QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

MAY, 1987

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Verna Koselka and Kurt Schmidt, telephone number 309-654-2241, extensions 2240 and 2147.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit 1

May 1-15

The month of May began with Unit One holding full load. A load drop to 740 MWe for surveillances was started at 0213. Following the surveillances, full power was restored by 0720. At 0905 load was dropped for EGC and the unit placed in EGC at 0920. At 2310 the unit went out of EGC and a load drop was begun to facilitate a rod pattern change. Following the control rod position changes, at 0510 on May 2, a slow power ascent was begun. The ascent was interrupted at 0815 by the need for a rod pattern adjustment and resumed at 0835. Full power was reached on May 5 at 0500 and held until 0800 on May 7 when load was reduced for EGC. The unit was placed in EGC at 0830, and operated until 0730 on May 9 when the EGC was tripped due to a reserve emergency. Starting at 0740 on May 9 the unit was brought to full power at 1020 and held until 1800. Power was reduced and the unit placed in EGC at 1819. EGC was off for surveillances from 0000 to 0055 on May 10 and from 0823 to 0849 on May 11. At 1000 on May 11, a half scram occurred during weekly instrument surveillances. The half scram caused rod G-7 to be inserted. EGC was tripped and load reduced to 600 MWe to allow recovery of the rod. Following special rod maneuvers for recovery, the unit was returned to full power starting at 1030 and completed by 1124. Full load was held until 1905 on May 12. Rod G-7 was fully inserted for control system repairs and at 2230 a load reduction to 600 MWe was started to facilitate recovery of rod G-7. On May 13, special rod maneuvers were performed from 0012 to 0053 and at 0130 an ascent to full power was begun. Full power was reached at 1315 and held until 0300 on May 15. Power was reduced and the unit run in EGC from 0315 to 0710. At 0715 the unit was returned to full power where it was held for the rest of the day.

May 16-31

The 16th started with the unit holding full load. At 0947, load was dropped and the unit placed in EGC at 1003. EGC was operated until 0515 on May 19 with 4 short interruptions for surveillances and change of limits. At 0720 on May 19, the unit was brought to full power and held until 0050 on May 20. The unit was placed in EGC and operated until 0655 on May 21. The unit was brought to full load by 0715 and held until May 22 at 0720. Power was reduced to allow for control rod movements and then returned to full by 0950. Full power was held until 2205 when it was reduced and the unit placed in EGC at 2223. The unit ran in EGC until 0723 on May 26. The unit was brought to full power by 0735 and held until May 27 at 0215 when power was reduced for EGC. EGC

was operated from 0220 to 0705 and the unit returned to full power at 0715. On May 28, EGC was operated from 0027 to 1005. The unit held full power from 1125 on May 28 to 2256 on May 29 when a load drop was started for rod pattern adjustments. The load drop was completed at 0156 on May 30. The control rod pattern adjustment was performed from 0230 to 0343. At 0500, a slow power ascent was begun from 600 MWe. The load ascent continued for the rest of the month and was completed at 0530 on June 1.

B. Unit 2

May 1-15

The month of May began with the unit operating in EGC. At 0617 EGC was tripped due to a reserve emergency and the unit raised to full power by 0635. Full power was maintained until 0030 on May 3 when load was reduced to facilitate a feed pump change over. The pump change over was completed at 0350 and the unit returned to full power by 0705. Full power was held until 0045 on May 5. The unit ran in EGC from 0110 to 0930 and at full power from 1005 to 1120. The unit was placed back in EGC at 1133, ran until 1328, and then went back to full load at 1340. Full load was held until 0025 on May 6 when it was reduced for EGC. The unit was placed in EGC at 0033. At 0230, four rapid trips of EGC were experienced due to input failures. EGC was left off and power held at 740 MWe until 0335 when it was increased to 750. At 0340 EGC was reset and ran until 0555 when it again tripped on input failure. The unit was raised to full load by 0620, which it held until 1618. The unit was placed in EGC and ran until 0020 on May 7. EGC was off for turbine surveillances from 0020 to 0158. At 0318, EGC was tripped because power had dropped below the lower limit of EGC. Power was raised back to the control band and the unit returned to EGC. EGC was off again from 0840 to 0900 to switch Reactor Vessel level controllers. EGC ran until 0600 on May 8 when power was raised. Full load was held from 0605 to 0130 on May 9. The unit was operated in EGC from 0142 until 0730 when it was tripped because of a reserve emergency. The unit held full load from 0741 until 2150. The unit was placed in EGC at 2205 and operated until 1600 on May 11. The unit held full load from 1610 until 2300 and was put back in EGC at 2310. EGC was tripped at 0209 on May 12 because of a reserve emergency. The unit held full load from 0218 until May 15 at 0315. The unit ran in EGC from 0335 until 0715. Full load was held from 0730 until 2350 when load was reduced to prepare for EGC.

