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IE24
1/1

PERFORMANCE SUMMARY

June 1986

The following summary describes the significant operational activities for the month of June. In support of this summary, a chronological log of significant events is included in this report.

The units remained in an administrative shutdown the entire month due to design control review, configuration control updating and resolution of significant employee concerns. Outage related maintenance and modifications are being performed. Unit 1 has been off-line 312 days. Unit 2 has been off-line 313 days.

SIGNIFICANT OPERATIONAL EVENTS

Unit 1

Date	Time	Event
06/01/86	0001C	The reactor was in mode 5. The administrative shutdown due to design control review, configuration control updating and resolution of significant employee concerns continues.
06/30/86	2400C	The reactor was in mode 5. The administrative shutdown due to design control review, configuration control updating and resolution of significant employee concerns continues.

Unit 2

Date	Time	Event
06/01/86	0001C	The reactor was in mode 5. The administrative shutdown due to design control review, configuration control updating and resolution of significant employee concerns continues.
06/30/86	2400C	The reactor was in mode 5. The administrative shutdown due to design control review, configuration control updating and resolution of significant employee concerns continues.

FUEL PERFORMANCE

Unit 1

The core average fuel exposure accumulated during June was 0 MWD/MTU with the total accumulated core average fuel exposure of 0 MWD/MTU.

Unit 2

The core average fuel exposure accumulated during June was 0 MWD/MTU with the total accumulated core average fuel exposure of 8097.51 MWD/MTU.

SPENT FUEL PIT STORAGE CAPABILITIES

The total storage capability in the spent fuel pit (SFP) is 1,386. However, there are five cell locations which are incapable of storing spent fuel. Four locations (A10, A11, A24, A25,) are unavailable due to a suction strainer conflict and one location (A16) is unavailable due to an instrumentation conflict. Presently, there is a total of 348 spent fuel bundles stored in the SFP. Thus, the remaining storage capacity is 1,033.

PORVs AND SAFETY VALVES SUMMARY

No PORVs or safety valves were challenged in June 1986.

LICENSEE EVENT REPORT(S)

The following licensee event reports (LERs) were reported to the Nuclear Regulatory Commission in June 1986.

LER	Description of Event
1-86022	Two containment ventilation isolations (CVIs) occurred May 15, 1986 at 0140 CST (Unit 1) and at 0356 CST (Unit 2) respectively. The first event was caused by electromagnetic interference generated by relay vibration on the radiation monitor 1-RM-90-131 while placing containment purge in operation. The second event occurred on unit 2 while performing maintenance and trying to prevent a similar event as described above from transpiring. It was decided to remove power from radiation monitor 2-RM-90-106 to avoid a CVI. Personnel were unaware that the power loss would initiate the system to the safe position and actuate a CVI. A design change request will be initiated to modify the radiation monitoring system and special instructions will be issued for future reference.

LICENSEE EVENT REPORT(S)
(continued)

LER	Description of Event
1-86023	The Technical Review Committee discovered that the radioactive liquid effluent monitoring instrumentation channel calibration had not been performed per surveillance requirement (SR) 4.3.2.9.C.5.a, which is required once every 18 months. A procedure for the calibration of a flow indicator on the condensate demineralizer line was never instituted. However, a rotometer cannot be calibrated and if found inaccurate must be discarded. A channel check was performed to verify operability as required. "Calibration of Compliance Instruments", (SI-198), will ensure in the future that the TVA test facilities will verify the accuracy of all rotometers that will be installed.
1-86024	On May 25, 1986, the hourly firewatch required by technical specification 3.7.12 was not fully completed between the hours of 1300 and 1400 CDT.
2-86002	A containment ventilation isolation (CVI) occurred due to a cognitive personnel error while performing Surveillance Instruction (SI)-166.29 "Control Air Check Test During Cold Shutdown". This was the result of a low flow condition to the upper containment radiation monitor (2-RM-90-112) when the monitor's containment isolation valves were allowed to drift partially closed during the test. Arching of the low flow switch initiated a spike causing a high radiation signal. During the writing of the test procedure there was failure to anticipate the potential of a CVI. A revision to the procedure will be submitted to advise personnel that such potential effects exist during performance of this test.
2-86003	Two CVIs occurred due to spurious spikes on radiation monitors. RM-90-106B spiked due to a loose wire on the flow switch. RM-90-112 spiked due to welding noise (EMI).

SPECIAL REPORT(S)

The following special report was submitted to the NRC in June 1986.

Report #	Description
86-06	On June 17, 1986, at 1414 CST, all of the plant fire pumps were removed from service for 37 minutes. The pumps were removed from service to allow for the replacement of three hypochlorite valves (0-VLV-50-552, 523, and 521) that are located on one-inch lines close to the pumps. The replacement of the subject valves was due to excessive leakage. These valves are used to chlorinate the fire protection system in the control of Asiatic clams.

SPECIAL REPORT(S) (continued)

Report #	Description
86-06	The fire detection system was operable during the entire activity. Upon detection of a fire, the pumps could have been started manually and the fire suppression system pressurized. Before removing the pumps from service, communications were established between the main control room and the work area in accordance with plant procedures. The maintenance activity complied with technical specifications and plant procedures. Physical Security Instruction 13, "FIRE," was followed during the event.

OFFSITE DOSE CALCULATION MANUAL CHANGES

Offiste dose calculation manual changes were finalized April 14, 1986. A copy of the changes is found in section V.

OPERATIONAL SUMMARY

OPERATING STATISTICS
(NRC REPORTS)

OPERATING DATA REPORT

DOCKET NO. 50-327
DATE JULY 3, 1986
COMPLETED BY D.C.DUPREE
TELEPHONE (615)870-6544

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1 NOTES:
2. REPORT PERIOD: JUNE 1986
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS 3 THROUGH 7)SINCE LAST REPORT, GIVE REASONS: _____
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY(NET MWE): _____
10. REASONS FOR RESTRICTIONS, IF ANY: _____

		THIS MONTH	YR.-TO-DATE	CUMULATIVE
11.	HOURS IN REPORTING PERIOD	720.00	4343.00	43824.00
12.	NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	0.00	24444.91
13.	REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14.	HOURS GENERATOR ON-LINE	0.00	0.00	20871.13
15.	UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16.	GROSS THERMAL ENERGY GENERATED (MWH)	0.00	0.00	77060971.91
17.	GROSS ELECTRICAL ENERGY GEN. (MWH)	0.00	0.00	25976386.00
18.	NET ELECTRICAL ENERGY GENERATED (MWH)	-3045.00	-20381.00	24922356.00
19.	UNIT SERVICE FACTOR	0.00	0.00	54.47
20.	UNIT AVAILABILITY FACTOR	0.00	0.00	54.47
21.	UNIT CAPACITY FACTOR(USING MDC NET)	0.00	0.00	49.54
22.	UNIT CAPACITY FACTOR(USING DER NET)	0.00	0.00	49.54
23.	UNIT FORCED OUTAGE RATE	100.00	100.00	28.85

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): _____

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: STARTUP IS UNDETERMINED AT THIS TIME PENDING DESIGN CONTROL REVIEW, CONFIGURATION CONTROL UPDATING, AND RESOLUTION OF SIGNIFICANT EMPLOYEE CONCERNS.

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

SEQUOYAH NUCLEAR PLANT
AVERAGE DAILY POWER LEVEL

DOCKET NO. : 50-327
UNIT : ONE
DATE : JULY 3,1986
COMPLETED BY : D. C. DUPREE
TELEPHONE :(615)870-6544

MONTH: JUNE 1986

DAY	AVERAGE DAILY POWER LEVEL (MWe Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe Net)
01	0	16	0
02	0	17	0
03	0	18	0
04	0	19	0
05	0	20	0
06	0	21	0
07	0	22	0
08	0	23	0
09	0	24	0
10	0	25	0
11	0	26	0
12	0	27	0
13	0	28	0
14	0	29	0
15	0	30	0

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327
UNIT NAME Sequoyah One
DATE July 5, 1986
COMPLETED BY D. C. Dupree
TELEPHONE (615)870-6544

REPORT MONTH JUNE 1986

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	851220	F	720	F	9				Design Control, Configruation Updating, and Employee Concerns.

[1]F: Forced
S: Scheduled

[2]Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3]Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing Outage
5-Reduction
9-Other

[4]Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit I-Same Source

OPERATING DATA REPORT

DOCKET NO. 50-328
DATE JULY 3, 1986
COMPLETED BY D.C.DUPREE
TELEPHONE (615)870-6544

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2 NOTES:
2. REPORT PERIOD: JUNE 1986
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS 3 THROUGH 7)SINCE LAST REPORT, GIVE REASONS: _____
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY(NET MWE): _____
10. REASONS FOR RESTRICTIONS, IF ANY: _____

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.00	4343.00	35784.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	0.00	21984.54
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	0.00	0.00	21494.42
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.00	0.00	69127977.22
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	0.00	0.00	23536780.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	-5586.00	-28770.00	22603187.60
19. UNIT SERVICE FACTOR	0.00	0.00	60.07
20. UNIT AVAILABILITY FACTOR	0.00	0.00	60.07
21. UNIT CAPACITY FACTOR(USING MDC NET)	0.00	0.00	55.02
22. UNIT CAPACITY FACTOR(USING DER NET)	0.00	0.00	55.02
23. UNIT FORCED OUTAGE RATE	100.00	100.00	30.43

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
THE RESTART OF UNIT-2 HAS BEEN SCHEDULED FOR JANUARY 1987.

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

SEQUOYAH NUCLEAR PLANT
AVERAGE DAILY POWER LEVEL

DOCKET NO. : 50-328

UNIT : TWO

DATE : JULY 3,1986

COMPLETED BY : D. C. DUPREE

TELEPHONE :(615)870-6544

MONTH: JUNE 1986

DAY	AVERAGE DAILY POWER LEVEL (MWe Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe Net)
01	0	16	0
02	0	17	0
03	0	18	0
04	0	19	0
05	0	20	0
06	0	21	0
07	0	22	0
08	0	23	0
09	0	24	0
10	0	25	0
11	0	26	0
12	0	27	0
13	0	28	0
14	0	29	0
15	0	30	0

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-328
UNIT NAME Sequoyah Two
DATE July 5, 1986
COMPLETED BY D. C. Dupree
TELEPHONE (615)870-6544

REPORT MONTH JUNE 1986

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	850821	F	720	F	4				Design Control, Configuration Updating, and Employee Concerns.

