

Attachment 1

Evaluation for LOCA Aging at 1880 MWt

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MONTICELLO NUCLEAR GENERATING PLANT		3494
TITLE:	CALCULATION COVER SHEET	Revision 5
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CALCULATION COVER SHEET

Title Evaluation For LOCA Aging At 1880 Mwt CA- 98- - 106 Add. _____

Vendor No. _____

Associated Reference _____

Assigned Personnel

Name (Print)	Signature	Title	Initials
Brian Linde	<i>Brian Linde</i>	Sr. Production Engr.	BL
Matt Antony	<i>Matt Antony</i>	Production Engr.	MSA
Alan Wojchowski	<i>Alan Wojchowski</i>	SSSE	AW

Record of Issues

Rev	Description	Total No. of Sheets	Last Sheet No.	Preparer	Verifier	Verifier	Approval	Approval Date
1	Revision 1, addition of data to table 1 on page 5	16	108	<i>BL</i>	<i>MSA</i>		<i>AW</i>	4-7-98

☐ Vendor Verification/Approval in Document

Verification Method(s)

☒ Review ☐ Alternate Calculation ☐ Test ☐ Other

References/Filing Locations

- _____
- _____

Associated Subjects/Component - EQ

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: _____

FOR ADMINISTRATIVE USE ONLY	Resp Supv: GSE-NGS ARMS: 3494	Assoc Ref: AWI-05.01.25 Doc Type: 3042	SR: N Admin initials: _____	Freq: 0 yrs Date: _____
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APPROVED (Signatures available in Master File)

PURPOSE

The purpose of this calculation is to evaluate specific environmentally qualified (EQ) equipment located in the Drywell for DBA LOCA accident aging at the Rerate power level of 1880 MWt in response to an NRC request for additional information to demonstrate margin. This will be accomplished by comparing the 1880 MWt DBA LOCA test profile to the test profile for the DG O'Brien containment penetrations and General Atomic Radiation monitors. These components were selected by the NRC for review based on meetings held at NRC headquarters in White Flint, MD on March 30, 1998.

BACKGROUND

EQ components located in the containment are qualified for their required operating time, plus margin as required by 10 CFR 50.49. For those components with a short operating time, the test profile bounds the required operating time of the component and qualification is demonstrated. In the case of equipment with a longer operating time requirement, the qualification test duration may not envelope the required operating time. In some cases due to the methodology used during the testing of equipment, portions of the test profile may not bound the accident profile (see figures in attachment 2).

When required, component evaluations are made using the Arrhenius Methodology as provided in Reference 3. The Arrhenius Model establishes a correlation between thermal aging of different temperature time histories through consideration of material properties of the constituent components in an equipment item.

For this calculation, the following discussion is based on data appropriate for 1880 MWt with the use of the ANS 5.1 decay power time history as defined for the Power Rerate Program. For the DBA LOCA, at about 17 days it is expected that the drywell temperature will drop below 135°F [Ref. 6].

The limiting drywell parameters for the DBA LOCA for the first 24 hours are available in Reference 9, File 1880_DBA. Drywell parameters for the DBA LOCA from 1 day through 180 days are provided in Reference 6. These curves provide the basis for the composite profile at the 1880 MWt conditions.

The function of the General Atomic Radiation monitor and associated DG O'Brien electrical penetration is to provide high range monitoring of the dose rates inside containment post accident. This function is not required for breaks evaluated in SBA and IBA since under these conditions there is no fuel damage assumed and therefore no high radiation doses that would require monitoring. The monitoring function is required for the DBA LOCA since this accident provides the source for high dose rates.

To provide an evaluation of margin the test profiles for these devices will be compared to the 1880 MWt DBA LOCA profile and margin quantified and compared to the margin calculated in CA-98-105 "Qualified Life Evaluation For Containment EQ Components" Rev 1 Dated 3/28/98 [10]. This calculation used a profile which included conservative steps for thermal aging as shown in the figure in attachment 3. An evaluation will be performed to compare the results of the test profiles to the actual curve of the 1880 MWt profile and the stepped profile, to demonstrate the margin in this approach.

METHODOLOGY

The bounding DBA LOCA accident profile for the 1880 MWt Rerate power level [Ref. 2] will be evaluated for the equivalent aging using the Arrhenius methodology [Ref. 3]. The equivalent aging for the 1880 MWt case will be compared to the test profile for the General Atomic Radiation monitor and associated DG O'Brien electrical penetration using the Arrhenius methodology. To demonstrate margin, the Arrhenius methodology will be applied to the DBA LOCA profile (Profile {2} in Table 1) using similar time steps to those used in calculation CA-98-105 [Ref. 10] and this will be compared to the results using the actual data time steps from the DBA profile (Profile {1} in Table 1). This will result in a more realistic value of aging based on the 1880 MWt DBA LOCA curve for comparison to the aging data from CA-98-105.

ACCEPTANCE CRITERIA

The comparison of the 1880 MWt DBA LOCA profile to the test profiles must demonstrate that the test profile provides adequate margin between the accident profile and the test profile. This evaluation will be performed using an EXCEL spreadsheet in attachment 1. The margin calculated for the DBA LOCA curve at 1880 MWt must demonstrate an increase in margin over the value determined in CA-98-105 [Ref. 10].

INPUTS

Inputs to this calculation are listed below:

1. Output data from NSP calculation CA-97-176 "Equivalent Temperature Evaluation for EQ Equipment in Containment" Rev 2. [Ref. 2]
2. DBA LOCA test data for the General Atomic Radiation monitor and associated DG O'Brien electrical penetration located in the MNGP containment as listed in the MNGP EQ Central File [Ref. 5].
3. Arrhenius methodology as defined in EWI 08.11.01 "Equipment Qualification User's Manual" [Ref. 3]
4. Output of Calculation CA-97-197, Evaluation of Drywell Response to DBA LOCA for EQ, Rev. 0 [Ref. 6]

5. Output of Calculation CA-98-105 "Qualified Life Evaluation For Containment EQ Components" [Ref. 10]

ASSUMPTIONS

1. Qualified life for EQ components is typically evaluated based on an average drywell temperature of 135°F over the life of the component prior to the accident. The mixed drywell temperature of 135°F is used as an initial assumption for primary containment accident response evaluations. Computer alarms along with administrative controls are established to insure appropriate actions are taken if this temperature is ever reached during normal plant operation. Therefore the use of 135° to evaluate aging of components while in service for the purposes of this calculation is an appropriate and conservative assumption.

