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of June 1991.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

R. A. Robey
Technical Superintendent

RAR/CALS/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

JUNE 1991

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

TS 14

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Cynthia A. Losek-Short and Debra Kelley, telephone number 309-654-2241, extensions 2938 and 2240.

II. SUMMARY O OPERATING EXPERIENCE

A. Unit One

Unit One remained shutdown from the previous month due to main transformer replacement. The reactor became critical at 1009 hours on the 24th and the unit generator went on-line at 0115 hours on the 26t respectively. The unit remained at full power for the rest of the month with the exception of load drops on the 28th, 29th, and 30th, per Chicago Load Dispatch requests.

B. Unit Two

Unit Two began the month of June operating in Economic Generatic Control (EGC). The unit was taken out of EGC on the 3rd and 16 for Turbine Stop Valve Testing and "C" Reactor Feed Pump Trouble Shooting. The unit was also taken out of EGC on 2nd, 4th, 5th, 13th, 17th, 18th, 19th, 20th, 23rd, 24th, and 25th, per Chicago Load Dispatch requests.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Technical Specifications for the reporting period.

An Amendment to the Facility License was that the Offsite Dose Calculation Manual (ODCM) was changed and is reportable to the Nuclear Regulatory Commission (NRC) in accordance with station Technical Specifications. This document contains the models for the public dose assessment from gaseous effluents, liquid discharges, and direct radiation.

The NRC, per Generic Letter 89-01, requested that the information found in the station Radiological Effluent Technical Specifications to be transferred to the ODCM and Process Control Program (PCP), as applicable. The major changes to the ODCM complied with this request. The remaining minor changes were editorial, outstanding from the previous distribution, Revision 0, dated March, 1989.

This revision does not reduce the accuracy and reliability of dose calculations or set point determination methodology.

Any changes to the ODCM computer program required by this document are in the process of being implemented now and should be fully available by July 8, 1991.

MAJOR CHANGES

The generic manual both introduces and references the Radiological Effluent Technical Specifications contained in the site annexes, in Chapter 12, now titled the "Radiological Effluent Technical Standards (RETS)". "Operability Requirements" in the RETS are equivalent to Limiting Conditions for Operations in the station Technical Specifications.

The generic manual is common for six sites, therefore the reporting requirement for "Changes to the ODCM" deviates slightly from the recommendation given in the Generic Letter. An effective date for generic manual implementation is to specified in the station Onsite Review. As in the past, the official transmittal to the NRC is made through the stations' Monthly Operating Report within 90 days of the effective date.

MINOR CHANGES

The remaining minor changes are editorial. Below is the abbreviated version of an itemized list of changes available in Emergency Preparedness in at the Corporate Office:

- Revised titles for the Corporate Office Department and Positions
- Prepared a stand alone section for the EPA's Clean Air Act (if and when enacted)
- Deleted references to Commonwealth Edison (or CECo)
- Added text regarding the Wilmington Public Water Supply nearby Braidwood Station
- Corrected parameter symbols, values and units.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q93249	6601	Investigate and repair 1/2 D.G. which tripped high engine temperature.	As Found: Tested thermostats and two did not open to the full 3/4" position. They all stayed closed at 160 degrees and the thermostat seals were brittle and cracked.As Left: Installed and tested 9 new thermostats and they were found to work properly. Cleaned all gasket surfaces and holes for thermostats, also ran brushes thru all the tubes.
Q80309	7504	Investigate and repair air in leakage around door when Standby Gas Treatment is run.	Removed and replaced gasket material around access doors, as well as, adjusted door latches.
Q92973	0901	LPRM found with high light on and no alarm on 901-5 panel. Investigate this occurrence since several LPRMS have done this in the past.	As Found: Found all alarms and alarms indications to be reset. As Left: Notified SCRE of jumpering alarm points on the 901-34 and 901-37 panel. NSO got alarm and cleared it and repeated with same results. No problem was found. Functionally tested high alarm for LPRM 48-49A verified hi light on 901-37 panel came up and alarm D7 on 901-5 panel came up and alarm light on full core display. Reset alarms and put everything back to normal.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	EID DESCRIPTION	WORK PERFORMED
Q93330	3941	Investigate adnormal readings from U2 DG CWP Flow Meter 2-3941-26.	As Found: Indicator was reading 1200 GPM Flow. When valve out and equalized the reading dropped to 0 GPM. As Left: Backfilled sensing lines and checked calibration which was in tolerance. Performed final calibration and returned instrument to service.
Q93248		Investigate 1/2 DG tripped on high temperature suspect a bad temperature switch.	As Found: Temperature switches 1/2-6641=ETS and 1/2-6641-TC would not calibrate to whithin tolerance. As Left: Installed and calibrated new temperature switches for 1/2-6641-ETS and 1/2-6651-TC. Alsc, functionaly tested the 1/2-6641-ETS and 1/2-6642-TV for local panel annunciator alarms.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT 1