[1]F: Forced
S: Scheduled

[2]Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3]Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing Outage
5-Reduction
9-Other

[4]Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit I-Same Source

OPERATING STATISTICS
(TVA REPORTS)

TVA 6560C (PP-3-76)

UNIT OUTAGE AND AVAILABILITY

Sequoyah Nuclear Plant

Licensed Reactor Power 3411 MW(th)

Unit No. Two

Generator Rating 1220.5 MW(e)

Design Gross Electrical Rating 1183 MW

Month/Year June 1986

Period Hours 720

Day	Time Unit Available						Time Not Available								Unit				OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION
	Total		Gen.		Not Used		Turbine		Gen.		Reactor		Unit		Time Out		Time In					
	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min				
1	00	00	00	00			24	00	24	00	24	00	24	00					Design Control, Configuration	N/A	Mode 5	
2	00	00	00	00			24	00	24	00	24	00	24	00					Updating, and Employee Concerns			
3	00	00	00	00			24	00	24	00	24	00	24	00								
4	00	00	00	00			24	00	24	00	24	00	24	00								
5	00	00	00	00			24	00	24	00	24	00	24	00								
6	00	00	00	00			24	00	24	00	24	00	24	00								
7	00	00	00	00			24	00	24	00	24	00	24	00								
8	00	00	00	00			24	00	24	00	24	00	24	00								
9	00	00	00	00			24	00	24	00	24	00	24	00								
10	00	00	00	00			24	00	24	00	24	00	24	00								
11	00	00	00	00			24	00	24	00	24	00	24	00								
12	00	00	00	00			24	00	24	00	24	00	24	00								
13	00	00	00	00			24	00	24	00	24	00	24	00								
14	00	00	00	00			24	00	24	00	24	00	24	00								
15	00	00	00	00			24	00	24	00	24	00	24	00								
16	00	00	00	00			24	00	24	00	24	00	24	00								
17	00	00	00	00			24	00	24	00	24	00	24	00								
18	00	00	00	00			24	00	24	00	24	00	24	00								
19	00	00	00	00			24	00	24	00	24	00	24	00								
20	00	00	00	00			24	00	24	00	24	00	24	00								
21	00	00	00	00			24	00	24	00	24	00	24	00								
22	00	00	00	00			24	00	24	00	24	00	24	00								
23	00	00	00	00			24	00	24	00	24	00	24	00								
24	00	00	00	00			24	00	24	00	24	00	24	00								
25	00	00	00	00			24	00	24	00	24	00	24	00								
26	00	00	00	00			24	00	24	00	24	00	24	00								
27	00	00	00	00			24	00	24	00	24	00	24	00								
28	00	00	00	00			24	00	24	00	24	00	24	00								
29	00	00	00	00			24	00	24	00	24	00	24	00								
30	00	00	00	00			24	00	24	00	24	00	24	00								
31																						
Total	00	00	00	00			720	00	720	00	720	00	720	00								

TVA 6560C (PP 3-76)

UNIT OUTAGE AND AVAILABILITY

Sequoyah Nuclear Plant

Licensed Reactor Power 3411 MW(th)

Unit No. One

Generator Rating 1220.5 MW(e)