ANALYSIS

The Arrhenius Methodology was used for all calculations in Attachment 1 [Reference 3 and Reference 1]. Activation energies used in the analysis are from Reference 5. The equation used for calculations in Attachment 1 is based on the Arrhenius equation from Appendix 14.2 of Reference 3. The Arrhenius equation is:

$$t_a = t_i \exp \{ (eV) \times 1.16 \times 10^4 (1/T_a - 1/T_i) \}$$

where:

t_a is thermal aging life derived from the Arrhenius equation, exposure time limit at one specified temperature (T_a)

eV is the activation energy

t_i is the test exposure time, or in this case the time spent at the accident temperature associated with this time step

T_i is the test temperature, or in this case the accident temperature

T_a is the specified temperature to which the Arrhenius equation is doing the comparison

The first column of each spreadsheet calculation shows time after the start of the LOCA test in seconds. The second column shows the temperature of the containment at the time shown for each associated test or accident time history. The third column shows the equivalent thermal life based on a reference temperature of 135°F for that time interval. The equation used for the tested accident profile evaluation with an activation energy given in the top cell in the third column is shown below:

$$=(B4-B3)*EXP((0.8*11600)*((1/330.372)-(1/(((C3-32)/1.8)+273.15))))$$

Where:

(B4-B3) is the time from the previous time step to the current time step

* is indication of multiplication

EXP is indication of e raised to the power shown by the remaining equation

0.8 is the activation energy, i.e. 0.8 eV, which varies with component type

11600 is a conversion factor to convert eV to °K (Inverse of Boltzman's constant)
 $1/330.372$ is 1 divided by 330.372°K or 135°F which is to determine the thermal aging life at this temperature
 $1/(((C3-32)/1.8)+273.15)$ is the test, or accident, temperature based on the value provided for the previous time step as shown in column 2

This results in evaluating accident aging by always maintaining the temperature at the higher temperature from the previous time step over the time from the previous time step to the current time step. This provides a conservative value for accident aging since the evaluation is essentially always done at values equal to or higher than the accident time history curve.

The test profile is evaluated by averaging the temperature of the previous and current time steps. This results in a conservatively lower value for temperature and therefore a more conservative evaluation of the test profile aging. This provides a conservative value for test aging since the evaluation is essentially done at values equal to or less than the aging time history curve.

CONCLUSION

The following table provides a summary of the results of the comparison of the component specific test profile to the DBA LOCA accident profile for the 1880 MWt case, as well as the 1670 MWt and 1880 MWt composite profiles from Ref. 10.

Table 1 Aging Comparison

Accident Profile Case	General Atomic Radiation Monitor Aging / % Margin	DG O'Brien Ctmt Penetration Aging / % Margin
1670 MWt Composite [Ref. 10]	0.42 Yr	0.13 Yr
1880 MWt Composite [Ref. 10]	1.17 Yr	0.24 Yr
Qualification Test Aging [Ref. 10] Test Profile	1.4 Yr	0.3 Yr
1880 MWt DBA LOCA (2) (with step curve function)	1.11 Yr / 79.3%	0.238 Yr / 79.3%
1880 MWt DBA LOCA (1) (Actual profile)	0.34 Yr / 24.3%	0.14 Yr / 46.7%

The % Margin values given in the table above reflect the percent of total test profile aging that the accident profile represents. This value is derived by dividing the accident aging by the Qualification Test Aging.

above the test profile as demonstrated on figures in attachment 2. This does not impact the environmental qualification evaluation since the additional margin demonstrated in table 1 clearly shows that the test profile aging bounds the accident profile aging with significant margin.

The spreadsheet calculation as described in the Analysis section is provided as Attachment 1. A review of the aging results in table 1 shows that there is significant margin between the aging

defined for the test profile and the required accident aging for the 1880 MWt profile {2} above used for evaluation. Additionally, significant margin is demonstrated by a comparison of the test profile aging to the Arrhenius methodology applied to the actual DBA curve {1} above.

FUTURE NEEDS

None identified.

ATTACHMENTS

1. EXCEL Spreadsheet for Aging Analysis
2. Graphs showing linear and Log plots of accident and test temperature profiles
3. Graph showing Comparison on DBA LOCA profile at 1880 MW to the Step function profile
4. Verification Check List form 3495

REFERENCES:

1. Nuclear Power Plant Equipment Qualification Reference Manual, published by Electric Power Research Institute, Copyright 1992
2. CA-97-176 "Equivalent Temperature Evaluation for EQ Equipment in Containment" Rev 2
3. EWI 08.11.01 "Equipment Qualification Users Manual" Rev. 0
4. Letter from NRC to R.O. Anderson, NSP, "Monticello Nuclear Generating Plant - Request for Additional Information on License Amendment Request Entitled 'Supporting the Monticello Nuclear Generating Plant (MNGP) Power Rerate Program' (TAC No. M96238)" February 11, 1998
5. Monticello Nuclear Generating Plant (MNGP) EQ Central File
6. CA-97-197, "Evaluation of Drywell Response to DBA LOCA for EQ", Rev. 0
7. Containment Response Evaluation, August 1997, GE-NE-T2300731-1, DRF T23-00731
8. AE-083-0983, "Monticello Nuclear Power Plant Extended Drywell Temperature Analysis", October 1983
9. GE letter from P. T. Tran to S. J. Hammer, GLN-97-031, Suppression Pool Temperature Time Histories for 400 Days (Task 6.0), dated September 3, 1997
10. CA-98-105 "Qualified Life Evaluation For Containment EQ Components" Rev 1 Dated 3/28/98

Spread Sheets for Evaluation of Test Data

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The data below is for the Accident aging for the Composite Accident profile developed in Calculation CA-97-176. This is the 1880 MWt bounding curve.

