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Company

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June 29, 1987
NLR-N87117

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

INSERVICE INSPECTION PROGRAM
REPLY TO NRC REQUEST FOR ADDITIONAL INFORMATION
HOPE CREEK GENERATING STATION
DOCKET NO. 50-354

Attached please find six (6) copies of the Hope Creek Generating Station (HCGS) Inservice Inspection Program Long Term Plan, herein referred to as the Plan (Enclosure 2). This information is provided in response to your Request for Additional Information dated March 6, 1987 as amended by a conference call held on April 22, 1987 between PSE&G ISI and Licensing and Regulation personnel and Messrs. G. Rivenbark and M. Hum of your staff along with Mr. B. Brown of EG&G Idaho.

In addition to the requested information, the enclosed Plan also contains all the information previously submitted to the NRC in the original ISI submittal dated October 10, 1986. As such, the enclosed Plan supercedes the initial submittal.

Our responses to the specific items of the RAI are discussed below.

A. Isometric and/or Component Drawings

Inservice Inspection Boundary Diagrams which were previously transmitted on December 8, 1986, have also been provided in the Plan (Attachment 5-1).

Examination Figures for Nuclear Class 1 and 2 components and piping have been provided and are included in the Plan (Attachment 5-14, Appendices A & B).

A component Support Allocation Table has been provided in the Plan (Attachment 5-12) as agreed upon in lieu of System Isometric Drawings.

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SEE ATTACHED DIST: OF ENCL

B. List of Systems and Acronyms

This information has been provided in the Plan, pages 3 and 4.

C. List of Exemptions by Code Class

This information has been provided in Section 2 of the Plan.

Specific component and piping line exemption criteria are shown in the appropriate boundary tables which are provided in the Plan (Attachments 5-3 and 5-8).

D. Line Lists for each system

Line lists are part of the appropriate boundary tables (Attachments 5-3 and 5-8) of the Plan which contain the requested information.

E. Exam Tables for each system

This information has been provided in the Plan as follows:

Class 1 Component Examination Tables - Attachment 5-4.

Class 2 Component Examination Tables - Attachment 5-6.

Class 3 Component Examination Table - Attachment 5-9.

Component Supports Examination Table - Attachment 5-11.

A list of procedures has been provided separately as Enclosure 1 to this letter. The procedure titles adequately describe the examinations performed by the various procedures.

F. Summary Tables

This information has been provided in the Plan as follows:

Class 1 Allocation Tables - Attachment 5-5.

Class 2 Allocation Tables - Attachment 5-7.

Class 3 Allocation Tables - Attachment 5-11.

The ASME categories are identified in these allocation tables. However, the ASME Item Numbers are not specifically identified for the following reasons:

- The item description was used where appropriate in place of ASME Item Number to more clearly describe the type of component or weld.

- Piping allocations where the percentages applied, were further separated by system and pipe sizes to ensure representative selections among the sizes.

- Multiple ASME Item Numbers containing the identical examination requirement, apply to most individual component supports. Listing of these component supports by unique ASME Item Number would have required listing those supports several times.

G. List of NDE Procedures

As stated above, a list of procedures has been provided separately as Enclosure 1 to this letter. This list was not included in the Plan (Enclosure 2) because the procedures are subject to revision. Specific cycle (outage) examination plans will include procedure references in the examination table listing.

H. List of Calibration Blocks

This information has been provided in the Plan, Section 2.5, with the examinations included in the appropriate table, (Attachment 5-4, 5-6 and 5-9).

J. Appendix J

Containment leakage rate testing is outside the scope of the Inservice Inspection Program. Containment and Isolation Valve leakage testing is performed in accordance with 10 CFR 50, Appendix J, and the Hope Creek Technical Specifications, Section 4.6.1.2.

K. Relief Requests

The requests for relief from code requirements based on examination limitations have been revised to delete those components not selected for this First Ten Year Interval. The limitations have been revised to include estimated coverage percentages against ISI requirements, revised basis for relief, and to show alternate examinations where applicable.

If additional information is required, please do not hesitate to contact us.

Sincerely,

Stan E Miltenberger
for CAM

Enclosure

Enclosure 1 - List of NDE Procedures expected to be used during the First Ten Year Interval.