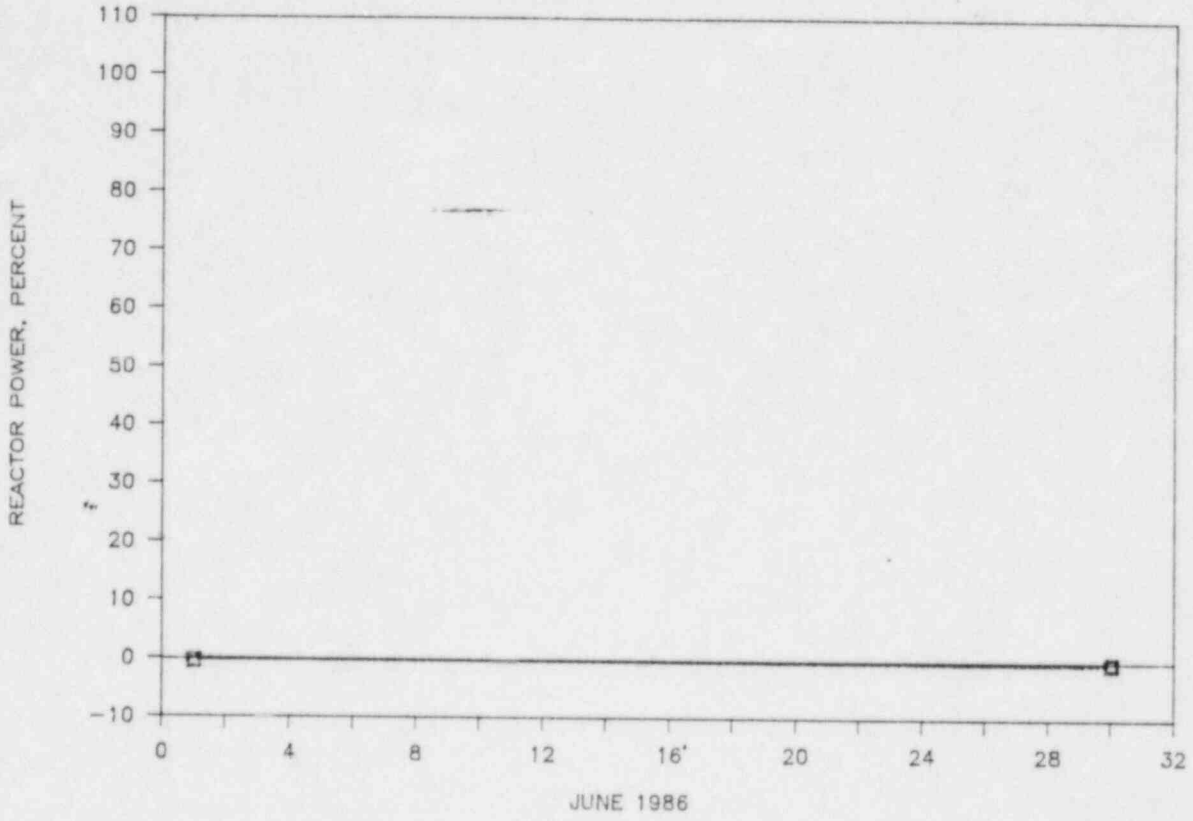
Design Gross Electrical Rating 1183 MW

Month/Year June 1986

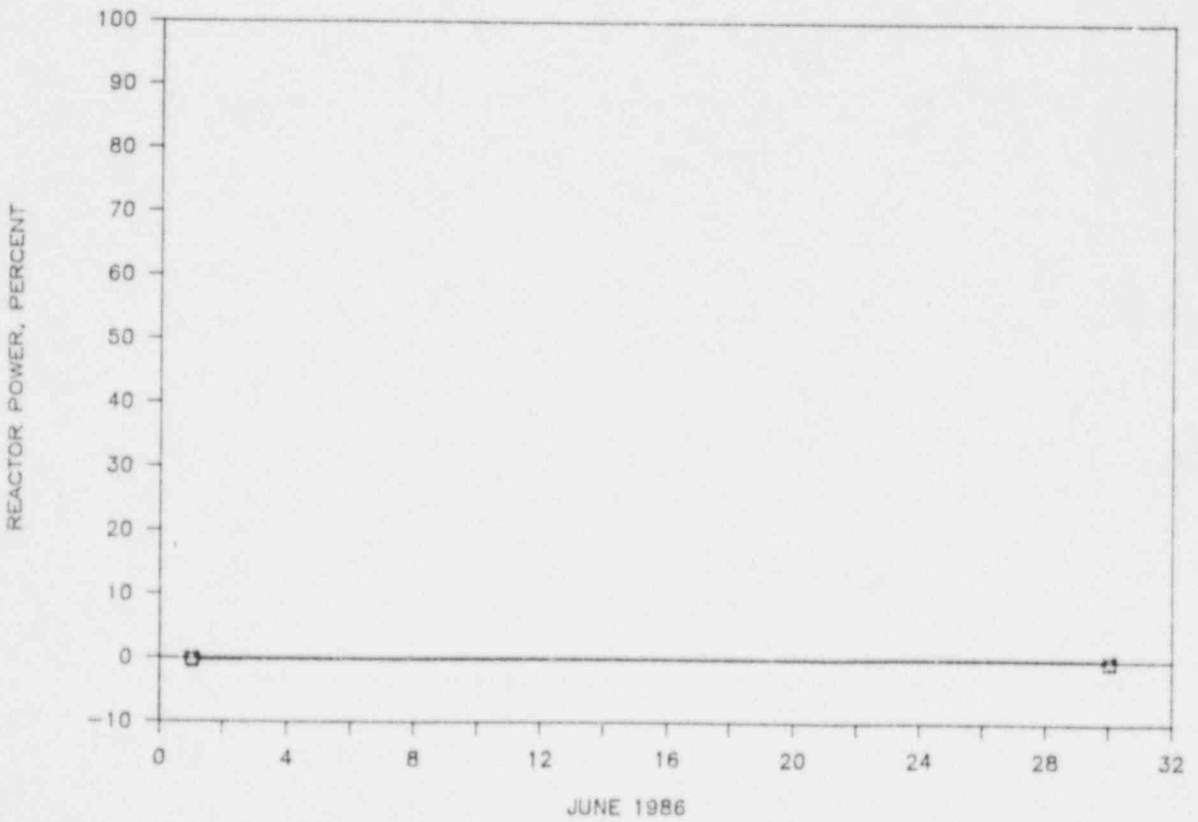
Period Hours 720

Day	Time Unit Available				Time Not Available				Unit		OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION
	Total	Gen.	Not Used	Turbine	Gen.	Reactor	Unit	Time Out	Time In					
	Hrs Min	Hrs Min	Hrs Min	Hrs Min	Hrs Min	Hrs Min	Hrs Min	Hrs Min	Hrs Min					
1	00 00	00 00		24 00	24 00	24 00	24 00			Design Control, Configuration Updating, and Employee Concerns	N/A	Mode 5	
2	00 00	00 00		24 00	24 00	24 00	24 00						
3	00 00	00 00		24 00	24 00	24 00	24 00						
4	00 00	00 00		24 00	24 00	24 00	24 00						
5	00 00	00 00		24 00	24 00	24 00	24 00						
6	00 00	00 00		24 00	24 00	24 00	24 00						
7	00 00	00 00		24 00	24 00	24 00	24 00						
8	00 00	00 00		24 00	24 00	24 00	24 00						
9	00 00	00 00		24 00	24 00	24 00	24 00						
10	00 00	00 00		24 00	24 00	24 00	24 00						
11	00 00	00 00		24 00	24 00	24 00	24 00						
12	00 00	00 00		24 00	24 00	24 00	24 00						
13	00 00	00 00		24 00	24 00	24 00	24 00						
14	00 00	00 00		24 00	24 00	24 00	24 00						
15	00 00	00 00		24 00	24 00	24 00	24 00						
16	00 00	00 00		24 00	24 00	24 00	24 00						
17	00 00	00 00		24 00	24 00	24 00	24 00						
18	00 00	00 00		24 00	24 00	24 00	24 00						
19	00 00	00 00		24 00	24 00	24 00	24 00						
20	00 00	00 00		24 00	24 00	24 00	24 00						
21	00 00	00 00		24 00	24 00	24 00	24 00						
22	00 00	00 00		24 00	24 00	24 00	24 00						
23	00 00	00 00		24 00	24 00	24 00	24 00						
24	00 00	00 00		24 00	24 00	24 00	24 00						
25	00 00	00 00		24 00	24 00	24 00	24 00						
26	00 00	00 00		24 00	24 00	24 00	24 00						
27	00 00	00 00		24 00	24 00	24 00	24 00						
28	00 00	00 00		24 00	24 00	24 00	24 00						
29	00 00	00 00		24 00	24 00	24 00	24 00						
30	00 00	00 00		24 00	24 00	24 00	24 00						
31													
Total	00 00	00 00		720 00	720 00	720 00	720 00						

SEQUOYAH ONE REACTOR HISTOGRAM



SEQUOYAH TWO REACTOR HISTOGRAM



SUMMARY OF MAINTENANCE ACTIVITIES

MAINTENANCE SUMMARY
(ELECTRICAL)

ELECTRICAL MAINTENANCE MONTHLY SUMMARY 07-14-86 PAGE 1

MR.COMP	U	FUNC SYS	ADDRESS.	COMP DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
B105193	1	GENB 082	0001B	01/31/86	1-GENB-082-0001B-B,[*NPRD*],1BB D/G BARELY PASSES SI 7 TS 3.8.1.2 CRITERIA OF REACHING 60HZ+OR-1.2 IN 10 SEC.LAST TEST TOOK 9.9 SEC.INVESTIGATE & IMPROVE START TIME TO PREVENT FUTURE FAILURES OF SURVEILLANCE TESTS	1-GENB-82-1B-B IDLE SPEED SETTING WAS SET TOO LOW TO MEET TEST CRITERIA. ADJUSTED PNEUMATIC BOOSTER AND IDLE SPEED SETTING PER THE VENDOR INSTR ALSO ADJUSTED MINIMUM-MAX EXCITATION CURRENT LEVEL POINT PER VENDORS MANUAL MR B105193
B105202	1	ZS 063	0071	2 02/11/86	1-ZS-063-0071 2-A,[*NPRD*,*10CFR50.49*],CLOSED LIGHT SOCKET WILL NOT GIVE INDICATION OF CLOSED POSITION SUSPECT PROBLEM WITH LIGHT SOCKET	1-ZS-63-71 2-A LIMIT SWITCH MOUNTING BRACKET WAS BENT PREVENTING LIMITS FROM MAKING UP. STRAIGHTENED LIMIT SWITCH MOUNTING BRACKET AND CHECKED FOR PROPER LIGHT INDICATION. MR B105202
B105470	1	GENB 082	0001A	01/31/86	1-GENB-082-0001A-A,[*NPRD*],INVESTIGATE VOLT. & FREQ NOT WITHIN TECH SPEC LIMITS DURING INITIAL START F/AMBIENT CONDITIONS >10 SECS PERIODICALLY	1-GENB-82-1A-A EXCITATION CONTROL TIME DELAY TRIP CURRENT LEVEL WAS OUT OF ADJUSTMENT. ADJUSTED THE MINIMUM-MAX EXCITATION CONTROL TIME DELAY TRIP CURRENT LEVEL TO MINIM SET POINT PER VALIDATED INSTRUCTION MANUAL MR B105470
B105492	2	BCTA 202	CJ /2	01/22/86	2-BCTA-202-CJ /2-,[*NPRD*],COIL BURNED IN BKR WOULD NOT TRIP	2-BCTA-202-CJ/2 TRIP COIL HAD BURNED PREVENTING BREAKER FROM TRIPPING. REPLACED TRIP COIL AND CHECKED FOR PROPER OPERATION. WR B105492
B111744	1	GENB 082	0001A	02/05/86	1-GENB-082-0001A-A,HS-82-165 CONTACT APPEAR TO MAKE UP INTERMITTANTL.	0-HS-82-165-A ROTARY SWITCH WAS BAD DUE TO AGE OR CYCLIC FATIGUE. REPLACED ROTARY SWTICH AND CHECKED FOR PROPER OPERATION MR B111744
B111745	2	GENB 082	0002A	02/05/86	2-GENB-082-0002A-A,HAND SWITCH 82-195 CONTACTS APPEAR TO MAKE UP INTERMITTANTLY	0-HS-82-225-A ROTARY SWITCH WAS BAD DUE TO AGE AND CYCLIC FATIGUE. REPLACED ROTARY SWITCH AND CHECKED FOR PROPER OPERATION MR B11745
B113221	2	GENB 082	0002B	02/18/86	2-GENB-082-0002B-B,[*NPRD*],ENG 1 IMMERSION HTR CONTACTOR CONTACTS ARE BURNED AND PITTED. DG 721	2-GENB-82-2B-B IMMERSION HEATER CONTACTOR CONT ACTS WERE BURNED AND PITTED DUE TO AGE. REPLACED CONTACTS IN HEATER CONTROLLER CONTACTER. MR B113221
B113222	2	GENB 082	0002B	02/18/86	2-GENB-082-0002B-B,[*NPRD*],ENG 2 IMMERSION HTR CONTACTOR CONTACTS ARE BURNED AND PITTED. DG EL 721	2-GENB-82-2B-B IMMERSION HEATER CONTACTOR CONT ACTS WERE BURNED AND PITTED DUE TO AGE, OR NORMAL WEAR. REPLACED CONTACTS IN HEATER CONTROLLER CONTACTOR MR B113222
B115938	1	HS 030	0451C	06/09/86	1-HS-030-0451C-A,WHEN YOU GO TO START POSITION WITH HS IT WILL NOT START FAN WHEN YOU GO TO STOP POSITION WITH HS IT WILL NOT STOP FAN INVEST AND REPAIR AS NEEDED	CONTACT IN SWITCH WAS BROKEN. DISASSEMBLED AND REPLACED ROTOR CAM B115938
B118255	1	XS 063	0334C	05/27/86	1-XS-068-0334C-B,HANDSW BLACK GRIP IS BROKEN. REPLACE HS	POSSIBLY, SWITCH HANDLE BROKE FROM WEAR. REPLACED SWITCH HANDLE WR B118255
B121188	0	MTRB 078	0020	05/28/86	0-MTRB-078-0020-,[*NPRD*],REPAIR RUPP	PUMP MOTOR 'B' WAS BAD. REF WR B117263

WR,COMP	U	FUNC SYS ADDRESS,	COMP DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
				MTR B WITH PARTS FROM RWPP MTR A FEF WR B121176	FOR WORK ON PUMP, REMOVED MOTOR SHAFT AND ROTOR AND REPLACED WITH THE SHAFT FROM O-MTRB-78-19 AND CHECKED FOR PROPER OPERATION, WR B121188

1 records listed.

MAINTENANCE SUMMARY
(INSTRUMENTATION)

INSTRUMENT MAINTENANCE MONTHLY REPORT FOR JUNE 1986

UNIT 1

During performance of SI-102, fuel oil pressure indicators, PI-18-68/2 and PI-18-83/2, were determined to be inoperable because of clogged snubbers (PRO 1-86-131). The snubbers were replaced and the instruments were returned to service.

UNIT 2

PRO-2-86-044 was initiated when EMI from steam generator U-tube welding caused a containment vent isolation signal to be generated from radiation monitor loop 2-R-90-112.

During channel calibration testing of steam generator level loop L-3-93, LT-3-93 was found outside the desired tolerance. An evaluation of all loop component "as found" values revealed that the technical specifications allowable value for LS-3-93-B was not satisfied. PRO-2-86-045 was initiated and the loop was recalibrated.

During performance of SI-484, instrument mechanics discovered a partial loss of fill fluid in the train A upper plenum sense line for the Reactor Vessel Level Instrumentation System. Work is currently underway to determine cause and necessary corrective action.

Prepared special maintenance instruction SMI-2-90-1 to perform noble gas calibration for the shield building postaccident radiation monitors.

COMMON

Completed Instrument Maintenance Section's responsibilities for replacement of parts on the units 1 and 2 hydrogen analyzers to maintain environmental qualification.

Initiated PRO-1-86-145 to evaluate possible problems with the main feedwater control valves not meeting the required technical specification response times for feedwater isolation.

Completed review of preliminary setpoint methodology report. Coordination with Westinghouse to resolve comments and correct problem areas is currently underway.

Completed Workplan 12056 to correct drawing discrepancies (as described in Discrepancy Report No. SQ-DR-86-03-082R) with time delay relay settings for control room isolation signal.