DG OBRIEN			GA RAD MON		
Time, sec	CA-97-176	Accident Aging, sec	Time, Sec	CA-97-176	Accident Aging, Sec
0.1	335		0.1	335	
300	335	2.49E+04	300	335	3.52E+05
300	330	8.30E+00	300	330	1.18E+02
600	330	2.29E+04	600	330	3.09E+05
600	285	0.00E+00	600	285	0.00E+00
1500	285	3.09E+04	1500	285	2.58E+05
55000	230	1.84E+06	55000	230	1.53E+07
432000	163	4.23E+06	432000	163	1.81E+07
1140480	163	1.56E+06	1140480	163	2.51E+06
Sum, sec		7.71E+06	Sum, sec		3.68E+07
Sum, hr		2141	Sum, hr		10228
Sum, yr		0.24	Sum, yr		1.17

DG OBRIEN			eV=0.5	GA RAD MON			eV=0.8
AGING FOR DBA @1880				AGING FOR DBA @1880			
Time, Sec	Temp	Accident Aging, Sec		Time, Sec	Temp	Accident Aging, Sec	
0.1	285	10299		0.1	285	85955	
300	285	0		300	285	0	
300	285	10302		300	285	85984	
600	285	30907		600	285	257952	
1500	285	1837233		1500	285	15333828	
55000	285	0		55000	285	0	
55000	228	4050161		55000	228	16832585	
432000	228	0		432000	228	0	
432000	163	1560222		432000	163	2505542	
1140480	163			1140480	163		
SUM, SEC		7499124		SUM, SEC		35101847	
SUM, YR		0.237796		SUM, YR		1.113072	

The data above is for the DBA LOCA 1880 MWt profile using the steps as evaluated in Calculations CA-97-197 and CA-97-176. This method of step function is used to evaluate the DBA LOCA profile in a conservative manner. This is presented visually in the graph in attachment 3.

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D.G. OBrien			Act Enrgy = 0.5	GENERAL Act Enrgy = 0.8		
time, Sec	Test Temp	Aging, sec		time, Sec	Test Temp	Aging, sec
0.01	135			0.1	75	
10	225	3.44E+01		38	295	3.35E+02
20	280	1.81E+02		90	355	4.68E+04
30	315	4.33E+02		300	355	4.14E+05
40	330	6.73E+02		300	365	0.00E+00
60	330	1.53E+03		10800	365	2.65E+07
120	331	4.62E+03		10800	345	0.00E+00
180	340	5.02E+03		21600	345	1.65E+07
240	340	5.40E+03		21600	270	0.00E+00
360	338	1.06E+04		32400	270	1.95E+06
540	340	1.59E+04		32400	78	0.00E+00
570	340	2.70E+03		Aging Sum, Sec =		4.54E+07
600	300	1.93E+03		Aging Sum, Yr =		1.4
720	260	3.75E+03		REQ'D OP TIME		
780	200	6.74E+02		1140480		
960	165	6.60E+02		Accident Aging =		1.17
1080	158	2.54E+02		This data is a compilation of the aging for the Test profiles for the DG OBrien and General Atomic Rad monitor. This data is compiled in calculation CA-98-105 [Ref 10].		
1200	150	2.07E+02				
1365	150	2.54E+02				
1410	340	6.97E+02				
1440	335	2.59E+03				
1680	340	2.07E+04				
1920	335	2.07E+04				
2100	300	1.11E+04				
2700	300	2.72E+04				
3120	300	1.90E+04				
3480	298	1.60E+04				
3660	297	7.79E+03				
3960	293	1.24E+04				
4260	290	1.16E+04				
4500	283	8.48E+03				
4680	280	5.79E+03				
4920	277	7.28E+03				
5220	274	8.60E+03				
5400	271	4.87E+03				
5580	269	4.63E+03				
5760	265	4.37E+03				
6060	262	6.79E+03				
6300	260	5.17E+03				
6480	258	3.72E+03				
6540	250	1.12E+03				
6720	248	3.03E+03				
6900	246	2.91E+03				
7080	246	2.85E+03				
7500	247	6.72E+03				
7620	247	1.94E+03				
7920	240	4.50E+03				
8460	239	7.45E+03				
8640	232	2.28E+03				

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9660	231	1.18E+04						
10320	229	7.41E+03						
10860	223	5.55E+03						
11580	217	6.47E+03						
11820	215	1.97E+03						
12480	211	5.05E+03						
13020	221	4.43E+03						
16800	211	3.10E+04						
18660	213	1.39E+04						
25020	212	4.81E+04						
86400	209	4.44E+05						
172800	205	5.75E+05						
259200	208	5.69E+05						
345600	210	6.03E+05						
432000	212	6.32E+05						
518000	212	6.44E+05						
604000	212	6.44E+05						
691200	212	6.53E+05						
777600	210	6.32E+05						
864000	208	6.03E+05						
950400	200	5.36E+05						
1036800	200	4.87E+05						
1123200	202	4.99E+05						
1209600	202	5.11E+05						
1296000	206	5.36E+05						
1468800	82	2.25E+05						
Aging Sum, Sec =		9.22E+06						
Aging Sum, Yr =		0.3						

See Text box at end of Data below						
DG O'BRIEN			eV=0.5	GA RAD MON		
Time, Sec	DBA Tem	Accident Aging, Sec		Time, Sec	DBA Tem	Accident Aging, Sec
0						
0.1	135	13		0.1	135	13
13	284.9	411		13	284.9	3429
25	278.2	211		25	278.2	1632
32	278.2	151		32	278.2	1165
37	278.2	181		37	278.2	1399
43	278.8	183		43	278.8	1424
49	277.2	207		49	277.2	1582
56	277.6	209		56	277.6	1602
63	277.6	209		63	277.6	1602
70	277.4	178		70	277.4	1365
76	277.3	237		76	277.3	1814
84	277.3	178		84	277.3	1360
90	277.2	207		90	277.2	1582
97	277.2	1273		97	277.2	9720
140	277.6	2058		140	277.6	15789
209	277.3	2136		209	277.3	16325
281	276.8	1998		281	276.8	15182
349	276.4	1982		349	276.4	14996
417	276.2	1830		417	276.2	13808
480	276.3	1833		480	276.3	13851
543	276.4	2216		543	276.4	16761
619	276.5	7039		619	276.5	53313
860	276.6	7228		860	276.6	54809
1107	276.8	7344		1107	276.8	55817
1357	276.6	7316		1357	276.6	55475
1607	276.4	7288		1607	276.4	55134
1857	276.1	7246		1857	276.1	54626
2107	275.7	6931		2107	275.7	52013
2348	274.6	5913		2348	274.6	43806
2558	272.8	8456		2558	272.8	61348
2869	269.9	8221		2869	269.9	57653
3189	265.6	7104		3189	265.6	47346
3490	261.5	7525		3490	261.5	47745
3836	257.8	8436		3836	257.8	51186
4254	254.6	8736		4254	254.6	50970
4716	252.4	7772		4716	252.4	44133
5146	251.2	8058		5146	251.2	45083
5603	250.6	7697		5603	250.6	42746
6045	249.9	7741		6045	249.9	42617
6496	248.3	7704		6496	248.3	41573
6960	246	6820		6960	246	35758
7391	243.8	7509		7391	243.8	38292
7888	241.4	7324		7888	241.4	36226
8398	239.4	6949		8398	239.4	33507