May 16-31

The 16th began with the unit preparing to operate in EGC. EGC was run from 0015 until 0240 on May 17. At the request of the load dispatcher, load was reduced in steps, to 575 MWe at 0550. This load was held until 0615 and then the unit raised back to full power by 0815. Full load was held until May 18 at 1425. At that time, a malfunction of the recirc MG

speed control required a load reduction to correct. The speed control was returned to normal at 1550 and the unit raised to full power. Full power was maintained from 1616 until 0030 on May 19. The unit was operated in EGC from 0040 until 0820. Full load was held from 0835 until 0255 on May 20, when the unit was placed in EGC. At 0350, EGC tripped on input failure. Also, at 0350, bus 28 was lost with the cause believed to be due to a lightning strike. The loss of bus 28 caused a 1/2 scram, 1/2 isolations on Group I and II, and various other equipment trips. By 0405, bus 28 had been recovered, the scram and isolations reset and loads returned to normal. At 0715 a power increase was begun. Full power was reached at 0820 and held until 1952. EGC was operated from 2010 to 2335. EGC was off for surveillances until 0112, and then ran until 0700 on May 21. Full load was held from 0720 until 0130 on May 22. The unit was operated in EGC from 0200 until 1034. From 1034 until 1235 EGC was off for surveillances. The unit was then operated in EGC until May 26 at 0725. The unit was raised to full power and held from 0745 until 0217 on May 27. EGC was operated from 0227 until 0704 when the unit was taken to full power. Full power was maintained from 0725 until 0000 on May 28 when it was reduced for EGC. The unit was operated in EGC from 0022 until 0827, when power was raised. The unit ran at full load from 0850 on May 28 until 0145 on May 30. EGC was operated until 1712 when it was tripped due to a problem with computer inputs. The unit was returned to EGC at 1905 and operated until 1125 on May 31. The unit was raised to full load by 1140 and finished the day holding full load.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST NO.: Q55169

LER NUMBER: N/A

COMPONENT: System 1600 - Replaced power supply to indicators 1-1640-10B & 11B.

CAUSE OF MALFUNCTION: The cause of malfunction of the containment pressure (PI-1-1640-11B) and the suppression chamber level (PI-1-1640-10B) instrumentation was the high voltage condition of their power supply.

RESULTS & EFFECTS ON SAFE OPERATION: As alternate indications for this instrumentation gave normal readings, safety implications were minimal.

ACTION TAKEN TO PREVENT REPETITION: The power supply was replaced with the voltage regulator allowing only 40 volts and 0.5 amps instead of 72 volts at 0.3 amps. This should preclude similar high voltage conditions in the future.

WORK REQUEST NO.: Q55243

LER NUMBER: N/A

COMPONENT: System 261 - Repaired acoustic monitor sensor cable to meet E.Q. requirements.

CAUSE OF MALFUNCTION: The cause of this event was improper documentation. The EQ document at the Station make no mention of how the acoustic monitors are to be spliced at the drywell penetrations.

RESULTS & EFFECTS ON SAFE OPERATION: Station Nuclear Engineering determined that the monitors would function properly in the event of an accident. Therefore, the safety consequences were minimal.

ACTION TAKEN TO PREVENT REPETITION: The Station informed the Station Nuclear Engineering Department of the EQ document discrepancies so they may be corrected. Also, the cable has been repaired to meet EQ requirements.

UNIT 2 MAINTENANCE SUMMARY

There were no Deviation Reports or License Events Report associated with the Safety Related Work Request Log this month.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

UNIT 1

Licensee Event Report Number	Date	Title of Occurrence
87-08	5-26-87	1C RHRSW pump - piping exceeds allowable stress
87-09	5-26-87	Cable Spread Room Fire Protection OOS
87-10	5-19-87	Control Room HVAC on Recirc

UNIT 2

87-07	5-20-87	Group III Isolation CU high temperature

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE JUNE 4 1987

COMPLETED BY KURT A. SCHMIDT

TELEPHONE 309-654-2241

OPERATING STATUS

0000 050187

1. Reporting period: 2400 053187 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	744.0	3560.6	106373.3
6. Reactor reserve shutdown hours	0.0	0.0	3121.9
7. Hours generator on line	744.0	3544.6	102861.1
8. Unit reserve shutdown hours.	0.0	0.0	909.2
9. Gross thermal energy generated(MWH)	1[illegible]2	8446472	217504290
10. Gross electrical energy generated(MWH)	577254	2791726	70545110
11. Net electrical energy generated(MWH)	552291	2670288	66117490
12. Reactor service factor	100.0	98.3	80.6
13. Reactor availability factor	100.0	98.3	83.2
14. Unit service factor	100.0	97.8	77.9
15. Unit availability factor	100.0	97.8	78.6
16. Unit capacity factor (Using MDC)	96.5	95.8	65.1
17. Unit capacity factor (Using Des.MWe)	94.1	93.4	63.5
18. Unit forced outage rate	0.0	.7	5.6

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254
UNIT ONE
DATE JUNE 4 1987
COMPLETED BY KURT A. SCHMIDT
TELEPHONE 309-654-2241

