

50-454/455
50-456/457



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 12, 1998

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING THE REVISED STEAM
GENERATOR TUBE RUPTURE ANALYSIS - BYRON AND BRAIDWOOD
STATIONS (TAC NOS. M97315, M97316, M97317 AND M97318)

Dear Mr. Kingsley:

By letter dated November 13, 1996, Commonwealth Edison Company (ComEd) submitted a revised steam generator tube rupture (SGTR) analysis for Byron Station, Units 1 and 2, and Braidwood Station, Units 1 and 2. The analysis was revised to account for physical differences between the original steam generators and the replacement steam generators on Byron, Unit 1, and Braidwood, Unit 1, and associated changes in certain operator actions. Additional information was provided by letters dated March 20, June 2, August 19, November 3 and December 19, 1997, and April 13, 1998.

During the course of our review, we have identified the need for further information. Your letter dated November 3, 1997, states that the auxiliary feedwater (AFW) flow rate used in the analysis for margin to overflow (MTO) is nonconservative. The analysis assumes the maximum AFW flow is 464 gpm, including uncertainties. The letter also states that ComEd has performed an operability evaluation which concludes that Byron and Braidwood are still in compliance with the licensing requirements for mitigation of a SGTR. The letter further states that for Byron, Unit 1, and Braidwood, Unit 1, modifications will be performed during the steam generator replacement outages (B1R08 and A1R07, respectively) to limit the AFW flow to 464 gpm (for Byron, Unit 1, this was scheduled for fall 1997). Finally, the letter states "[p]lans for Byron, Unit 2, and Braidwood, Unit 2, will be dispositioned in refueling outages B2R08 and A2R07, respectively."

The safety evaluation for the revised SGTR analysis for Byron, Unit 1, and Braidwood, Unit 1, covering the plant conditions after steam generator replacement, was sent to ComEd by letter dated January 28, 1998, as supplemented by letter dated March 11, 1998. This safety evaluation was based, in part, on a maximum AFW flow of 464 gpm, which is consistent with your stated corrective actions for those units.

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O. Kingsley

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May 12, 1998

Please describe the corrective actions you plan to take, including schedule, to restore consistency between the AFW flow and the SGTR analysis for Byron, Unit 2, and Braidwood, Unit 2.

Your response is requested within 30 days of the date of this letter.

Sincerely,

ORIG. SIGNED BY
Stewart N. Bailey, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455,
STN 50-456, STN 50-457

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O. Kingsley

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Please describe the corrective actions you plan to take, including schedule, to restore consistency between the AFW flow and the SGTR analysis for Byron, Unit 2, and Braidwood, Unit 2.

Your response is requested within 30 days of the date of this letter.

Sincerely,



Stewart N. Bailey, Project Manager
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Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455,
STN 50-456, STN 50-457

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