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8903	237.7	7298	8903	237.7	34428
9453	236.4	7910	9453	236.4	36692
10066	235.3	13243	10066	235.3	60565
11117	234.2	13424	11117	234.2	60520
12208	233.9	13019	12208	233.9	58465
13273	234	13500	13273	234	60707
14375	234.2	13337	14375	234.2	60131
15459	234.5	13585	15459	234.5	61489
16556	234.7	13321	16556	234.7	60449
17627	234.8	13562	17627	234.8	61622
18715	234.9	13304	18715	234.9	60528
19780	234.9	13916	19780	234.9	63313
20894	234.9	13641	20894	234.9	62063
21986	234.9	13566	21986	234.9	61722
23072	234.9	13541	23072	234.9	61608
24156	234.9	13554	24156	234.9	61665
25241	234.9	13691	25241	234.9	62290
26337	234.7	13296	26337	234.7	60336
27406	234.6	13516	27406	234.6	61252
28495	234.5	13734	28495	234.5	62162
29604	234.3	13317	29604	234.3	60118
30684	234.1	13554	30684	234.1	61029
31788	233.9	13679	31788	233.9	61430
32907	233.6	13019	32907	233.6	58240
33979	233.4	13459	33979	233.4	60048
35092	233.1	13707	35092	233.1	60920
36233	232.8	12640	36233	232.8	55954
37292	232.5	13280	37292	232.5	58561
38412	232.3	13010	38412	232.3	57220
39514	232	12949	39514	232	56727
40618	231.7	12433	40618	231.7	54254
41685	231.4	13012	41685	231.4	56556
42809	231.1	12490	42809	231.1	54073
43895	230.8	12591	43895	230.8	54296
44997	230.5	12349	44997	230.5	53046
46085	230.1	12286	46085	230.1	52499
47177	229.8	12720	47177	229.8	54137
48315	229.4	12298	48315	229.4	52067
49425	229	11631	49425	229	48981
50484	228.7	12121	50484	228.7	50845
51595	228.3	12015	51595	228.3	50133
52706	227.8	12001	52706	227.8	49743
53828	227.4	11715	53828	227.4	48301
54933	227	11612	54933	227	47622
56038	226.5	11681	56038	226.5	47590
57162	226.1	11712	57162	226.1	47462
58299	225.6	11419	58299	225.6	45969
59420	225.1	11041	59420	225.1	44151
60516	224.6	11128	60516	224.6	44202
61633	224.2	10921	61633	224.2	43147
62739	223.7	11141	62739	223.7	43724

[illegible]

List of Figures :

