July 12, 1991
LIC-91-190R

Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Reference: Docket No. 50-285

Gentlemen:

SUBJECT: June Monthly Operating Report (MOR)

Please find enclosed the June 1991 Monthly Operating Report for the Fort Calhoun Station Unit No. 1 as required by Technical Specification Section 5.9.1.

Also enclosed is a revision to the "Operating Data Report" section of the May 1991 MOR. While preparing the May MOR, an error occurred during data entry which affected results in the Year-To-Date and Cumulative columns. However, the Monthly calculations were correct. Appropriate corrective actions to prevent recurrence have been taken.

If you should have any questions, please contact me.

Sincerely,

W. G. tes
Division Manager
Nuclear Operations

WGG/sel

Enclosures

c: LeBoeuf, Lamb, Leiby & MacRae
R. D. Martin, NRC Regional Administrator, Region IV
R. P. Mullikin, NRC Senior Resident Inspector
D. K. Sentell, Combustion Engineering
R. J. Simon, Westinghouse
Office of Management & Program Analysis (2)
INPO Records Center
American Nuclear Insurers

IE24
1/1

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	Fort Calhoun Station
DATE	July 8, 1991
COMPLETED BY	M.L. Edwards
TELEPHONE	(402)636-2451

MONTH June 1991

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	336	17	449
2	339	18	448
3	336	19	433
4	336	20	237
5	338	21	423
6	340	22	461
7	341	23	470
8	341	24	473
9	341	25	471
10	341	26	468
11	337	27	465
12	334	28	463
13	333	29	462
14	334	30	462
15	335		
16	380		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO 50-285
DATE 07/10/91
COMPLETED BY ML Edwards
TELEPHONE (402) 636-2451

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: June 1991
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	4,343.0	155,713.0
12. Number Of Hours Reactor Was Critical	720.0	4,163.0	120,951.7
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	720.0	4,152.2	119,582.1
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	912,096.5	4,874,757.6	156,158,246.9
17. Gross Electrical Energy Generated (MWH)	294,926.0	1,604,632.0	51,354,758.2
18. Net Electrical Energy Generated (MWH)	279,012.6	1,514,441.6	48,999,217.9
19. Unit Service Factor	100.0	95.6	76.8
20. Unit Availability Factor	100.0	95.6	76.8
21. Unit Capacity Factor (Using MDC Net)	81.1	73.0	65.8
22. Unit Capacity Factor (Using DER Net)	81.1	73.0	65.8
23. Unit Forced Outage Rate	0.0	4.4	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending June 1991

1. Scheduled date for next refueling shutdown. — January 31, 1992

2. Scheduled date for restart following refueling. — May 1, 1992

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be? — Incorporate specific requirements resulting from reload safety analysis.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — N/A

   c. If no such review has taken place, when is it scheduled? — N/A

4. Scheduled date(s) for submitting proposed licensing action and support information. — November 1991

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — New fuel supplier; New LOCA Analysis

6. The number of fuel assemblies:
   a) in the core — 133 Assemblies
   b) in the spent fuel pool — 477 Assemblies
   c) spent fuel pool storage capacity — 729 Assemblies
   d) planned spent fuel pool storage capacity — Planned to be increased with higher density spent fuel racks

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1995 *

* Capability of full core offload of 133 assemblies lost.

Prepared by Kevin Holthaus Date 7-8-91

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE July 9, 1991
COMPLETED BY M.L. Edwards
TELEPHONE (402) 636-2451

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June 1991

No.	Date	Type (1)	Duration (Hours)	Reason (2)	Method of Shutting Down Reactor (3)	Licensee Event Report #	System Code (4)	Component Code (5)	Cause & Corrective Action to Prevent Recurrence
91-03	910619	F	0	A	4	N/A	WG	VALVEX	Power level reduced to 55% to repair condenser waterbox valves. Repaired a failed shaft coupling and valve seat.

1 F-Forced
S-Scheduled

2 Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administative
G-Operational Error
H-Other (Explain)

3 Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit 1 - Same Source

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

June 1991
Monthly Operating Report

I. OPERATIONS SUMMARY

A. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

Fort Calhoun Station operated at 75% power until June 16, 1991, when power was raised to 98% to meet seasonal demand. On June 19, 1991, power was reduced to 55% for repair of condenser waterbox valves. Following repairs to these valves on June 20, 1991, a power increase to 100% was initiated.

