



Commonwealth Edison
 72 West Adams Street, Chicago, Illinois
 Address Reply to: Post Office Box 767
 Chicago, Illinois 60690 - 0767

DMB

October 24, 1986

orig + 1

PRIORITY ROUTING

First	Second
RA	RC
DRA	ETC
DRP	SGA
ORS	HL
DESS	OL
ORMA	OI
	PAO

FILE *les*

Mr. James G. Keppler
 Regional Administrator
 U.S. Nuclear Regulatory Commission
 Region III
 799 Roosevelt Road
 Glen Ellyn, IL 60137

Subject: LaSalle County Station Unit 1
 Startup Commitments Regarding
 SOR Switches
NRC Docket No. 50-373

References (a): Telecon, September 19, 1986, among members
 of NRR, RIII, CECo Licensing and LaSalle
 Plant Staff.

(b): October 8, 1986 letter from C.M. Allen to
 H. R. Denton (Attached).

Dear Mr. Keppler:

This is to inform you of a telecon between R. Ramirez, A. Bournia
 and J. Knox of NRR and myself on October 20, 1986. That conversation
 reverified that the level drop test completed September 24, satisfies the
 test commitment made in reference (a). The results of this test are
 discussed in reference (b). Note, however, CECo still has a commitment to
 do a level drop test on shutdown of Unit 1 for refueling.

Please direct any questions you may have regarding this matter to
 this office.

Very truly yours,

C. M. Allen
 Nuclear Licensing Administrator

lm

Attachments

cc: G. Wright - NRR
 W. Guldemond - NRR

2308K

B610290347 B61024
 PDR ADOCK 05000373
 P PDR

IEO1
 OCT 27 1986



Commonwealth Edison
72 West Adams Street, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690 - 0767

October 8, 1986

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: LaSalle County Station Unit 1
SOR Differential Pressure Switch
Setpoint Characterization Program
950 psig Reactor Vessel Level Drop
Test
NRC Docket No. 50-373

- References (a): J. S. Abel letter to D. L. Farrar dated
September 16, 1986 with Results of
0.0 psig Level Drop Test for
Operability Verification.
- (b): Telecon, September 19, 1986, among members
of NRR, Region III, CECo Licensing, and
LaSalle Station Staff.

Dear Mr. Denton:

Reference (a) provides results of the 0.0 psig Reactor Vessel Level Drop operability tests for the Level III Scram and ADS Permissive functions performed by SOR differential pressure switches on Unit 1 of LaSalle County Station. This letter serves to document results of a Level Drop Test conducted September 24, 1986 on the Unit 1 vessel while it was at normal operating pressure. The test, which had previously been scheduled for the first planned outage following three months of operation after startup of the second refueling cycle (plant conditions permitting), was instead conducted September 24, 1986 per the telecon discussion referenced in (b) above.

Special Test Procedure LST-86-208 was performed with the reactor at a pressure of 942 psig. Water level was lowered by blowing down reactor inventory through the Reactor Water Cleanup System to the condenser. Water level was monitored by recording the outputs of differential pressure transmitters located on the same instrument racks and sharing the same sensing lines as the SOR differential pressure switches performing the SCRAM and ADS permissive functions. Also, during the test, reactor pressure was monitored as well as the contact status of the relays which are de-activated by the SOR differential pressure switches to perform the scram and ADS logic confirmation functions.

~~86 15270449~~

October 8, 1986

The acceptance criteria for switch trip points had been pre-defined. It was expected that all six switches should trip between a low of +12.2" RWL and a high of +18.1" RWL. In addition, by using switch-specific static shift data from the SOR Setpoint characterization program, the latest known switch actuation points from the previous 0.0 psig level drop test, and the known switch drift characteristics as observed in the SOR, Inc. long term time test data, the anticipated switch trip points were predicted.

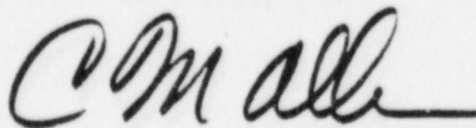
The results of the test are shown in table 1. Four of the six switches tripped within 1" (Reactor Water Level) of their predicted values. The two switches that did not trip within 1" of the predicted values were within the acceptance limits, but they tripped conservatively higher than predicted. The range of trip values, however, was comparable to the range of trip values experienced in the LaSalle Unit 2 950 psig Level Drop Test conducted on August 10, 1986.

In general, the setpoint stability for level 3 SCRAM and ADS permissive functions was demonstrated by this test to be consistent with the LaSalle SOR Setpoint Characterization Program, and was demonstrated to be adequate for performance of the safety actions required by the LaSalle Technical Specifications.

Please direct any questions you may have regarding this matter to this office.

One signed original and ten (10) copies of this letter and its attachments are provided for your use.

Very truly yours,



C. M. Allen
Nuclear Licensing Administrator

lm

Attachments

cc: Dr. A. Bournia - NRR
Resident Inspector - LSCS

2225K

TABLE 1

Summary of Results from Operating Pressure

Reactor Vessel Level Drop Test

LST-86-208, conducted 9/24/86

<u>Switch Tag #</u>	<u>Predicted* Trip Actuation at 1000 psig</u>	<u>950 psig Level Drop Test Switch Actuation</u>
1B21-N024A	15.8" RWL	15.2" RWL
1B21-N024B	15.6" RWL	14.8" RWL
1B21-N024C	16.8" RWL	17.1" RWL
1B21-N024D	15.0" RWL	17.6" RWL
1B21-N038A	16.1" RWL	17.9" RWL
1B21-N038B	14.9" RWL	15.0" RWL

* The predicted values were calculated by adding the switch-specific static shift values from the LaSalle SOR Setpoint Characterization Program, and the drift rate found in the SOR, Inc. Long Term Time Test, to the actuation points on the latest 0.0 psig Reactor Vessel Level Drop Test conducted on September 20, 1986. (See Table 2)

TABLE 2

Summary of Results of 0.0 psig

Reactor Vessel Level Drop Test

LST-86-207, conducted September 20, 1986

<u>Switch Tag #</u>	<u>Function</u>	<u>Measured Trip Point 9/9/86 ("RWL)</u>	<u>Measured Trip Point 9/20/86 ("RWL)</u>	<u>Instrument Drift ("W.C.)</u>
1B21-N024A	SCRAM Channel A1	19.8	18.4	+ 0.98
1B21-N024B	SCRAM Channel B1	18.9	17.9	+ 0.70
1B21-N024C	SCRAM Channel A2	19.3	18.4	+ 0.63
1B21-N024D	SCRAM Channel B2	19.6	18.1	+ 1.06
1B21-N038A	ADS Permissive	19.7	19.2	+ 0.35
1B21-N038B	ADS Permissive	18.7	18.4	+ 0.21