Figure 1 DG O'Brien vs DBA LOCA @ 1880 Mwt

Figure 2 DG O'Brien vs DBA LOCA @ 1880 Mwt

Figure 3 GA Rad Monitor vs DBA LOCA @1880 Mwt

Figure 4 GA Rad Monitor vs DBA LOCA @1880 MWt

DG OBRIEN vs DBA LOCA @ 1880 MWt

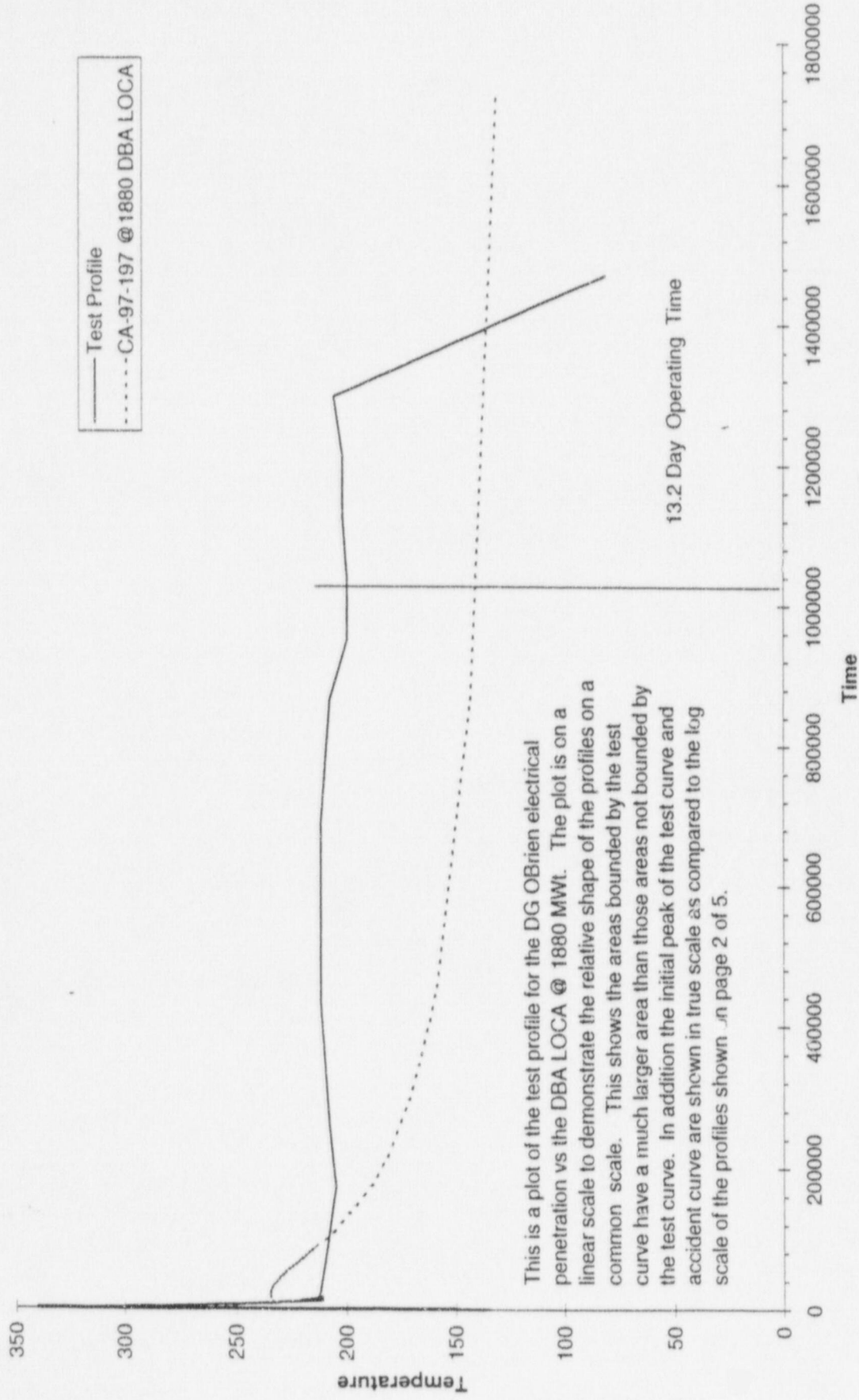