Initiated program to verify the technical adequacy of all surveillance instructions required by technical specifications. The problem involves a technical review, revision, and reperformance (as required) of each instruction, and is scheduled for completion before startup.

WR#	U	FUNC SYS	ADDRESS	COMP DATE	DESCRIPTION	CORRECTIVE ACTION
B124605	1	099		06/30/86	1--099--,RECORD THE SSPS OUTPUT VOLT IN THE SW GEAR COMPARTMENT FOR TRIP CHANNELS A AND B FOR THE REACT TRIP AND BYPASS REACT TRIP BKRS	NONE. RECORD VOLT. WR B124605
B124606	2	SSPS 099		06/30/86	2-SSPS-099--,RECORD THE SSPS OUTPUT VOLT IN THE SW GEAR COMPARTMENT FOR TRIP CHANNELS A AND B FOR THE REACT TRIP AND BYPASS REACT TRIP BKRS	NONE. RECORD VOLT. WR B124606
B131909	1	PS 082	200	06/11/86	1-PS-082-200-,SW DEFECTIVE REPLACE AND CALIB PER SI 170.2	DEFECTIVE. REPLACED
B132371	1	PT 063	62	06/30/86	1-PT-063-62-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. VERIFY MOUNTING. NOT NPRDS REPORTABLE. WR B132371

4 records listed.

MR HIST U	FUNC SYS	SOURCE	COMP DATE	DESCRIPTION	CORRECTIVE ACTION
B102684 1	LIC 003	156	06/03/86	1-LIC-003-156-A,C*APRD*],INDICATOR PEGGED HIGH WILL NOT DRIVE	DEFECTIVE FEEDBACK POTENTIOMETER REPLACED FEEDBACK POTENTIOMETER RECALIBRATED CONTROLLER AND RETURNED TO SERVICE WR B102684
B113829 0	RM 090	122A	06/04/86	0-RM-090-122A-,LOW FLOW SW SHOWS FLOW W/NO FLOW THROUGH NOW REPAIR OR REPLACE AS REQ	DIRTY FLOW SWITCH CLEANED FLOW SWITCH WR B113829
B116688 0	RM 090	S/R326 2	06/04/86	0-RM-090-S/R326-28-,REPAIR HIGH VOLTAGE	DEFECTIVE A2031 REPLACED A203
B120569 1	PC 001	SPARE	06/11/86	1-PC-001-SPARE-,FUNCTIONALLY TEST AND RETURN TO POWER STORES	NONE. FUNCTIONALLY TEST RETURN TO POWER STORES WR B120569
B124738 1	LR 003	43POWL	06/12/86	1-LR-003-43POWL-K,RED PEN MISSING. SIM GEN #1 LVL RECORDER	PEN FELL OFF, OUT OF CALIBRATION REPAIRED RECALIBRATED WR B124738
B131166 0	RM 090	1336	06/02/86	0-RM-090-1336-A,CHECK CALIBRATION ON MONITOR RED TRIP LIGHT COMES ON EVERY FEW MINUTES	HI BACKGROUND RADIATION CHANGED SET POINT WR B131166
B131399 2	FR 002	35	06/12/86	2-FR-002-35-,CHARTED#3,THE PEN STICKS PLEASE INVEST AND REPAIR	DIRTY DRIVE GEARS CLEAN AND LUBRICATED THE DRIVE GEARS AND RETURNED TO SERVICE WR B131399
B131964 2	PS 003	160A	06/19/86	2-PS-003-160A-B,REPLACE EXISTING 1/4" PIPE TO 1/4" MALE SWAGELOCK TUBING ADAPTER W/COMPATIBLE SHORT PARKER EQUIV AND REPLACE SWAGELOCK FEMALE NUTS ON SENSE LINE FROM ABOVE FITTING DOWN TO SH PARKER 'TEE'	UNKNOWN. INSTALL TUBING CORRECTLY WR B1319964 NOT NPRDS REPORTABLE
B131965 2	PS 003	136B	06/19/86	2-PS-003-136B-B,REPLACE 1/4" SWAGELOCK CAP ON TEST "TEE" FITTING W 1/4" SH PARKER EQUIVALENT	UNKNOWN. INSTALL CORRECT CAP WR B131965 NOT NPRDS REPORTABLE
B131966 2	PS 003	160A	06/19/86	2-PS-003-160A-B,REPLACE EXISTING 1/4" PIPE TO 1/4" MALE SWAGLOCK TUBING FITTING W/COMPATIBLE SHORT PARKER EQUIVALENT DISCOVERED DURING PERF OF SI 207	UNKNOWN. INSTALL CORRECT FITTINGS WR B131966 NOT NPRDS REPORTABLE
B132368 1	FT 003	103B	06/05/86	1-FT-003-103B-E,VERIFY MOUNTING PER SAI-8-317-26	UNKNOWN TORQUED BOLTS NOT NPRDS REPORTABLE WR B132368

11 records listed.

WR COMP U	FUNC SYS	ADDRESS	COMP DATE	DESCRIPTION	CORRECTIVE ACTION
B100055 2 FN	030		06/23/86	2-FN-030--,CONTROLLER DOES NOT APPEAR TO INCREASE THE SAMPLE VELOCITY TO MATCH THE STACK VELOCITY	OUT OF CALIBRATION. RECALIBRATED WR B100055
B100463 1 PS	003	121A	06/17/86	1-PS-003-121A-,VERIFY MOUNTING PER TOA 0x63 47W600-65 47W600 293 DET E293	UNKNOWN VERIFIED MOUNTING WR B100463
B102677 2 PX	003	147	06/27/86	2-PX-003-147-,[**NPRD**],INSTALL PROTECTIVE SHIELD IN POWER SUPPLY	NO TYPE OF SHIELDING ON THE UNDERSIDE OF THE POWR SUPPLY. INSTALLED PROTECTIVE SHIELD. NOT NPRD REPORTABLE
B102678 2 PX	003	155	06/26/86	2-PX-003-155-,INSTALL PROTECTIVE SHIELD IN POWER SUPPLY	UNKNOWN. INSTALL PROTECTIVE SHIELD WR B102678 NOT NPRDS REPORTABLE
B102679 2 PX	003	163	06/27/86	2-PX-003-163-,[**NPRD**],INSTALL PROTECTIVE SHIELD IN POWER SUPPLY	NO PROTECTIVE SHIELDING ON THE UNDERSIDE OF THE POWER SUPPLY. INSTALLED PROTECTIVE SHIELDING. NO NPRDS REPORTABLE
B102680 2 PX	003	170	06/26/86	2-PX-003-170-,INSTALL PROTECTIVE SHIELD IN POWER SUPPLY	UNKNOWN. INSTALL PROTECTIVE SHIELD. NOT NPRD REPORTABLE WR B102680
B102695 1 PDS	043	0200	06/05/86	1-PDS-043-0200-A,[**10CFR50.49**],REPLACE PDS TO START NEW QUALIFIED LIFE FOR THE DEVICE	END OF QUALIFIED LIFE REPLACED PRESSURE SWITCH WR#B102695
B104117 1 LI	003		06/06/86	1-LI-003--,INITIATE APPLICABLE PORTIONS OF SI 604.1 TO BACKFILL LEGS OF S/G LVL INSTRUMENTATION.	SENSING LINES NEEDED BACKFILLING. BACKFILLED SENSING LINES NOT NPRDS REPORTABLE WR B104117
B115562 1 TE	068	250	06/06/86	1-TE-068-250-,[**10CFR50.49**],INSTRUMENT WAS TORQUED ON WR A30L670 WHICH USED TORQUE WRENCH E04039. THIS RTE DOES NOT CLICK WHEN TORQE VLU IS REACHED	UNKNOWN. RETORQUED NOT NPRDS REPORTABLE WR# B115562
B119697 2 CHR	313	450	06/23/86	2-CHR-313-450-,*I* COND WON'T START-COMPR AND AHU OPERATING. RM AT PRESENT IS 100 DEG F. AUX BLD APPROX 759' ROOF	2-PS-313-805 OUT OF CALIBRATION. RECALIBRATED SWITCH WR B119697
B120556 1 LT	003	94	06/24/86	1-LT-003-94-,[**10CFR50.49**],REPLACE MISSING HIGH AND LOW SIDE DRAIN VALVE HANDLES.	UNKNOWN. REPLACED HANDLES. NOT NPRDS REPORTABLE WR B120556
B120561 0 PC	001	73	06/05/86	0-PC-001-73-,HIGH AND LO LIMITS DO NOT HAVE PROPER AFFECT ON CONTROLLER	DEFECTIVE Q7 AND Q8 TRANSISTORS AND CR7 DIODE. REPLACE TRANSISTORS AND DIODE RECALIBRATED AND RETURNED B120561
B120563 1 PX	068	69	06/05/86	1-PX-068-69-,REPLACE EXISTING A/C CABLE WITH LOCKING TYPE A/C CABLE PLUG	DESIGN OF THE EQUIPMENT MODIFY RECEPTACLE TO ACCEPT A LOCKING TYPE PLUG NOT NPRDS REPORTABLE WR B120563
B120568 1 PT	001	72	06/17/86	1-PT-001-72-,REMOVE WATER FROM XMTR TERMOBLOCK AND SEAL CONDUIT TO PREVENT	UNKNOWN SEAL CONDUIT DRY OUT TRANSMITTER RETERMINATE BROKEN WIRE

