

JUN 22 1987

Docket No. 50-344

LICENSEE: Portland General Electric Company (PGE)
FACILITY: Trojan Nuclear Plant
SUBJECT: MEETING SUMMARY REGARDING THE APRIL 8, 1987 MEETING ON
F* - STEAM GENERATOR TUBE PLUGGING CRITERIA

A meeting was held in Bethesda, Maryland on April 8, 1987 with Portland General Electric Company to discuss the technical issues related to PGE's amendment request for F* steam generator tube plugging criteria. PGE's submittal LCA 152 dated February 20, 1987 provided background information on this topic. A list of attendees is provided in Enclosure 1.

PGE provided a presentation on the subject, and stated that the need for the amendment was due to "questionable" indications detected during the 1986 inspection of the steam generators (SG), in the region of the SG tubes where the F* criteria could be applied. The indications were labeled "questionable" since they could not be accurately characterized. Enclosure 2 contains PGE's presentation handouts.

Allen Nuclear Associates, Inc. followed with a presentation on the eddy current testing performed at Trojan, which included discussions on eddy current probe uncertainties, observed indications and inspection technique.

The meeting was then closed to the public during the presentation by Westinghouse. Westinghouse presented proprietary information regarding the analytic aspects of the F* evaluation.

The staff's preliminary conclusion was that the amendment request was technically acceptable, subject to certain changes to the proposed Technical Specifications. PGE agreed that the changes were necessary, and would be formally submitted. These changes were submitted by letter dated April 20, 1987.

Original Signed by TL CHAN
Terence L. Chan, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
As stated

DRSP/PDV <i>PK</i>	DRSP/PDV
TChan:cd	G. Brighton
6/16/87	6/17/87

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PDR ADDCK 05000344
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Portland General Electric Company

Trojan Nuclear Plant

cc:
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DISTRIBUTION

Docket File

NRC & Local PDRs

PD#5 Reading

GWKnighton

TChan

OGC-Bethesda

EJordan

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NRC Participants

ACRS (10)

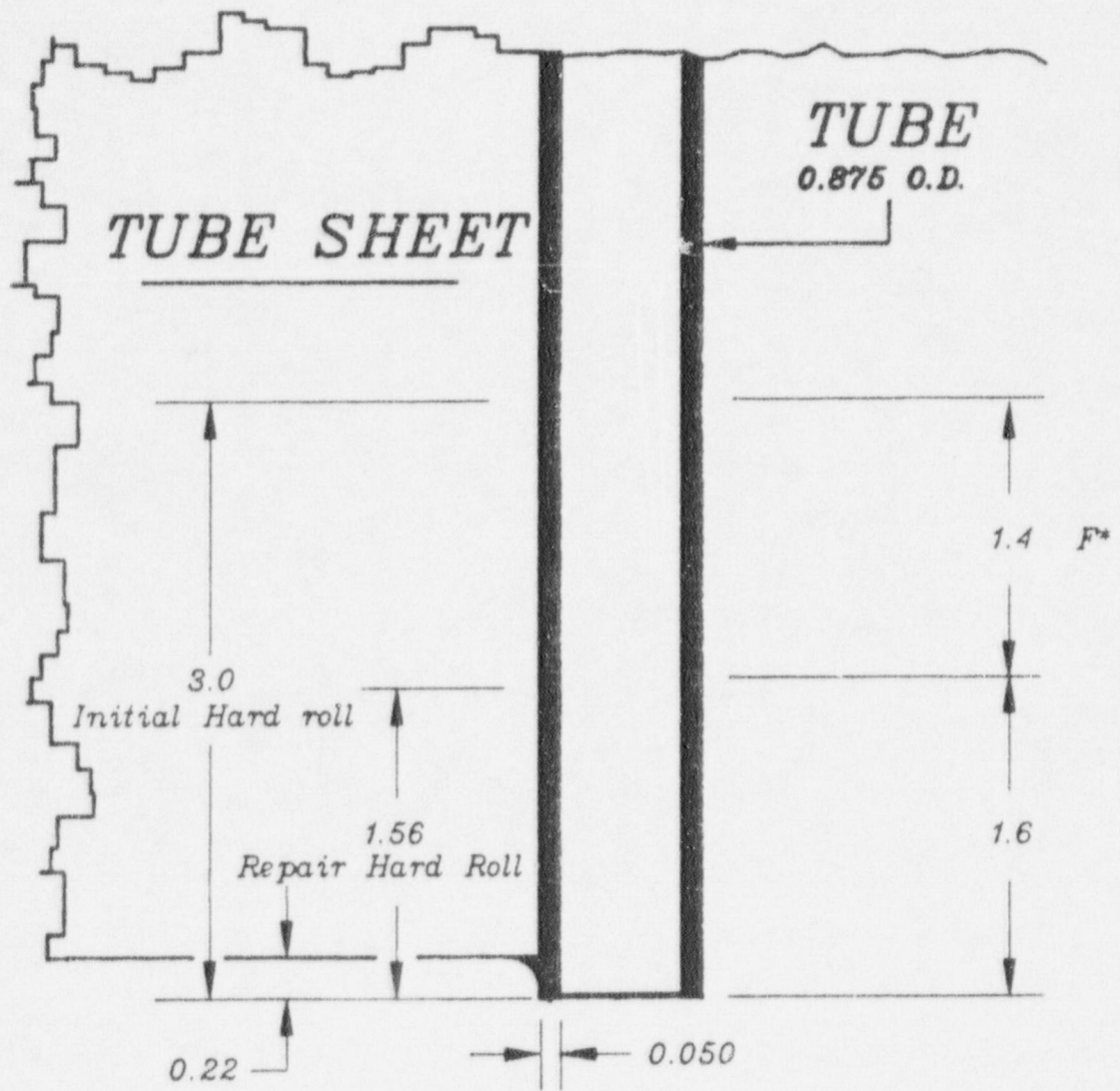
HBClayton (RI)