Figure 2

DG OBRIEN vs DBA LOCA @ 1880 MWt

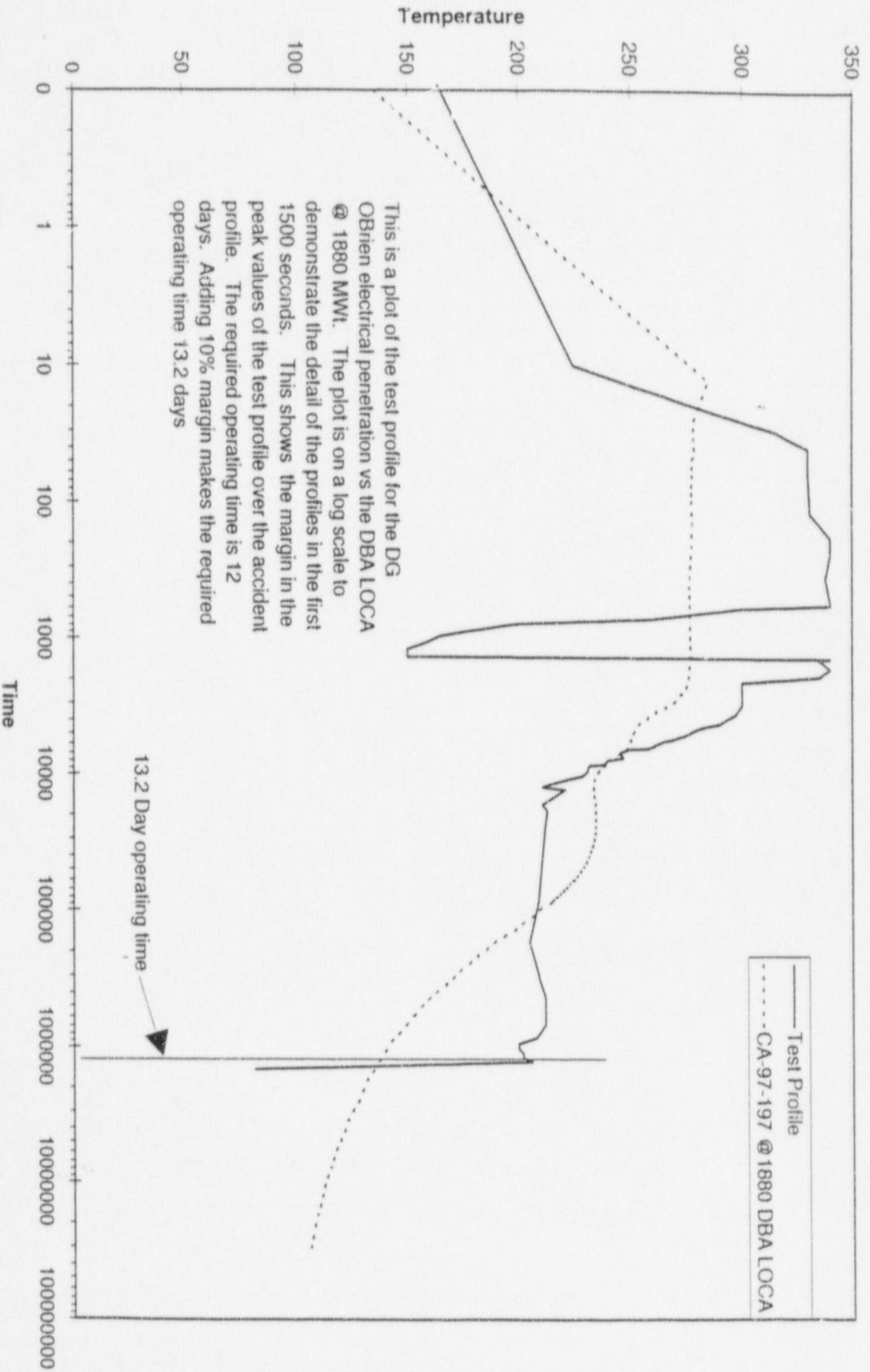


Figure 3

GA Rad Monitor vs DBA LOCA @1880 MWt

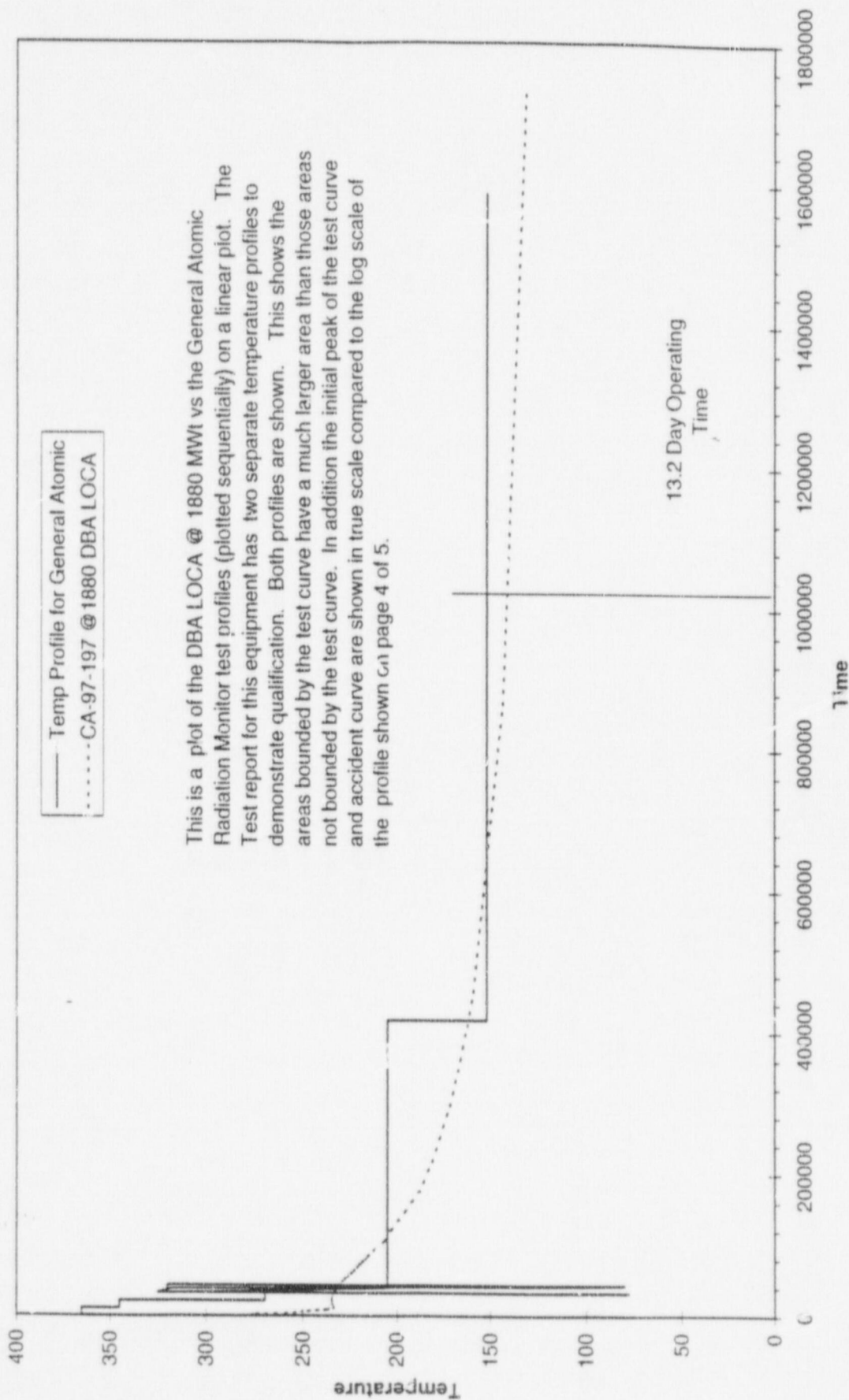
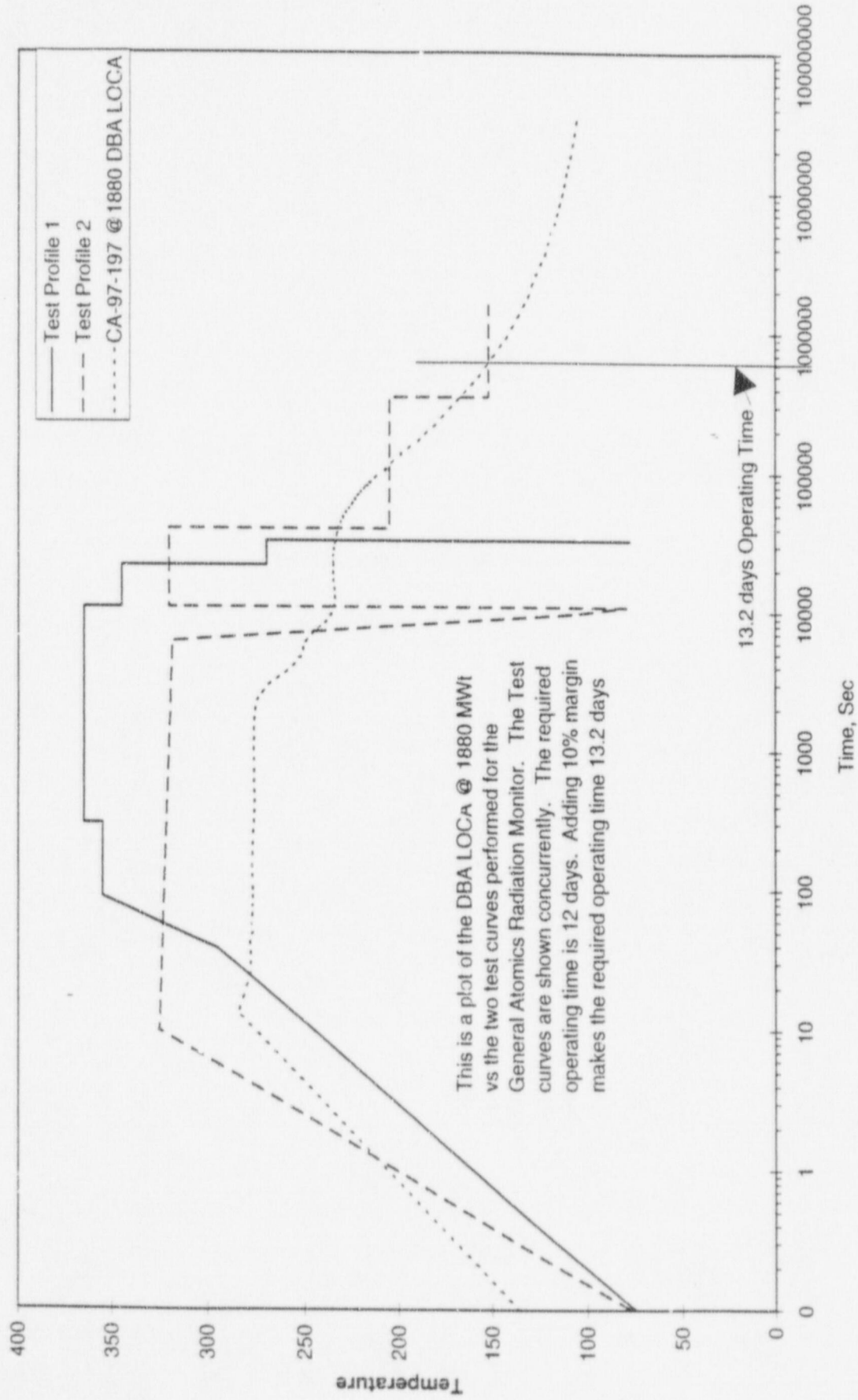