WR.COMP	U	FUNC	SYS	ADDRESS	COMP DATE	DESCRIPTION	CORRECTIVE ACTION
						WATER FROM ENTERING TERM BLOCK	CALIBRATE WR B120568 NOT RPRDS REPORTABLE
B120574	1	FM	001		21F 06/19/86	1-FM-001-21F-,[*RPRD*],MODULE WOULD NOT CALIBRATE PER MFG INSTRUCTIONS DURING PERFORMANCE FOR WP10845	RESISTORS R52 R54 R55 R56 WERE FOUND TO BE OUT OF TOLERANCE REPLACED RESITORS R52 R54 R55 R56 RECALIBRATED DEVICE AND RETURNED TO SERVICE WR B120574
B120579	1	XM	092		5003 06/20/86	1-XM-092-5003-,P6 PERMISSIVE RESET VLV OUT OF TOLERANCE ON SI 93.2	OUT OF CALIBRATION. RECALIBRATED WR B120579
B120593	2	FS	030		242 06/25/86	2-FS-030-242-,SW WAS FND OUT OF TOLERANCE DURING SI 249.2 INI 30 PG 112	SETPOINT DRIFT. RECALIBRATED SWITCH
B120621	0	PS	082		240 06/20/86	0-PS-082-240-,COMPR WON'T OPER IN AUTO. POSITION AIR PRESS ON MAIN TNK WAS DOWN TO 180 PSI FND ON SI 102	BLOWN FUSE. REPLACED FUSE. WR B120621
B120622	0	PS	082		241 06/20/86	0-PS-082-241-,COMPR WON'T OPER IN AUTO POSITION PRESS IN MAIN TNK WAS AT 230 PSI. FND ON SI 102	BLOWN FUSE. REPLACED FUSE WR B120622
B120725	0	LS	087	SPARE	06/12/86	0-LS-087-SPARE-,TEST SPARE SW FOR STATIC SHIFT	NONE. PERFORMED TEST WR B120725
B120742	0	RM	090		134 06/24/86	0-RM-090-134-,NO SIGNAL TO MON	BROKEN CONNECTION. RESOLDERED CONNECTION. WR B120742
B121309	1	RM	090		112 06/18/86	1-RM-090-112-,*IN INSTRUMENT MALF WON'T CLEAR.	UNKNOWN ALARMS WERE FOUND CLEAR NONE WR B121309
B121841	2	HZE	043		200 06/26/86	2-HZE-043-200-,[*10CFR50.49*],REPLACE O-RINGS IN CELL	END OF QUALIFIED LIFE. INSTALLED NEW O-RING
B121851	2	POS	003		173 06/20/86	2-POS-003-173-,[*10CFR50.49*],COORD THIS WR W/WR B106054 AND REPLACE 2 LCO 3 173 POSITIONER LEADS	UNKNOWN. REPLACED HARNESS. NOT RPRDS REPORTABLE WR B121851
B121900	2	HS	063		133A 06/27/86	2-HS-063-133A-,PERFORM SMI 0 63 1 TO TEST MANUAL SI HANDSW TO MEET NRC COMMITTMENT CONTROL NO NCO 86 0195 001 DUE 6/30/86 IE INSPECT REPORT 327 328/86-20-04	NONE. PERFORMED TEST TO MEET NRC COMMITTMENT WR B121900
B124167	1	XE	092		5001 06/25/86	1-XE-092-5001-B,SRM N-31 IS DRIFTING HIGH	UNKNOWN. MONITORED CHANNEL FOR SEVERAL AND NO ABNORMALITIES WERE NOTED
B124191	1	TI	068		398 06/23/86	1-TI-068-398-,[*RPRD*],THE TI WILL INTERMITTANTLY RISE TO >300 DEGREE AND THEN DROP BACK DOWN TO APPROX 120 DEGREE	UNKNOWN. NO PROBLEM FOUND. NOT RPRDS REPORTABLE WR B124191
B124196	0	RR	090		206 06/04/86	0-RR-090-206-,CHECK CALIB ON RECORDER SUSPECT RECORDER MAY BE OUT OF CALIB	OUT OF CALIB. RECALIB
B124243	1	PT	068		311C 06/24/86	1-PT-068-311C-,PERFORM A FIELD VERIFICATION ON THE TRANSMITTER PER	NONE. NONE. NOT RPRDS REPORTABLE B124243

WR	COMP ID	FUNC	SYS	ADDRESS	COMP DATE	DESCRIPTION	CORRECTIVE ACTION
						AI-19 PART IV ATT 7	
B124240	2	PI	068	311C	06/17/86	2-PI-068-311C-,PERFORM A FIELD VERIFICATION SO THE TRANSMITTER PER AI-19 PART IV ATT 7	OBTAIN NEW DATA AND UPDATE EQIS NOT NPRDS REPORTABLE WR B124240
B125380	2	RE	090	100	06/27/86	2-RE-090-100-,IODINE FLOW ALARM PICKED UP AND IODINE FLOW IS FLUCTUATING BETWEEN 0 AND 6	IODINE FLOW SWITCH OUT OF CALIBRATION. CALIBRATED FLOW SWITCH PER SI 206.2
B125651	2	XI	092	5001	06/18/86	2-XI-092-5001-,WHILE PERFORMING SI 93.2 HIGH FLUX AT SHUTDOWN ALARM XA 55 4B 8 AND CNTMT HORN DID NOT ACTUATE	TRAIN "A" SSPS OUT OF SERVICE NONE WR B125651
B125736	0	RM	090	133A	06/27/86	0-RM-090-133A-, YELLOW LIGHT IS IN S1204	LOW BACKGROUND. SETPOINT TOO HIGH LOWERED SETPOINT
B125737	0	RM	090	140A	06/27/86	0-RM-090-140A-, YELLOW LIGHT IS IN SI 204	LOW BACKGOUND. SETPOINT TOO HIGH LOWERED SETPOINT
B130619	0		032	VARIOUS	06/19/86	0--032-VARIOUS -,CHECK FOR AND REPAIR LEAKS IN SENSOR LINE CONN UP TO AND INSIDE PNL 0 L 321	LOOSE FITTINGS TIGHTENED FITTINGS WR B130619
B131167	0	PCV	026	15	06/13/86	0-PCV-026-15-,PRESSURE WAS 180 PSI AT DISCHARGE OF PUMPS WITH 1AA AND 1BB FIRE PUMPS RUNNING SUSPECT PCV 26-15 NOT FUNCTIONING PROPERLY INVES AND REPAIR	DEFECTIVE POSITIONER. REPAIRED POSITIONER WR B131167
B131176	2	RM	090	106	06/06/86	2-RM-090-106-B,CHECK RAD MON FOR CAUSE OF SPIKE THAT RESULTED IN CONT VENT ISOL NEEDED TO ANSWER PRO 2 86 043	LOOSE ELECTRICAL CONNECTION ON IODINE FLOW SWITCH TIGHTENED LOOSE CONNECTION WR B131176
B131182	2	FI	072	34	06/26/86	2-FI-072-34-,CONT SPRAY PUMP 2A-A FLOW INDICATOR SHOWING APPROX 1000 GPM FLOW	FLOW TRANSMITTER SENSING LINES NEEDED BACKFILLING. BACKFILLED SENSING LINES
B131358	0	RR	090	103	06/24/86	0-RR-090-103-,BLUE PEN ON RECORDER IS NOT MOVING WHEN MONITOR IS SOURCE CHECKED	LOOSE CONNECTION ON RECORDER TIGHTENED CONNECTION WR B131358
B131363	2	RM	090	112	06/17/86	2-RM-090-112-,PULL CVI RELAYS FOR 2 RM 90 112 A AND B REINSTL WHEN WR 122280 IS COMPLETE	NONE REINSTALLED WR B131363
B131382	0	RM	090	122	06/20/86	0-RM-090-122-,MON LOW FLOW ALARM STICKS OCCASIONALLY	DIRTY SWITCH. CLEANED SWITCH WR B131382
B131963	2	FIC	046	57	06/18/86	2-FIC-046-57-,CONTROLLER DOESN'T SWITCH MODES AUTO AND MANUAL WHEN SWITCHED IN CONTROL ROOM	CONTROLLER OUT OF CALIBRATION RECALIBRATED WR B131963
B132258	1	FIC	046	0057	06/03/86	1-FIC-046-0057-S,OBTAIN NAME PLATE DATA FOR THE ABOVE LISTED DEVICE	UNKNOWN. UPDATE EQIS.
B132261	1	LIC	003	VARIOUS	06/03/86	1-LIC-003-VARIOUS-,OBTAIN NAME PLATE DATA FOR 1LIC-3-140 156 164 171 172 173	UNKNOWN. UPDATE EQIS. NOT NPRDS REPORTABLE.

MR.COMP	U	FUNC	SYS	ADDRESS	COMP DATE	DESCRIPTION	CORRECTIVE ACTION
						174 175	
B132351	2	PT	001	5	06/06/86	2-PT-001-5-G,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPROS REPORTABLE WR B132351
B132358	1	PT	001	30	06/06/86	1-PT-001-30-G,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLT NOT MPROS REPORTABLE WR B132358
B132361	1	FT	003	35A	06/06/86	1-FT-003-35A-D,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPROS REPORTABLE WR B132361
B132362	1	FT	003	35B	06/06/86	1-FT-003-35B-E,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPROS REPORTABLE WR B132362
B132363	1	FT	003	48A	06/06/86	1-FT-003-48A-D,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUE BOLT NOT MPROS REPORTABLE WR B132363
B132364	1	FT	003	48B	06/06/86	1-FT-003-48B-E,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPROS REPORTABLE WR B132364
B132365	1	FT	003	90A	06/06/86	1-FT-003-90A -D,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUE BOLTS NOT MPROS REPORTABLE WR B132365
B132366	1	FT	003	90B	06/06/86	1-FT-003-90B-E,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPROS REPORTABLE WR B132366
B132367	1	FT	003	103A	06/06/86	1-FT-003-103A-D,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUE BOLTS WR B132367 NOT MPROS REPORTABLE
B132532	1	FT	072	0013	06/26/86	1-FT-072-0013-B,PIS CALIBRATE FLOW INDICATOR 1-FI-72-13 FOR PERFORMANCE OF SI-37	NONE. VERIFIED CALIBRATION RETURNED TO SERVICE. NOT MPROS REPORTABLE WR B132532
B132751	2	PT	001	9A	06/02/86	2-PT-001-9A-D,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN.. TORQUED BOLTS. NOT MPROS REPORTABLE.
B132752	2	PT	001	9B	06/02/86	2-PT-001-9B-E,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT MPROS REPORTABLE
B132753	2	PT	001	12	06/06/86	2-PT-001-12-F,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPROS REPORTABLE WR B132753
B132754	2	PT	001	20A	06/02/86	2-PT-001-20A-D,VERIFY MOUNTING PER SMI-0-316-26	UNKNOWN. TORQUED BOLT. NOT MPROS REPORTABLE.
B132755	2	PT	001	20B	06/02/86	2-PT-001-20B-E,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUED BOLTS. NOT MPROS REPORTABLE
B132756	2	PT	001	23	06/06/86	2-PT-001-23-,VERITY THE MOUNTINGF PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPROS REPORTABLE WR B132756
B132757	2	PT	001	30	06/06/86	2-PT-001-30-G,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPRO REPORTABLE WR B132757
B132758	2	PT	001	72	06/06/86	2-PT-001-72-E,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPROS REPORTABLE WR B132758
B132759	2	PT	001	73	06/06/86	2-PT-001-73-D,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT MPROS REPORTABLE WR B132759
B132769	2	PT	030	30C	06/06/86	2-PT-030-30C-,VERIFY MOUNTING PER	UNKNOWN TORQUED BOLTS WR B132769