Licensee Event Report Number	Date	Title of Occurrence
91-010*	04-27-91	Plant Shutdown due to RCIC INOP Ref. T.S 3.5.E.4.
91-010	06-14-91	1/2 Scram on Loss of A 24/48 VDC (near Miss).
91-013	05-14-91	Control Room Smoke Detectors are below the minimum.

UNIT 2

There were no licensee event reports for Units 2 for this reporting period.

* 91-010 Report for the Month of April has been downgraded.

V. DATA TABU'ATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-254
Unit One
Date July 1, 1991
Completed By Cynthia Short
Telephone 309-654-2241

MONTH APRIL

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	-7	17	-7
2	-8	18	-7
3	-8	19	-7
4	-7	20	-7
5	-7	21	-7
6	-7	22	-7
7	-7	23	-7
8	-7	24	-7
9	-7	25	-7
10	-7	26	564
11	-7	27	783
12	-7	28	699
13	-7	29	727
14	-7	30	759
15	-7	31	
16	-7		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-265
Unit Two
Date July 1, 1991
Completed By Cynthia Short
Telephone 309-654-2241

MONTH APRIL

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	722	17	675
2	640	18	702
3	688	19	707
4	678	20	721
5	702	21	761
6	762	22	707
7	783	23	663
8	717	24	688
9	747	25	691
10	737	26	738
11	741	27	727
12	728	28	778
13	713	29	745
14	759	30	707
15	739	31	
16	525		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

NOTE: CORRECTIONS FOR JUNE

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-254
Unit One
Date June 3, 1991
Completed By Cynthia Short
Telephone 309-654-2241

MONTH APRIL

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	146	17	792
2	292	18	794
3	409	19	455
4	666	20	736
5	670	21	676
6	673	22	3
7	749	23	-7
8	800	24	-7
9	798	25	-7
10	798	26	-7
11	798	27	-7
12	796	28	-7
13	796	29	-7
14	796	30	-7
15	794	31	-7
16	793		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

NOTE: CORRECTIONS FOR JUNE

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-265
Unit Two
Date June 3, 1991
Completed By Cynthia Short
Telephone 309-654-2241

MONTH APRIL

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	752	17	745
2	744	18	730
3	732	19	719
4	707	20	764
5	648	21	770
6	754	22	780
7	740	23	773
8	738	24	766
9	754	25	731
10	754	26	730
11	732	27	688
12	735	28	760
13	774	29	763
14	772	30	750
15	771	31	768
16	764		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX C

OPERATING DATA REPORT

DOCKET NO 50-254

UNIT One

DATE July 9, 1991

COMPLETED BY Cynthia Losek-Short

TELEPHONE 309-654-2241

OPERATING STATUS 0000 060191

1. REPORTING PERIOD: 2400 063191 GROSS HOURS IN REPORTING PERIOD: 720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY (MWe-Net): 769
   DESIGN ELECTRICAL RATING (MWe-Net): 789
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	157.8	779.0	132260.9
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3241.9
7. HOURS GENERATOR ON LINE	118.8	652.3	128021.6
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	909.2
9. GROSS THERMAL ENERGY GENERATED (MWH)	277555.0	1390411.0	274121851.0
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	87684.0	444402.0	88838256.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	80310.0	401844.0	83576085.0
12. REACTOR SERVICE FACTOR	21.9	17.9	78.5
13. REACTOR AVAILABILITY FACTOR	21.9	17.9	80.5
14. UNIT SERVICE FACTOR	16.5	15.0	76.0
15. UNIT AVAILABILITY FACTOR	16.5	15.0	76.5
16. UNIT CAPACITY FACTOR (Using MDC)	14.5	12.0	64.5
17. UNIT CAPACITY FACTOR (Using Design MWe)	14.1	11.7	62.9
18. UNIT FORCED OUTAGE RATE	0.0	0.0	5.7