Figure 4

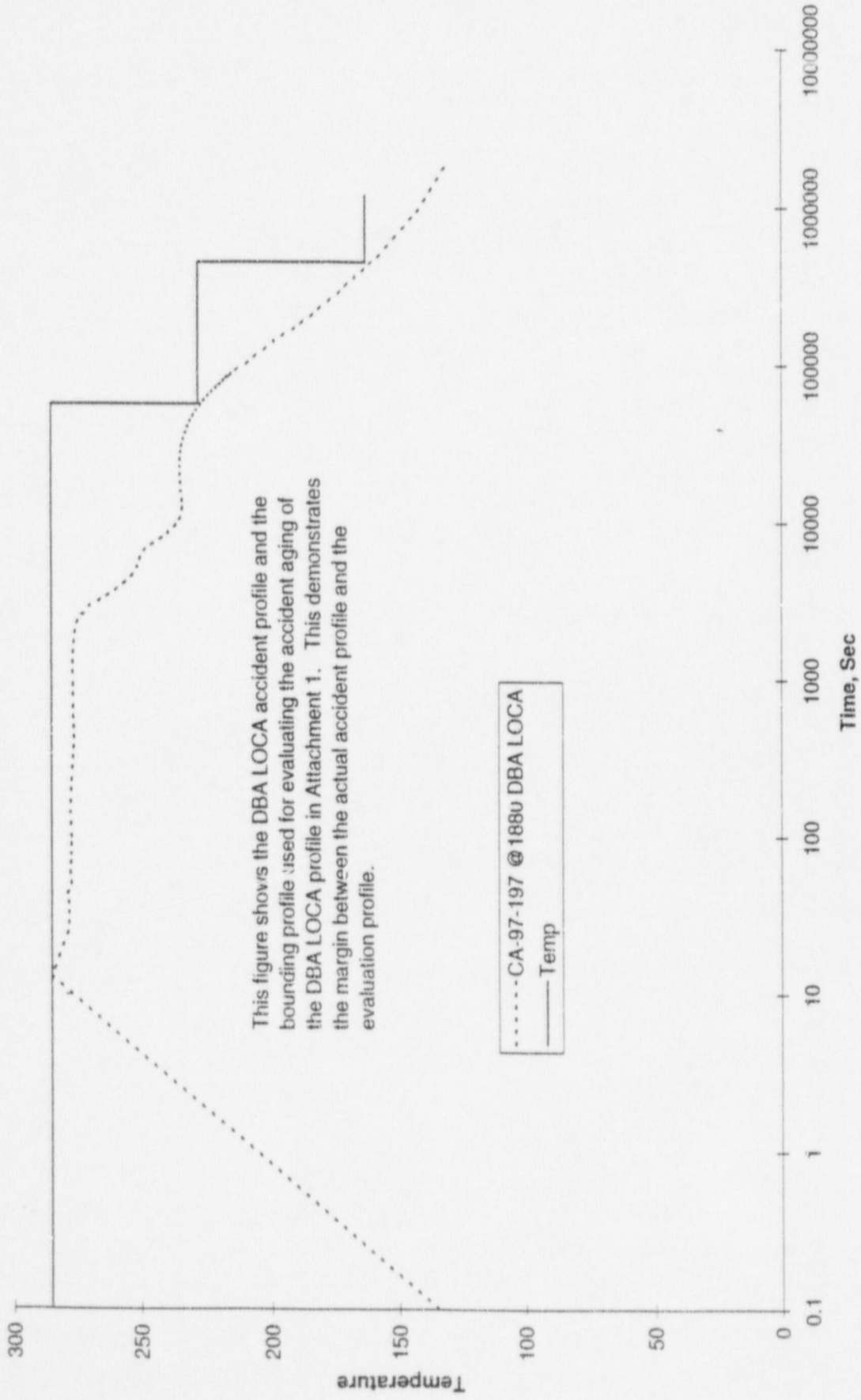
General Atomic RM vs DBA LOCA @ 1880 MWt



List of Figures :

Figure 1 Comparison of DBA LOCA @ 1880 MWt to Evaluation Profile

Comparison of 1880 MWt DBA LOCA to Evaluation Curve



MONTICELLO NUCLEAR GENERATING PLANT		3495
TITLE:	CALCULATION/ANALYSIS VERIFICATION CHECKLIST	Revision 5
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Place initial by items verified.

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Attachment 4
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REVIEW

1. Inputs correctly selected.
2. Assumptions described and reasonable.
3. Applicable codes, standards and regulations identified and met.
4. Appropriate method used.
5. Applicable construction and operating experience considered.
6. Applicable structure(s), system(s), and component(s) listed.
7. Formulas and equations documented, unusual symbols defined.
8. Detailed to allow verification without recourse to preparer.
9. Neat and legible, pages all correctly numbered.
10. Signed by preparer.
11. Interface requirements identified and satisfied.
12. Acceptance criteria identified, adequate and satisfied.
13. Result reasonable compared to inputs.
14. Basis of all assumptions, acceptance criteria and inputs are identified.
15. Conclusions not in conflict with previous analysis, USAR, Technical Specifications or NRC Safety Evaluations.

Verified

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ALTERNATE CALCULATION

16. Alternate calc results consistent with original.
17. Items 1-4 above verified. (Required by ANSI N.45.2.11)

NA - MSA
NA - MSA

TESTING

18. Testing requirements fully described and adequate.
19. Shows adequacy of tested feature at worst case conditions.
20. If test is for overall design adequacy, all operating modes considered in determining test conditions.
21. If model test, scaling law and error analysis established.
22. Results meet acceptance criteria, or documentation of acceptable resolution is attached.