On June 20, 1991, a gas pressure switch for house service transformer T1A-4 was tripped due to technician error, locking out 161KV offsite power. The transfer to the 345KV system occurred without incident. After investigation of the cause, the switch was reset and 161KV power was restored.

During June 1991, a hot weather test was performed on emergency diesel generator DG-1. The test results in conjunction with a detailed system analysis were used to support raising the ambient temperature operating limits on emergency diesel generators DG-1 and DG-2 to 110°F.

The following NRC Inspections took place in June 1991:

IR 91-09	Corrective Action
IR 91-16	Resident's Periodic Inspection
IR 91-17	Radiation Protection
IR 91-18	Security

The following LERs were submitted:

LER-91-04 R1 Offsite Power Low Signal Outside Design Basis

B. RESULTS OF LEAK RATE TESTS

The results of the reactor coolant leak rate tests for June 1991, indicate the the reactor coolant system (RCS) and chemical and volume control system (CVCS) are both relatively leak tight. Total RCS leakage averaged about 0.12 gpm during June.

The maximum leak rate for the month was recorded on June 20, 1991 when the total leak rate was 1.824 gpm and the unknown leak rate was 1.757 gpm. The total leak rate dropped to 0.012 gpm and the unknown leak rate dropped to -0.054 gpm when the test was repeated. The plant was undergoing power changes on June 20, 1991 which are believed to have affected leak rate test accuracy.

C. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
None	

D. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF JUNE 1991

The raw water inlet valve (HCV-2883A) for component cooling heat exchanger AC-1D would not shut using either the control room or the local switch. A malfunction of the relay coil due to heat was the apparent cause of failure. The relay coil was replaced.

Steam generator RC-2B pressure channel pressure indication controller (A/PIC-905) power supply failed due to the age of the component. The power supply was replaced.

The component cooling water outlet valve solenoid valve (HCV-478-20) for spent fuel pool heat exchanger AC-8 developed an air leak. The possible cause of failure was aging/cyclic fatigue and the solenoid valve was replaced.

Maintenance activities on charging pump CH-1C included:

- Replaced plunger alignment parts in the gear box.
- Overhauled the hydraulic end of the pump.
- Installed new style packing.
- Disassembled, cleaned, inspected, replaced bellows gaskets, seat and disc on the discharge relief valve (CH-181).

Maintenance activities on Diesel Generator #1 included:

- Replaced jacket water thermostats and O-rings.
- Replaced the glycol solution with water and rust inhibitor.

Representatives from the Furmanite Company were brought in to reinject the leaking inlet flange on pressurizer (RC-4) relief valve RC-142.

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE 07/10/91
COMPLETED BY MLEdwards
TELEPHONE (402) 636-2451

OPERATING STATUS

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2. Reporting Period: May 1991
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

N/A

Notes

Revised on July 10, 1991 because of errors in Year-to-date and cumulative columns. Lines revised are marked by "/".

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative	
11. Hours In Reporting Period	744.0	3,623.0	154,993.0	/
12. Number Of Hours Reactor Was Critical	744.0	3,443.0	120,231.7	/
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5	
14. Hours Generator On-Line	744.0	3,432.2	118,862.1	/
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0	
16. Gross Thermal Energy Generated (MWH)	838,257.3	3,962,661.1	155,246,150.4	/
17. Gross Electrical Energy Generated (MWH)	273,060.0	1,309,706.0	51,059,832.2	/
18. Net Electrical Energy Generated (MWH)	256,829.8	1,235,429.0	48,720,205.3	/
19. Unit Service Factor	100.0	94.7	76.7	/
20. Unit Availability Factor	100.0	94.7	76.7	/
21. Unit Capacity Factor (Using MDC Net)	72.2	71.3	65.8	/
22. Unit Capacity Factor (Using DER Net)	72.2	71.3	65.8	/
23. Unit Forced Outage Rate	0.0	5.3	3.6	/

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			