Enclosure 2 - Hope Creek Generating Station
Inservice Inspection Program
Long Term Plan - First Ten Year Interval
June 1987, Volumes A, B and C.

C Mr. G. W. Rivenbark (w/o enclosure)
USNRC Licensing Project Manager

Mr. R. W. Borchardt (w/o Enclosures)
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ENCLOSURE 1
LIST OF NDE PROCEDURES
EXPECTED TO BE USED
DURING THE FIRST 10 YEAR INTERVAL

PROCEDURE NUMBER	PROCEDURE TITLE	EXAMINATION METHOD
SwRI-NDT-200- 1	Liquid Penetrant Examination, Color Contrast Method	PT
SwRI-NDT-300- 1	Dry Powder Magnetic Particle Examination	MT
SwRI-NDT-300- 2	Fluorescent Magnetic Particle Examination	MT
SwRI-NDT-600- 5	Manual Ultrasonic Examination of Nuclear Reactor Pressure Vessel Flange Ligaments	UT
SwRI-NDT-600-11	Manual Ultrasonic Examination of Vessel-to-Nozzle Inner Radius Sections	UT
SwRI-NDT-600-15	Manual Ultrasonic Examination of Pressure Vessel Welds	UT
SwRI-NDT-600-30	Manual Ultrasonic Examination of Pressure-Retaining Welds in Heat Exchangers	UT
SwRI-NDT-600-31	Manual Ultrasonic Examination of Austenitic Pressure Piping Welds	UT
SwRI-NDT-600-41	Manual Ultrasonic Examination of Ferritic Pressure Piping Welds	UT
SwRI-NDT-700- 6	Mechanized Ultrasonic Examination of Ferritic Vessels Greater than 2.0 Inches in Thickness	MECH UT
SwRI-NDT-800- 20	Manual Ultrasonic Examination of Pressure-Retaining RPV Studs 2 In. and Greater in Diameter Containing Access Holes at Hope Creek	UT
SwRI-NDT-800- 36	Manual Ultrasonic Examination of Austenitic Thin Wall Piping Welds	UT
SwRI-NDT-800- 55	Manual Ultrasonic Examination of Nozzle Inside Radius Sections from the Vessel Base Material	UT

ENCLOSURE 1
LIST OF NDE PROCEDURES
EXPECTED TO BE USED
DURING THE FIRST 10 YEAR INTERVAL

PROCEDURE NUMBER	PROCEDURE TITLE	EXAMINATION METHOD
SwRI-NDT-800- 97	Manual Ultrasonic Examination of Pump Studs with Partially Drilled Heater Holes at Hope Creek	UT
SwRI-NDT-800-100	Manual Ultrasonic Examination of Corrosion-Resistant Clad Piping Welds at Hope Creek	UT
SwRI-NDT-800-104	Inner Surface Examination of the Access Holes in Pressure Retaining Studs 2 Inches and Greater in Diameter	UT
SwRI-NDT-800-107	Automated Examination of Pressure Piping Welds	UT
M9-IIP-02H (M9-IIP-201)	Visual Examination of Class 1, 2 and 3 Component and Pipe Supports	VT-3 VT-4
M9-IIP-01H (M9-IIP-202)	VT-1 Visual Examination of Nuclear Class 1 and 2 Bolting and Internal Surfaces	VT-1
M9-IIP-03H (M9-IIP-203)	System Leakage Test Nuclear Class 1 Components	VT-2
M9-IIP-04H (M9-IIP-203A)	Service Pressure Leak Exam. Nuclear Class 2 and 3 Components	VT-2
M9-IHP-01H (later)	Inservice Inspection Long Term Plan Hydrostatic Tests General Procedure Hope Creek Generating Station	VT-2
VENDOR (later)	Visual Examination of Reactor Vessel Accessable Interior Welds, Surfaces and Areas	VT-1 VT-3

HOPE CREEK UOLS. A, B+C

ENCL TO: GEORGE RIVENBARK - 1 CY
REG FILE - ORIGINAL
LPDR - 1 CY
PDR - 1 CY
ACRS - 1 CY
NSIC - 1 CY