Wisconsin Electric
POWER COMPANY

Point Beach Nuclear Plant
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NPL 99-0555

September 29, 1999

Document Control Desk
U.S. NUCLEAR REGULATORY COMMISSION
Mail Station P1-137
Washington, DC 20555

Ladies/Gentlemen:

DOCKETS 50-266 AND 50-301
SUBMITTAL OF ADDITIONAL INFORMATION
GENERIC LETTER 92-01, REVISION 1, SUPPLEMENT 1,
"REACTOR VESSEL STRUCTURAL INTEGRITY"
POINT BEACH NUCLEAR PLANT UNITS 1 AND 2

In a NRC letter to Wisconsin Electric (WE) dated August 4, 1999, the NRC closed out WE's actions on NRC Generic Letter 92-01, Revision 1, Supplement 1 (reference B. A. Wetzel (NRC) letter to M. B. Sellman (WE), TAC NOS. MA0564 and MA0565). The letter stated that potential discrepancies related to the fluences assigned for the surveillance capsule data (weld wire heat 61782) in the BAW-2325, Revision 1 report (topical report for B&W fabricated reactor pressure vessel welds) were discovered during the NRC Staff review of R.E. Ginna. The letter recommended that WE perform a review of this information contained in the Reactor Vessel Integrity Database (RVID) and inform the NRC by September 1, 1999 if any discrepancies are noted. The NRR Project Manager (B. A. Wetzel) for Point Beach Nuclear Plant (PBNP) agreed to extend this date to October 1, 1999.

Discussion:

WE has performed a review of the information contained in the RVID and the latest R.E. Ginna best estimate fluence values for their surveillance program. This information was used to perform a reevaluation of the chemistry factor for weld wire heat 61782 pursuant to 10 CFR 50.61 and Regulatory Guide 1.99, Rev. 2. The chemistry factor evaluation includes an adjustment to account for differences in chemical composition (i.e., copper and nickel contents) and irradiation environment (i.e., irradiation temperature) between the capsules and the vessel being assessed (i.e., R.E. Ginna surveillance capsules versus Point Beach Unit 1 weld metal). This evaluation calculated a chemistry factor for weld wire heat 61782 for Point Beach Unit 1 that differs slightly from that assigned in RVID Version 2. The evaluation was performed based on the guidance provided by the NRC Staff in a meeting with industry representatives on November 13, 1997.

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It should be noted that the 30 ft-lb transition temperature shift values for weld wire heat 61782 used in the evaluation for Point Beach differ from the values reported for R.E. Ginna in the ..VID-2 database. This difference is the result of a reevaluation of the R.E. Ginna surveillance data performed for the B&W Owners Group by Framatome Technologies, Inc. in 1998. This reevaluation uses a hyperbolic tangent curve fit to consistently plot surveillance data for all surveillance capsules and was transmitted to the NRC by a letter from D. L. Howell, Framatome Technologies, to the Document Control Desk dated February 2, 1999.

Included as an attachment to this letter is a WE calculation that provides an evaluation of the new surveillance data for assessing the integrity of the PBNP Unit 1 reactor vessel.

If you have any questions on this submittal or require further information, please contact us.

Sincerely,

A. J. Cayia
Manager,
Regulatory Services & Licensing

MAW/tat

Attachments

cc: NRC Regional Administrator
NRC Resident Inspector
NRC Project Manager
PSCW

ADDITIONAL INFORMATION REGARDING REACTOR PRESSURE VESSEL INTEGRITY AT POINT BEACH NUCLEAR PLANT UNIT 1

References:

1) B&W Owners Group Report "Response to Request for Additional Information Regarding Reactor Pressure Vessel Integrity," BAW-2325, Rev. 1, January 1999 (submitted to NRC by B&WOG on February 2, 1999).

2) WE Calculation Addendum 98-0156-00-A, "Evaluation of New Surveillance Data on Chemistry Factor for Weld Wire Heat 61782, Point Beach Unit 1," 9/22/1999 (attached).

3) NRC Reactor Vessel Integrity Database, Version 2, June 1999.

In accordance with the provisions of Generic Letter 92-01, Revision 1, Supplement 1, Wisconsin Electric Power Company (WE) has performed a re-evaluation of reactor pressure vessel (RPV) weld chemistry values previously submitted as part of the current licensing basis for Point Beach Nuclear Plant Unit 1.

The latest known best-estimate chemistry information provided in Reference 1 has been utilized for beltline weld materials. The attached Reference 2 calculation provides an evaluation of weld wire heat 61782 surveillance data for assessing the integrity of the Point Beach Unit 1 reactor vessel. This evaluation was performed in accordance with the NRC guidelines provided at the November 12, 1997 meeting. Using the R.E. Ginna plant-specific weld metal surveillance data and making the required adjustments to the data, the surveillance data chemistry factor was calculated to be 163.3°F. This value differs slightly from the value for weld wire heat 61782 that is currently listed in RVID Version 2 (the value in the RVID is 167.4°F - Reference 3).

Therefore, please make the necessary corrections to the Reference 3 database to account for this discrepancy in the PBNP reactor vessel data.