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

APPENDIX C

OPERATING DATA REPORT

DOCKET NO 50-265

UNIT Two

DATE July 9, 1991

COMPLETED BY Cynthia Losek-Short

TELEPHONE 309-654-2241

OPERATING STATUS 0000 060191

1. REPORTING PERIOD: 2400 063091 GROSS HOURS IN REPORTING PERIOD: 720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY (MWe-Net): 769
   DESIGN ELECTRICAL RATING (MWe-Net): 789
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	3887.0	129576.2
6. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	2985.8
7. HOURS GENERATOR ON LINE	720.0	3856.5	126143.5
8. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	702.9
9. GROSS THERMAL ENERGY GENERATED (MWH)	1647852.0	8994100.0	272225421.0
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	531554.0	2918681.0	87379888.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)	513892.0	2824038.0	82654590.0
12. REACTOR SERVICE FACTOR	100.0	89.5	77.6
13. REACTOR AVAILABILITY FACTOR	100.0	89.5	79.4
14. UNIT SERVICE FACTOR	100.0	88.8	75.6
15. UNIT AVAILABILITY FACTOR	100.0	88.8	76.0
16. UNIT CAPACITY FACTOR (Using MDC)	92.8	84.5	64.4
17. UNIT CAPACITY FACTOR (Using Design MWe)	90.5	82.4	62.8
18. UNIT FORCED OUTAGE RATE	0.0	11.0	7.9

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

NOTE: CORRECTIONS FOR JUNE

APPENDIX C
OPERATING DATA REPORT

Docket No. 50-254
Unit One
Date June 3, 1991
Completed By Cynthia Losek-Short
Telephone 309-654-2241

OPERATING STATUS

1. Reporting Period 0000 050191 / 2400 053191 Gross Hours in Report Period: 744

2. Currently Authorized Power Level (MWt): 2511 Max. Depend. Capacity (MWe-Net): 769
Design Electrical Rating (MWe-Net): 789

3. Power Level to Which Restricted (If Any) (MWe-Net): N/A

4. Reasons For Restriction (If any):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. Number of Hours Reactor Was Critical	505.6	621.2	132103.1
6. Reactor Reserve Shutdown Hours	0.0	0.0	3421.9
7. Hours Generator On Line	505.6	533.5	127902.8
8. Unit Reserve Shutdown Hours	0.0	0.0	909.2
9. Gross Thermal Energy Generated (MWh)	1079784.0	1112856.0	273844296.0
10. Gross Electrical Energy Generated (MWh)	353533.0	356718.0	88750572.0
11. Net Electrical Energy Generated (MWh)	340254.0	321574.0	83495775.0
12. Reactor Service Factor	68.0	17.1	78.8
13. Reactor Availability Factor	68.0	17.1	80.8
14. Unit Service Factor	68.0	14.7	76.3
15. Unit Availability Factor	68.0	14.7	76.8
16. Unit Capacity Factor (Using MDC)	59.5	11.5	64.7
17. Unit Capacity Factor (Using Design MWe)	58.0	11.2	63.1
18. Unit Forced Outage Rate	0.0	0.0	5.3

19. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

20. If Shut Down at End of Report Period, Estimated Date of Startup: _____

21. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	_____	_____
Initial Electricity	_____	_____
Commercial Operation	_____	_____

NOTE: CORRECTIONS FOR JUNE

APPENDIX C
OPERATING DATA REPORT

Docket No. 50-265
Unit Two
Date June 3, 1991
Completed By Cynthia Losek-Short
Telephone 309-654-2241

OPERATING STATUS

1. Reporting Period 0000 050191 / 2400 053191 Gross Hours in Report Period: 744

2. Currently Authorized Power Level (MWt): 2511 Max. Depend. Capacity (MWe-Net): 769
Design Electrical Rating (MWe-Net): 789

3. Power Level to Which Restricted (If Any) (MWe-Net): N/A

4. Reasons For Restriction (If any):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. Number of Hours Reactor Was Critical	744.0	3167.0	108856.2
6. Reactor Reserve Shutdown Hours	0.0	0.0	2985.9
7. Hours Generator On Line	744.0	3136.5	125423.5
8. Unit Reserve Shutdown Hours	0.0	0.0	702.9
9. Gross Thermal Energy Generated (MWh)	1748947.0	7346248.0	270577569.0
10. Gross Electrical Energy Generated (MWh)	573640.0	2406206.0	86867413.0
11. Net Electrical Energy Generated (MWh)	554986.0	2328594.0	82159146.0
12. Reactor Service Factor	100.0	87.4	77.5
13. Reactor Availability Factor	100.0	87.4	79.3
14. Unit Service Factor	100.0	86.5	75.5
15. Unit Availability Factor	100.0	86.5	75.9
16. Unit Capacity Factor (Using MDC)	97.0	83.6	64.3
17. Unit Capacity Factor (Using Design MWe)	94.5	81.4	62.7
18. Unit Forced Outage Rate	0.0	13.1	8.0

19. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

20. If Shut Down at End of Report Period, Estimated Date of Startup: _____

21. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	_____	_____
Initial Electricity	_____	_____
Commercial Operation	_____	_____

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME QUAD CITIES UNIT 1

COMPLETED BY CYNTHIA A. LOSEK-SHORT

DATE JULY 1, 1991

REPORT MONTH JUNE

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
91-02	91 501	F	562.2	B	2	- - - - -	- -	- - - - -	UNIT CONTINUED TO BE SHUTDOWN DUE TO MAIN TRANSFORMER PROBLEM.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME QUAD CITIES UNIT 2

COMPLETED BY CYNTHIA A. LOSEK-SHORT

DATE JULY 1, 1991

REPORT MONTH JUNE

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
91-08	910602	S	8.1	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-09	910603	S	6.3	B	5	- - - -	-	- - -	LOAD DROP FOR TURBINE VALVE TESTING
91-10	910604	S	7.5	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-11	910605	S	5.3	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-12	910613	S	3.9	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-13	910616	S	8.3	B	5	- - - -	-	- - -	LOAD DROP FOR "C" RFP TROUBLE SHOOTING
91-14	910617	S	11.9	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-15	910618	S	5.8	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-16	910619	S	5.8	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-17	910620	S	3.6	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-18	910623	S	8.1	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-19	910624	S	6.5	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH
91-20	910625	S	6.0	H	5	- - - -	-	- - -	LOAD DROP REQUESTED PER CHICAGO LOAD DISPATCH

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

There was no Control Rod Drive scram timing data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q1 Reload: 11 Cycle: 12

2. Scheduled date for next refueling shutdown: 9-5-92

3. Scheduled date for restart following refueling: 12-5-92

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

NOT AS YET DETERMINED.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1405

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

APPROVED
OCT 30 1989
Q.C.O.S.R.

14/0395t

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q2 Reload: 10 Cycle: 11

2. Scheduled date for next refueling shutdown: 12-28-91

3. Scheduled date for restart following refueling: 3-7-92

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   NOT AS YET DETERMINED.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 2287

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

APPROVED
OCT 30 1989
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VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM - Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI - American National Standards Institute
APRM - Average Power Range Monitor
ATWS - Anticipated Transient Without Scram
BWR - Boiling Water Reactor
CRD - Control Rod Drive
EHC - Electro-Hydraulic Control System
EOF - Emergency Operations Facility
GSEP - Generating Stations Emergency Plan
HEPA - High-Efficiency Particulate Filter
HPCI - High Pressure Coolant Injection System
HRSS - High Radiation Sampling System
IPCLRT - Integrated Primary Containment Leak Rate Test
IRM - Intermediate Range Monitor
ISI - In-service Inspection
LER - Licensee Event Report
LLRT - Local Leak Rate Test
LPCI - Low Pressure Coolant Injection Mode of RHRs
LPRM - Local Power Range Monitor
MAPLHGR - Maximum Average Planar Linear Heat Generation Rate
MCPR - Minimum Critical Power Ratio
MFLCPR - Maximum Fraction Limiting Critical Power Ratio
MPC - Maximum Permissible Concentration
MSIV - Main Steam Isolation Valve
NIOSH - National Institute for Occupational Safety and Health
PCI - Primary Containment Isolation
PCIOMR - Preconditioning Interim Operating Management Recommendations
RBCCW - Reactor Building Closed Cooling Water System
RBM - Rod Block Monitor
RCIC - Reactor Core Isolation Cooling System
RHRS - Residual Heat Removal System
RPS - Reactor Protection System
RWM - Rod Worth Minimizer
SBGTS - Standby Gas Treatment System
SBLC - Standby Liquid Control
SDC - Shutdown Cooling Mode of RHRS
SDV - Scram Discharge Volume
SRM - Source Range Monitor
TBCCW - Turbine Building Closed Cooling Water System
TIP - Traversing Incore Probe
TSC - Technical Support Center