WR	COMP ID	FUNC	SYS	COMP ADDRESS	DATE	DESCRIPTION	CORRECTIVE ACTION
						SMI-0-317-26	
B132770	2	PT	063	61	06/03/86	2-PT-063-61-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUED BOLTS. NOT NPRDS REPORTABLE. WR B132770
B132771	2	PT	063	62	06/03/86	2-PT-063-62-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE WR B132771
B132772	2	PT	063	86	06/04/86	2-PT-063-86-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUED BOLTS. WR B132772 NOT NPRDS REPORTABLE
B132773	2	PT	063	88	06/04/86	2-PT-063-88-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE. WR B132773
B132774	2	PT	063	106	06/06/86	2-PT-063-106-,VERIFY MOUNTING PER SMI-0-317--26	UNKNOWN TORQUED BOLTS NOT NPRDS REPORTABLE WR B132774
B132775	2	PT	063	108	06/06/86	2-PT-063-108-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT NPRDS REPORTABLE WR B 132775
B132776	2	PT	063	126	06/03/86	2-PT-063-126-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE.. WR B132776
B132777	2	PT	063	128	06/03/86	2-PT-063-128-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE. WR B132777
B132778	2	FT	068	6A	06/04/86	2-FT-068-6A-D,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE. WR B132778
B132779	2	FT	068	6B	06/04/86	2-FT-068-6B-E,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE. WR B132779
B132780	2	FT	068	6B	06/04/86	2-FT-068-6B-F,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE B132780.
B132781	2	FT	068	29A	06/04/86	2-FT-068-29A-D,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUED BOLTS. NOT NPRDS REPORTABLE WR B132781
B132782	2	FT	068	29B	06/04/86	2-FT-068-29B-E,RCS LOOP 2 COOLANT FLOW TRANSMITTER	UNKNOWN. TORQUED BOLTS. NOT NPRDS REPORTABLE. WR B132782
B132783	2	FT	068	29B	06/04/86	2-FT-068-29B-F,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE. WR B132783
B132784	2	FT	068	48A	06/03/86	2-FT-068-48A-D,=VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE. WR B132784
B132785	2	FT	068	48B	06/04/86	2-FT-068-48B-E,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUED BOLTS. NOT NPRDS REPORTABLE WR B132785
B132786	2	FT	068	48B	06/04/86	2-FT-068-48B-F,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUED BOLTS. NOT NPRDS REPORTABLE WR B132786
B132788	2	PT	068	6B	06/16/86	2-PT-068-6B-F,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE MOUNTING BOLTS WR B132788 NOT NPRDS REPORTABLE
B132789	2	FT	068	71A	06/04/86	2-FT-068-71A-D,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUED BOLTS. NOT NPRDS REPORTABLE. WR B132789
B132790	2	FT	068	71B	06/04/86	2-FT-068-71B-E,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE. WR B132790

WR	COMP ID	FUNC SYS	ADDRESS	DATE	DESCRIPTION	CORRECTIVE ACTION
B132791	2 FT	068	710	06/04/86	2-FT-068-710-F,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE. WR B132791
B132792	2 PT	068	301	06/03/86	2-PT-068-301-,RC PRESSURIZER RELIEF	UNKNOWN. TORQUE BOLTS. WR B132792. NOT NPRDS REPORTABLE
B132793	2 PT	068	311C	06/03/86	2-PT-068-311C-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS NOT NPRDS REPORTABLE. WR B132793
B132794	2 LT	068	325C	06/06/86	2-LT-068-325C-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT NPRDS REPORTABLE WR B132794
B132795	2 LT	068	326C	06/06/86	2-LT-068-326C-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT NPRDS REPORTABLS WR B132795
B132796	2 PT	068	336C	06/02/86	2-PT-068-336C-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUED MOUNTING BOLTS. NOT NPRDS REPORTABLE
B132797	2 PT	068	337C	06/03/86	2-PT-068-337C-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUE BOLTS. NOT NPRDS REPORTABLE WR B132797
B132798	2 PT	068	342C	06/16/86	2-PT-068-342C-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN. TORQUED MOUNTING BOLTS. NOT NPRDS REPORTABLE WR B132798
B132951	1 PT	001	5	06/06/86	1-PT-001-5-,VERIFY MOUNTING PER SMI-0-317-26	UNKNOWN TORQUED BOLTS NOT NPRDS REPORTABLE WR B132951
B132952	1 PS	003	121B	06/17/86	1-PS-003-121B-,VERIFY MOUNTING PER TVA DWGS 47W600-65 47W600-293 DET G 293	UNKNOWN VERIFIED MOUNTING WR B132952
B132953	1 PS	003	121D	06/17/86	1-PS-003-121D-,VERIFY MOUNTING PER TVA DWGS 47W600-65 47W600-293 DET G293	UNKNOWN VERIFIED INSTALLATION WR B132953
B132954	1 PS	003	148	06/17/86	1-PS-003-148-B,VERIFY MOUNTING PER TVA DWGS 47W600-228 47W600 86 DET M86	UNKNOWN. VERIFIED MOUNTING NOT NPRDS REPORTABLE WR B132954
B132955	2 PS	003	148	06/17/86	2-PS-003-148-B,VERIFY MOUNTING PER TVA DWGS 47W600-228 47W600-86 DET M86	UNKNOWN VERIFY MOUNTING WR B132955 NOT NPRDS REPORTABLE
B132956	1 PS	003	156	06/17/86	1-PS-003-156-A,VERIFY MOUNTING PER TVA DWGS 47W600 228 47W600 86 DET M86	UNKNOWN VERIFIED MOUNTING NOT NPRDS REPORTABLE WR B132956
B132957	2 PS	003	156	06/17/86	2-PS-003-156-A,VERIFY MOUNTING PER TVA DWGS 47W600-228 47W600-86 DET M86	UNKNOWN VERIFY MOUNTING WR B132957 NOT NPRDS REPORTABLE
B132958	1 PS	003	164	06/17/86	1-PS-003-164-A,VERIFY MOUNTING PER TVA DWGS 47W600-228 47W600-86 DET M86	UNKNOWN VERIFIED MOUNTING NOT NPRD REPORTABLE WR B132958
B132959	2 PS	003	164	06/17/86	2-PS-003-164-A,VERIFY MOUNTING PER TVA DWGS 47W600-228 47W600 86 DET M86	UNKNOWN VERIFY MOUNTING WR B132959 NOT NPRDS REPORTABLE
B132960	1 PS	003	171	06/17/86	1-PS-003-171-B,VERIFY MOUNTING PER TVA DWGS 47W600-228 47W600-86 DET M86	UNKNOWN VERIFIED MOUNTING NOT NPRDS REPORTABLE WR B132960
B132961	0 PS	003	171	06/17/86	0-PS-003-171-B,VERIFY MOUNTING PER TVA DWGS 47W600-228 47W600 86 DET M86	UNKNOWN VERIFY MOUNTING WR B132961 NOT NPRDS REPORTABLE

103 records listed

MAINTENANCE SUMMARY
(MECHANICAL)

MECHANICAL MAINTENANCE MONTHLY REPORT FOR JUNE 1986

COMMON

1. Continued work on ERCW pump "RA" to replace the upper pump shaft which had become too rough to allow the shaft sleeve O-Ring to seal. A special maintenance instruction was prepared for this work which is currently on hold pending receipt of a new shaft sleeve.

2. The shaft sleeve and packing were replaced on "PB" ERCW pump.

3. Finalized our maintenance instruction enhancement plan utilizing assistance from four Westinghouse engineers under a personal services contract.

4. The Plant Operating Review Committee approved a maintenance instruction revision adding the maintenance instruction writers guide as well as issuing seventeen new maintenance instructions.

5. Provided fire door training to responsible carpenters, foreman, planners, engineers, engineering aides, and engineering section supervisor on the new fire door, channel calibration of safety inspection system accumulation water level revision, and Physical Security Instruction 13 ("Fire") requirements.

6. Completed rebuilding the "B" turbine building sump pump.

7. Performed a preventive teardown, repair, and reassembly of miscellaneous Crane Chem-pumps to prolong their life.

8. Began investigation for a generic FCR on stress corrosion cracking of B6 bolting in POSI-SEAL Butterfly Valves due to a problem found on an ERCW pump discharge valve.

9. Completed the replacement of the seats on CO_2 pilot valves at the diesel generator building.

10. Replaced a head gasket on "BB" auxiliary control air compressor.

11. Completed the annual inspection of 1BB diesel generator.

12. Provided carpenter support for various walkdowns including unit one snubber nameplate data collection.

13. Replaced a failed rupture disc on "A" boric acid evaporator package.

14. Removed sixteen red rubber gaskets from miscellaneous locations inside containment for degradation evaluation.

15. Seals in the auxiliary building crane gear boxes were replaced.

16. Participated in this radiological emergency plan drill.

17. Completed Surveillance Instruction (SI-149) on damper inspections.

MECHANICAL MAINTENANCE MONTHLY REPORT FOR JUNE 1986

UNIT 1

1. Completed replacement of the boric acid system valves and piping which exhibited signs of stress corrosion cracking.

2. Replaced wire rope on the polar crane small hook.

MECHANICAL MAINTENANCE MONTHLY REPORT FOR JUNE 1986

UNIT 2

1. Began inspecting and repairing the long cycle valves (2-FCV-3-191, 192, 193, 194 and 2-VLV-3-568) due to leak through problems.

