



Duquesne Light

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June 1, 1987

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Reference: Beaver Valley Power Station, Unit No. 1
Docket No. 50-334, License No. DPR-66
Proposed Operating License Change Request No. 140

Gentlemen:

Enclosed are:

1. One (1) original and thirty-seven (37) copies of an Application for Amendment to our Operating License to permit the design of the reactor coolant pump and steam generator supports to be revised in accordance with this submittal.
2. Thirty-eight (38) copies of documents designated Attachments A, B, C and D which set forth the change in the Operating License (Attachment A) and its no significant hazards determination (Attachment B) as supported by an evaluation of the reactor coolant system with the proposed modified support configuration (Attachment C) and a leakage detection assessment (Attachment D).
3. Five (5) copies of "Technical Justification for Eliminating Large Primary Loop Pipe Rupture as the Structural Design Basis for Beaver Valley Unit 1", dated March 1987, WCAP-11317 (Proprietary).
4. Five (5) copies of "Technical Justification for Eliminating Large Primary Loop Pipe Rupture as the Structural Design Basis for Beaver Valley Unit 1", dated March 1987, WCAP-11318 (Non-proprietary).
5. A Westinghouse authorization letter, CAW-87-031, Proprietary Information Notice, and accompanying Affidavit.

As Item 3 contains information proprietary to Westinghouse Electric Corporation, it is supported by an affidavit signed by Westinghouse, the owner of the information. The affidavit sets forth the basis on which the information may be withheld from public disclosure by the Commission and addresses with specificity the considerations listed in paragraph (b)(4) of Section 2.790 of the Commission's regulations.

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Change:	EG&G	1	INP
	LPOR	1	INP
	NRC POR	1	INP
	NSIC	1	INP

Prop Encls
To: Reg Files - 2
P. Tam - 3
PDI-4 PO - 1

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Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10 CFR Section 2.790 of the Commission's regulations. Correspondence with respect to the proprietary aspects of the Application for Withholding or the supporting Westinghouse Affidavit should reference CAW-87-031 and should be addressed to R. A. Weisemann, Manager of Regulatory & Legislative Affairs, Westinghouse Electric Corporation, P. O. Box 355, Pittsburgh, Pennsylvania 15230-0355.

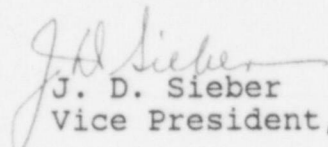
The proposed amendment will add a license condition stating that the design of the reactor coolant pump and steam generator supports may be revised in accordance with this submittal. This change will permit twenty-four (24) large bore snubbers installed solely to mitigate the dynamic effects of a reactor coolant system pipe rupture event to be permanently removed and six (6) additional large bore snubbers which have minimal thermal movement to be replaced with rigid struts.

This proposed amendment request has been reviewed by the OSC and ORC and has been determined to be safe and does not involve an unreviewed safety question. This proposed change to the reactor coolant system support load path design is being submitted for NRC review and approval in the form of a operating license amendment in accordance with the guidance provided in the NRC's response to Issue 5 in the Supplementary Information accompanying publication of the revised GDC-4 (51 FR 12504) dated April 11, 1986.

It is our intent to implement the proposed redesign of the reactor coolant system supports during the Beaver Valley Unit 1 sixth refueling outage which is currently scheduled for November, 1987. Therefore, timely NRC review and approval of this request is desired to assist us in planning for our upcoming outage. In order to support the NRC's review of this request, we propose a meeting with the NRC staff to discuss our request within three (3) weeks. We will contact our Project Manager, Mr. Peter S. Tam, to formalize a meeting time.

A check in the amount of \$150.00 is attached as payment of the license amendment application fee in accordance with 10 CFR 170.

Very truly yours,


J. D. Sieber
Vice President, Nuclear