MONTH May 1987

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	747.8	17.	736.2
2.	534.5	18.	773.5
3.	640.9	19.	756.8
4.	719.0	20.	742.5
5.	791.8	21.	772.8
6.	764.5	22.	781.9
7.	753.8	23.	726.3
8.	770.7	24.	736.9
9.	761.8	25.	738.5
10.	763.4	26.	778.9
11.	764.2	27.	764.3
12.	773.5	28.	759.4
13.	729.7	29.	768.0
14.	778.8	30.	614.6
15.	786.5	31.	703.3
16.	752.3		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE JUNE 4 1987

COMPLETED BY KURT A. SCHMIDT

TELEPHONE 309-654-2241

OPERATING STATUS

0000 050187

1. Reporting period: 2400 053187 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	744.0	2995.7	100711.4
6. Reactor reserve shutdown hours	0.0	0.0	2985.8
7. Hours generator on line	744.0	2942.7	97641.8
8. Unit reserve shutdown hours.	0.0	0.0	702.9
9. Gross thermal energy generated(MWH)	1829.58	6930844	208270543
10. Gross electrical energy generated(MWH)	582729	2247813	66607105
11. Net electrical energy generated(MWH)	558022	2149618	62746655
12. Reactor service factor	100.0	82.7	76.8
13. Reactor availability factor	100.0	82.7	79.1
14. Unit service factor	100.0	81.2	74.5
15. Unit availability factor	100.0	81.2	75.0
16. Unit capacity factor (Using MDC)	97.5	77.2	62.3
17. Unit capacity factor (Using Des.MWe)	95.1	75.2	60.7
18. Unit forced outage rate	0.0	5.3	7.6

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup _____

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265
UNIT TWO
DATE JUNE 4 1987
COMPLETED BY KURT A. SCHMIDT
TELEPHONE 309-654-2241

MONTH May 1987

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	769.4	17.	719.0
2.	758.2	18.	780.0
3.	709.4	19.	737.7
4.	783.5	20.	739.5
5.	770.0	21.	756.3
6.	738.5	22.	735.9
7.	735.5	23.	707.0
8.	777.4	24.	714.1
9.	767.3	25.	717.6
10.	754.5	26.	791.9
11.	757.4	27.	727.3
12.	774.0	28.	748.9
13.	775.8	29.	765.7
14.	767.2	30.	722.1
15.	770.7	31.	739.7
16.	735.0		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily [illegible] power output sheet should be footnoted to explain the apparent anomaly

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 50-254

UNIT NAME QUAD CITIES UNIT ONE

COMPLETED BY K. Schmidt

DATE 6-2-87

REPORT MONTH MAY 1987

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
87-11	870502	S	0	H	4	------	RB	CONROD	Power reduction for control rod pattern adjustment
87-12	870530	S	0	H	4	------	RB	CONROD	Power reduction for control rod pattern adjustment

APPROVED
AUG 16 1982
QCOSR

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 50-265

UNIT NAME QUAD CITIES UNIT ONE

COMPLETED BY K. Schmidt

DATE 6-2-87

REPORT MONTH MAY 1987

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
----	------	---	-----	--	---	-------	---	-------	No shutdowns/reductions

APPROVED
AUG 16 1982
QCOSR

VI. UNIQUE REPORTING REQUIREMENTS

The followirg items are included in this report based on prior commitments to the commission:

A. MAIN STEAM RELIEF VALVE OPERATIONS

There were no Main Steam Relief Valve Operations for the reporting period.

B. CONTROL ROD DRIVE SCRAM TIMING DATA FOR UNITS ONE AND TWO

There was no Control Rod Drive Scram Timing Data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: 01 Reload: 8 Cycle: 9

2. Scheduled date for next refueling shutdown: 9-14-87

3. Scheduled date for restart following refueling: 12-7-87

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: YES. TECHNICAL SPECIFICATION CHANGES WILL BE REQUIRED FOR NEW FUEL TYPES (MAPHLGR CURVES) AND A LICENSE AMENDMENT TO MOVE SINGLE LOOP OPERATION INTO TECHNICAL SPECIFICATIONS.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   AUGUST 21, 1987

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   FIRST RELOAD OF GENERAL ELECTRIC, GE8E FUEL WITH 4 WATER-RODS AND LHGR LIMIT OF 14.4 KW/FT. FIRST CYCLE UTILIZING A DIFFUSION BASED CORE MONITORING CODE.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1573

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2008

APPROVED

APR 20 1978

Q.C.O.S.R.

QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: Q2 Reload: 8 Cycle: 9

2. Scheduled date for next refueling shutdown: 3-14-88

3. Scheduled date for restart following refueling: 5-22-88

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   DECEMBER 14, 1987

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1308

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2008

APPROVED

APR 20 1978

Q.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCIOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center

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Quad Cities Nuclear Power Station
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Telephone 309/654-2241

RAR-87-28

June 5, 1987

Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Document Control Desk

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of May, 1987.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

R. A. Robey

R. A. Robey
Services Superintendent

vk

Enclosure

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