2. Reassembled the 2A component cooling water pump after replacing the shaft sleeves.

3. Continued repairing the failed containment isolation valves.

4. Completed workplan 11948 stelliting the after terry-turbine steam trap internals due to erosion problems.

5. Reinforced the damaged insulation on the reactor head.

6. Completed post sludge-lancing inspections and closed up the steam generators.

MR.COMP	U	FUNC SYS	ADDRESS.	COMP DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A546226	1	FCV 067	0067	06/02/86	1-FCV-067-0067-B,[*NPRD*],VLV FAILED TO OPEN WHEN THE D/G STARTED DURING PERFORM OF SI 9.2ND TIME THIS HAS HAPPENED IN THE PAST MONTH.VLV WON'T OPEN OR CLOSE FROM CONTROL RM	MATERIAL RESTRAINT TO DAN CHAPMAN 2/4/86
B114836	0	VLV 082	0528-1B2	06/13/86	0-VLV-082-0528-1B2-B,[*NPRD*],VLV DID NOT PASS SI 102 M/A STEP 6.4	WORN SAFETY VLV. INSTALLED NEW RELIEF VLV PERFORMED SI-102 AND RETURNED TO SERVICE
B114837	0	VLV 082	0534-1B2	06/13/86	0-VLV-082-0534-1B2-B,[*NPRD*],VLV DID NOT PASS SI 102/M/A STEP 6.4	WORN SAFETY VLV. INSTALLED NEW SAFETY VLV CHECKED PSI SETTING AND RETURNED TO SERVICE
B117381	2	VLV 032	0387	06/24/86	2-VLV-032-0387-,[*NPRD*],2 VLV 32 387 FAILED SI 158.1 LEAK RATE TEST.	SEATS SCUFFED AND PLUG INSERT WORN. DISASSEMBLED VLV CLEANED INTERNALS AND LAPPED SEAT. INSTALLED NEW PLUG INSERT RAN SI 158.1 AND RETURNED TO SERVICE
B119388	0	VLV 082	0506-1B1	06/12/86	0-VLV-082-0506-1B1-B,[*NPRD*],1B1 AIR TNK AND 1B1 SPARE TNK PRESS IND READ THE SAME PRESS. AS THE ALIGNED TNK DROPS IN PRESS THE SPARE TNK PRESS DROPS WITH IT. THE CHECK VLV SEPARATING THE TWO TNKS IS PROBABLY NOT CLOSING OR LEAKING BADLY	DIRTY INTRNALS. DIS ASSEMBLED VLV AND CLEANED INTERNALS RAN SI-7 AND RETURNED TO SERVICE
B125514	2	VLV 003	0568	06/24/86	2-VLV-003-0568-,[*NPRD*],VLV WON'T SEAT OFF	SEAT SCARED. DISASSEMBLED VLV, LAPPED SEAT PERFORMED FUNCTIONAL CHECK AND RETURNED TO SERVICE
B130960	0	VLV 082	0508-1B1	06/13/86	0-VLV-082-0508-1B1-B,[*NPRD*],VLV DID NOT PASS SI 102 M/A STEP 6.4	WORN VLV. INSTALLED NEW VLV PERFORMED FUNCTIONAL TEST AND RETURNED TO SERVICE
B132013	0	CMP 032	0086	06/20/86	0-CMP-032-0086-,AIR IS LEAKING ARDUND SHAFT.	WORN INTERNALS. REBUILT COMPRESSOR

8 records listed.

MAINTENANCE SUMMARY
(MODIFICATIONS)

SUMMARY OF WORK COMPLETED

MODIFICATIONS

JUNE 1986

NUREG 0588

ECN 6231 - Remove Interferences

The removal of one support and the addition of three supports were completed; however, additional work is anticipated (see ECN 6681).

ECN 6552 - 0588 Solenoids

Functional testing was performed on one valve because of other work in progress on this valve by Maintenance.

ECN 6616 - Damper Fusible Link

Workplan preparation was completed, and it was placed in the approval cycle.

APPENDIX R

ECNs 5435, 6343, and 6344 - Fire Doors

The replacement and repair of the doors and doorframes (ECNs 6343 and 6344) were completed. The installation of weatherstripping continues as resources permit.

ECN 6235 - Reroute Various Cables

This work is complete.

ECN 6311 - Operator Extension on PORV

This work has been completed except for final cycling at startup.

ECN 6315 - Replace Fuses

This work is complete.

ECN 6319 - Fire Protection Piping

This work is complete.

ECN 6642 - Annulus Fire Protection and Sealing Containment Penetrations

This work is complete.

OTHER ITEMS

ECN 2783 - Installation of Fifth Diesel

Six electrical-related workplans for the fifth diesel installation have been placed in the workplan closing cycle.

ECN 5009 - ERCW Piping Changeout From Carbon Steel to Stainless Steel

The workplan for the replacement of isolation valves remains in the approval cycle. Some insulation activities remain incomplete. DNE is revising the USQD. Based upon DNE's analysis, some additional hanger work may be required for the piping replaced on the upper compartment cooler lines inside containment.

ECNs 5034, 5713, 5743, and 6064 - Various Platforms in Lower Containment

The first trolley was modified by the vendor and found acceptable. The remaining trollies are being prepared for shipment to the vendor for modification. Work will continue when the trollies are returned.

ECN 5202 - Interface of Fifth Diesel With Other Diesels

This work is no longer scheduled for this time period.

ECN 5620 - Add Instrumentation for Auxiliary Feedwater Pump

Work is on hold and is not scheduled for this time period.

ECN 5657 - Installation of MSR Drain Valves

Reinsulation remains incomplete. Caps have been installed on the valve nipples.

ECNs 5703 - Reinforcement of Block Walls

During the installation of the restraints, some mortar joints were broken. Efforts are underway to repair these joints before the restraints are installed.

ECN 5724 - CVCS/RHR Supports

Three hangers and two notifications of indication remain incomplete.

ECN 5914 - Improve Reliability of Steam Dump

This work is complete.

ECNs 5938, 6305, and 6571 - Replace Feedwater Heater and Eroded Pipe

Mechanical Modifications is supporting postmodification testing (PMT) on both units as requested. Insulation work on unit 1 continues. Installation of the monorail on unit 2 continues.

ECN 6057 - Cable Tray Covers

Approximately 240 out of 290 cable tray covers have been remanufactured or replaced.

ECN 6147 - Airlock Packing Nut

Final testing of the airlocks on unit 2 remains incomplete.

ECN 6185 - Offsite Paging System

Because additional versatility is needed, new EPROMs and crystals were ordered. Further work will be required.

ECN 6196 - Pressurizer Hangers and Valves

PMT is incomplete and scheduled for startup.

ECN 6204 - Electrical Penetration Overcurrent Protection

Fuse replacement and fuse block installation are complete. Electrical Modifications is awaiting a procedure change to place the circuits in service.

ECN 6259 - Moisture Separator Reheater Tube Bundle Replacement

Insulation activities are continuing. The decision was made to replace the unit 2 bundles, and work was begun. The moisture separator reheaters were stripped, and three high-pressure bundles were removed. Work is continuing.

ECN 6380 - Replacement of Barton Pressure Transmitters

Preliminary workplan preparation continues.

ECNs 6402 and 6439 - Pressurizer Instrumentation Relocation

All work is complete except seal welding fill tees. This will be completed after Instrument Maintenance completes fill and calibration. A unit 2 instrument was relocated to meet Appendix R separation criteria.

ECN 6417 - Install Alternate Seal Water for Pumps, CDWE

Electrical drawings remain to be issued.

ECNs 6491 and 6534 - ERCW Supports

Drawings were received, and the workplan was written and approved. All hangers for unit 2 (ECN 6534) have been completed. Three hangers remain for unit 1.

ECN 6513 - Support of HVAC Expansion Tank

The workplan was written, approved, and placed in work. Work is continuing.

ECN 6573 - Shutdown Board Alternate Feeder

The workplan is in the approval cycle.

ECN 6599 - Unit 2 Shield Building Anchor Problem

DNE continues to evaluate the action required for small conduit anchors.

ECN 6601 - Removal of EGTS Backdraft Dampers

PMT remains to be completed.

ECN 6606 - Additional Penetration Fuses

The workplan is in the approval cycle.

ECN 6610 - Modify Air Return Fan Supports

Work continues on the unit 2 fans.

ECNs 6631 and 6683 - Scrubber Replacements/Modifications

Preliminary workplan preparation was begun.

ECN 6636 - Replace Cable in Valve Room

The workplan is in writing.

ECN 6648 - Time-Delay CCS Pump

The workplan is in writing.

ECN 6661 - Upgrade of ERCW Piping

The workplan remains in the approval cycle.

ECN 6667 - Delete Torque Switch MOVs

The workplan is in the approval cycle.

ECN 6681 - Supports for ERCW to Upper Compartment Coolers (Annulus)

Preliminary work was started on support design and workplan verification. This is continuing (see ECN 6231).

ECN 6698 - Repull 120-Volt Cables

The workplan is in writing.

ECN 6701 - Modify Foxboro Recorder Latches

Preliminary workplan preparation was started.

Dry Active Waste Building (DCR 1898)

Steel erection of columns and ceiling beams for the building frame has been completed, along with longitudinal purlins. Final alignment has been made of the building frame, and corner fabrications are being installed in preparation for side insulation and sheeting. Major equipment that would create a handling problem once the roof is installed is being set.

Additional Office Buildings (DCR 2260)

Building A office area is complete with the exception of the reconfiguration of the cubicles in accordance with a design revision, installation of power poles to cubicles including electrical and communications, and minor paint touchup.

Electrical room power panels have been received and are in the process of being installed. Potable waterlines and sewer lines are complete. Restroom facilities and lunchroom area are complete pending light and carpet installation.