NA - MSA
NA - MSA
NA - MSA
NA - MSA
NA - MSA

OTHER (Explain) _____

FINAL DOCUMENTATION (Verify applicable items included)

23. Alternate or check calcs
24. Summary of test results.
25. Comments (errors, discrepancies, recommendations).
26. Method of resolution of comments.

NA - MSA
NA - MSA
NA - MSA
NA - MSA

Completed By: Matthew S. Antony Date: 4/4/98

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated:							
FOR ADMINISTRATIVE USE ONLY	Resp Supv	GSE-NGS	Assoc Ref	AWI-05 01 25	SR N	Freq	0 yrs
	ARMS	3495	Doc Type	3042	Admin initials	Date	

APPROVED (Signatures available in Master File)

Comments to CA-98-106

- Complete
4/4/98
MSH
- Complete
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MSH
- Complete
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MSH
- Complete
4/4/98
MSH
1. The purpose should address why these two components were used to demonstrate margin. It appears that the referenced components provided the slimmest margin in the CA-98-105 calculation. Is this information requested based on the CA-98-105 calculation?
 2. The activation energy stated in the analysis Arrhenius definition is inconsistent with that used in the sample equation. It may be better to not use an example activation energy (just use eV). This may be less confusing.
 3. Attachment No. 1 1880 MW DBA-calculation for aging should indicate Reference 9 in addition to Calculation CA-97-197. They are both required to build the curve presented. Calculation CA-97-176 could also be referenced in lieu of CA-97-197 and Reference 9.
 4. Add Verification Checklist as Attachment 4.

MONTICELLO NUCLEAR GENERATING PLANT		3495
TITLE:	CALCULATION/ANALYSIS VERIFICATION CHECKLIST	Revision 5
		Page 1 of 1

Place initial by items verified.

CA - 98 - 106 Rev. 1
Attachment 4
Page 3 of 3
4/6/98

REVIEW

- Inputs correctly selected.
- Assumptions described and reasonable.
- Applicable codes, standards and regulations identified and met.
- Appropriate method used.
- Applicable construction and operating experience considered.
- Applicable structure(s), system(s), and component(s) listed.
- Formulas and equations documented, unusual symbols defined.
- Detailed to allow verification without recourse to preparer.
- Neat and legible, pages all correctly numbered.
- Signed by preparer.
- Interface requirements identified and satisfied.
- Acceptance criteria identified, adequate and satisfied.
- Result reasonable compared to inputs.
- Basis of all assumptions, acceptance criteria and inputs are identified.
- Conclusions not in conflict with previous analysis, USAR, Technical Specifications or NRC Safety Evaluations.

Verified

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MSA

ALTERNATE CALCULATION

- Alternate calc results consistent with original.
- Items 1-4 above verified. (Required by ANSI N.45.2.11)

NA-MSA

NA-MSA

TESTING

- Testing requirements fully described and adequate.
- Shows adequacy of tested feature at worst case conditions.
- If test is for overall design adequacy, all operating modes considered in determining test conditions.
- If model test, scaling laws and error analysis established.
- Results meet acceptance criteria, or documentation of acceptable resolution is attached.

NA-MSA

NA-MSA

NA-MSA

NA-MSA

NA-MSA

OTHER (Explain)

FINAL DOCUMENTATION (Verify applicable items included)

- Alternate or check calcs
- Summary of test results.
- Comments (errors, discrepancies, recommendations).
- Method of resolution of comments.

NA-MSA

NA-MSA

NA-MSA

NA-MSA

Completed By:

Matthew S. Anthony

Date: 4/6/98

3087 (DOCUMENT CHANGE, HOLD AND COMMENT FORM) incorporated: 97-1042					
FOR ADMINISTRATIVE USE ONLY	Resp Supv:	GSE-NGS	Assoc Ref:	AWI-05 01 25	SR: ON
	ARMS:	3495	Doc Type:	3042	Admin Initials: AT
				Freq:	0 yrs
				Date:	2/10/97

M/jrs

Attachment 2

MOV Information

1. *What are NSP's improvement plans to address possible motor damage for MO-2034 and MO-4229 at rerate conditions?*

NSP Response

Prior to operation at rerate conditions, NSP will initiate a condition report to evaluate whether the subject valves are capable of allowing a subsequent operation after the required isolation safety functions are completed. This evaluation may include an examination of assumptions and methodologies, additional administrative controls, and modifications. The evaluation will be completed in order to institute the corrective actions, if any, by the end of the next scheduled refueling outage.

2. *What are NSP's improvement plans regarding MOVs with relatively low capacity margin at rerate conditions?*

NSP Response

Of the five valves discussed with the staff, three valves now have acceptable excess capacity. These valves and the approximate margin values are listed below.

MO-2373 (Initial Margin 5%)

Actuator/Valve replaced this outage, now 32% margin.

MO-2015 (Initial Margin 6%)

Adjusted this outage to 9% based on rate of loading and full dp test.

MO-2006 (Initial Margin 6%)

Adjusted this outage to 17%.

The remaining valves include MO-2398 (3%) and MO-2034 (6%). These margins will be evaluated by a method similar to that described for issue 1. above. Prior to operation at rerate conditions, NSP will initiate a condition report to evaluate the capacity margin of MO-2398 and MO-2034. This evaluation may include an examination of assumptions and methodologies, additional administrative controls, and modifications. The evaluation will be completed in order to institute the corrective actions, if any, by the end of the next scheduled refueling outage.

Attachment 3

NSP Commitments for the Monticeilo Power Rerate

The following commitments apply to the implementation of the Monticello power rerate program.

All plant changes described in Exhibit D of NSP's power rerate license amendment request dated December 4, 1997 will be completed prior to implementation of power rerate.

All commitments contained in Exhibit H of NSP's power rerate license amendment submittal dated December 4, 1997 will be completed as described therein.

Feedwater and Condensate System area ambient temperatures will be monitored at rerate conditions to confirm that design temperatures are not exceeded.

Feedwater and Condensate System testing will be conducted as described by NSP's response to Question 17 contained in NSP's power rerate submittal dated March 26, 1998.

The rerate startup test program will include a requirement to monitor moisture separator drain system stability.

The adequacy of the service water system will be confirmed by monitoring the system and its loads during the rerate startup testing program.

The final empirical relation between turbine 1st stage pressure and percent reactor power will be determined during the rerate startup testing program.