T. O. Martin - EDO

LIST OF ATTENDEES

APRIL 8, 1987

<u>NAME</u>	<u>ORGANIZATION</u>
T. L. Chan	NRC
E. J. Sullivan	NRC
C. Y. Cheng	NRC
C. D. Sellers	NRC
E. L. Murphy	NRC
G. A. Zimmerman	PGE
A. O. Wogen	PGE
B. L. Curtis	Allen Nuclear Associates
R. F. Keating	Westinghouse
D. A. Lindgren	Westinghouse
C. K. Lewe	NUS Corporation



STEAM GENERATOR "B"
HOT LEG

ET Indications with F*

SCALE: 1"=1"

ALL DIMENSIONS IN INCHES

PREVIOUS EDDY CURRENT INDICATIONS
IN TUBE SHEET REGION

- TUBE SHEET (T/S) AREA INDICATIONS FIRST REPORTED IN 1985
- T/S INDICATIONS PRIMARILY S/G B HOT LEG
- KNOWN INDICATIONS AFFECTED BY PROPOSED AMENDMENT ARE IN S/G
B HOT LEG

S/G B HOT LEG T/S INDICATIONS

- 111 T/S AREA INDICATIONS (UNK)

- 106 OF THESE ARE WITHIN SHOP HARD ROLL

- ~ 85 INDICATIONS ARE BELOW THE F* DISTANCE

EXAMPLES OF HARD ROLL AREA INDICATIONS

R 13 C 53
R 13 C 83

CHARACTERISTICS OF HARD ROLL AREA INDICATIONS

- OD ORIGINATED
- AXIALLY ORIENTED
- 1/4" AXIAL LENGTH


CONCLUSIONS FROM PAST EDDY CURRENT DATA

- INSUFFICIENT PREVIOUS DATA TO ESTABLISH TREND

- OPERATING EXPERIENCE IS NO DETECTABLE PRIMARY-TO-SECONDARY LEAKAGE

NEED FOR AMENDMENT

○ ELIMINATES TUBE END TRANSITION FROM CONSIDERATION

○  85 TUBES WOULD NOT REQUIRE CONSIDERATION FOR POSSIBLE PLUGGING