Building B final drywall plastering is in progress with an expected completion date of July 7, 1986, at which time painting and carpet and cubicle installation will begin.

Additional Office Buildings at Power Operations Training Center

The east building is complete and ready for carpet installation, which is scheduled for July 7, 1986. Although no cubicles are to be installed, power poles will be installed at various locations for electrical and communications. The location design will be issued July 7, 1986.

Final drywall plastering for the west building is in progress along with exterior fire door installation.

Burnout of the potable water system is scheduled for July 11, 1986. Septic tank and drain installation is nearing completion and is due for inspection on July 10, 1986.

Welding Project

Support was provided to Welding Project personnel as requested.

Walkdown Field Verification Support

SMI-0-317-27 - Concrete Edge Distance - Complete; DNE continues to evaluate the results.

SMI-0-317-24 - 79-14 Enhancement - 145 unit 2 supports have been inspected and are being evaluated. Approximately 1,100 unit 1 supports have been inspected and are being evaluated. Inspections continue. Approximately 60 additional supports will be added to unit 2. Fourteen unit 2 discrepancies have been identified as requiring fieldwork.

Alternate Analysis Walkdown

Work is continuing.

Unit 2 J-Tube Replacement

Work was completed as scheduled.

SCR Resolution

Many man-hours were spent resolving SCRs.

MODIFICATIONS

ECN	DCR	SYS	DESCRIPTION	UNIT
	19256	031	Replace exiting mallory capacitors in the FCO-31-475&476 with Maxwell Capacitors ITT #104857-AJ	1
L5265	D0741 S	031	Replace doors C-49 & C-50 per PT-522 Ref ECNs 5264 & 5347.	0
L5576	D1377 S	069	Add a double isolation valve on each vent line at the floor level on ele 734 (refueling floor). Connect the vent lines into the plant drain system pressurizer condensate reset.	2
L5591	D0837	317	Redesign control circuits to monitor neutral contracts on all W-2 spring return to center type switches or change to GE SBM switches.	2
L5667	D0674 S	063	Replace existing 1/2" S/S 6000 # socket weld cap with a 1/2" 6000 # threaded lap - SIS hot & cold leg and BIT injection flow orifices.	2
L5794	D1804 S	067	Replace retainer ring of Asim A-516 GR.70 with a retainer ring of 304 stainless steel for 6" 150 lb. Do SI-Seal Butterfly vlv dwg. #17W586-24 MK# SK-6874. Ref TACF 82-2168-6.	1
L5957	D1380	099	Replace the present SI reset timing relay, mfg. agastat - model 2412AE with an on delay/ off delay relay. TDIs (SSPS time delay).	1
L6209	F2588	363	Install a fire protection blanket around conduits and add pyrocrete to floor hatch for fire protection.	2

MODIFICATIONS

ECN	DCR	SYS	DESCRIPTION	UNIT
L6529	D0972	000	Seal class 1E devices at their conduit entry with a nuclear qualified seal and reduce the size of conduit opening on other devices with RTV sealant.	
L6549	D0972 S	000	Analyze junction boxes housing class 1E component located inside containment below postulated LOCA flood level and relocate if req.	1
L5169		077	Documentation to revise vlv marker tags to add tag numbers for PDIS 77-403,404, 405. QA chg #350 added unit 2 to WP 8432. Ref AC-449.	2
L6223	F2725 S	003	Repair leaking valves 1-3-524, -526 by injecting Fermonite into the bodies of 1-3-525, 527 filling valves and connecting pipe.	1
L6230	F2734 S	003	Cut and cap the six (6) drain lines involving the removal of valves 3-504,505, 506, 507, 520, 521, 522,523, 524, 525, 526 and 527. (FCR-2734 Cat D)	1
L6690	D2259 S	000	Bolt torque requirement for non high strength bolts in friction type connections for supports. No predecessor ECNs.	2
Not Req	D1967 S	001	Remove the tube lane blocking devices from the steam generators. The low-down pipe blocker assembly shall not be removed.	2

ENVIRONMENTAL QUALIFICATION
(E.Q.) SUMMARY

Date: 7/1/86

SCR No.	Description	ECN	Engineer	Workplan No.	Estimated Date of Completion U-1	Estimated Date of Completion U-2	Comments
EQP 8501	Disconnect 1- and 2-HS-62-61	6524	Peters	11901	C	C	U-2 QIR submitted.
EQP 8502	Replace penetrations 23 and 48	6490	Peters	11801, 11802, 11810, 11811	C	C	Complete QIR submitted.
EQP 8503	Relocate RE-90, 273, -274	6500	Peters	11810, 11811	C	N/A	Complete field verification sheets are in binder. No QIR needed.
EQP 8504	Splice methods not correct	N/A	Stockton	80 MRs	C	C	Complete QIR submitted.
EQP 8505	D[illegible]g	N/A	N/A	N/A	N/A	N/A	
EQP 8506	Seal containment isolation valve	6514	Kimsey	11880	C	C	
EQP 8507	Rewire MOV	N/A	Rutledge	11866, 11853	C	C	
EQP 8508	JB weepholes (press)	6523	Alas	11893	C	C	Complete QIR submitted.
EQP 8508R2	JB weepholes	6709	Alas				
EQP 8509R1	Conduit seals	6529	Kimsey	11903, 11904	C	C	
EQP 8509R2	Conduit seals	6615	N/A		N/A	N/A	No field work remaining.
EQP 8510	Disconnect local handswitches	6527	Peters	11901	C	C	
EQP 8511	Submerged JB inside containment	6549	Peters	11901	C	N/A	

Date: 7/1/86

SCR No.	Description	ECN	Engineer	Workplan No.	Estimated Date of Completion U-1	U-2	Comments
EQP 8512R2	Rewire JB	N/A	Amburn	11855, 11856	C	C	All boxes rewired.
EQP 8513	Weep holes (moisture)	6547	Alas	11898	C	C	Complete QIR submitted.
EQP 8513R2	Weep holes (moisture)	6565	Alas	11937	C	C	Complete QIR submitted.
EQP 8514	Motor insulation 74-1, -2	6540	Branham	11906	N/A	C	Complete QIR submitted.
EQP 8515	Replace 2-PDI-30-43	6554	Legg	11912	C	C	
EQP 8516	Replace 2-LT-3-174	N/A	Instrument Maint.	N/A	N/A	C	
EQP 8517	ABGTS humidity control	6578	Gonzalez		N/A	C	
EQP 8518	Submerged cables	6533	Various/6	Various	C	C	
EQP 8519	Tee drains	N/A	Electrical Maint.	N/A	C	C	Complete QIR submitted.
EQP 8520	Expired cables	6553	Gonzalez	11902	C	C	
EQP 8521	Delete TB and rework splices	6550	Stockton	11914, 11915	C	C	
EQP 8521R1	Delete TB and rework splices	6651					
EQP 8522	Rewire local panels	N/A	Stockton	11914, 11915	C	C	
EQP 8522R2	Rewire local panels	N/A	Instrument Maint.	12016	C	7/2	

Date: 7/1/86

SCR No.	Description	ECN	Engineer	Workplan No.	Estimated Date of Completion U-1	Estimated Date of Completion U-2	Comments
EQP 8523	Missing bolts and washers and misplaced brackets	N/A	Stockton	11914, 11915	C	C	
EQP 8524	Change setpoints	6551	Instrument Maint.	11916	C	C	
EQP 8525	Reterminate hydrogen recombiner	N/A (MR)	Electrical Maint.	N/A	C	C	Completed by Electrical Maint.
EQP 8526	Replace FSVs, U-1 1, U-2 11	6552	Mechanical Mods.	11897	C	C	
EQP 8527	Coat TB, U-1 3, U-2 8	N/A (MR)	Stockton	11914, 11915	C	C	
EQP 8528	Solder strain gauge Barton transmitters	IMI	Instrument Maint.	N/A	C	C	QIR submitted 1/24.
EQP 8529	PDT-30-42, -43	6554	Legg	11912	C	C	
EQP 8530	Gasket, Namco L/S	N/A (MR)	Electrical Maint.	N/A	C	C	
EQP 8531	Delete MOV heaters	6544	Rutledge	11866, 11853	C	C	
EQP 8532	Delete L/S 1-, 2-43-201, -202, -207, and -208	6630	Alas		N/A	N/A	No longer an issue.
EQP 8533	Delete dual voltage splice	N/A	Rutledge	11866, 11853	C	C	

Date: 7/1/86

SCR No.	Description	ECN	Engineer	Workplan No.	Estimated Date of Completion U-1	Estimated Date of Completion U-2	*Comments
EQP 8534	Resplice valve positioner 3-174 and -175	N/A	Maxwell	MR	C	C	Complete QIR submitted.
EQP 8535	Replace limit switches, U-1 10, U-2 12	6556	Stockton	11927, U-2 11928, U-1	C	C	
EQP 8536	Valve room submergence	6561	Mechanical	11939	C	C	
	Cap drains	6632	Mods.		C	C	
EQP 8536R1	Valve room submergence	6612	Electrical Mods.		C	C	
EQP 8537	Rebuild or replace JB 3078	6579	Amburn		N/A	C	
EQP 8538	Replace capacitors FCO-31-475, -476	N/A	Maxwell	11977	N/A	C	
EQP 8539	Replace capacitors	N/A	Instrument Maint.		C	C	
EQP 8540	Replace pigtails to Target Rock solenoid valves	6649	Peters		7/15	7/15	Workplan in approval cycle.
EQP 8541	Delete brakes	6582	Branham	11980	C	N/A	
EQP 8542	Replace unqualified cables	6680	Hall		7/30	N/A	Two cables will be replaced in accordance with a new issue.
EQP 8543	Replace JB wire	N/A	Amburn	11855, 11856	C	C	

Date: 7/1/86

SCR No.	Description	ECN	Engineer	Workplan No.	Estimated Date of Completion U-1	U-2	Comments
EQP 8601	Replace 1-, 2-PT-1-2A, -27A; upgrade 2-PT-1-2B, -27B	6588	Elkins	12014	C	7/2	
EQP 8602	Reroute control cables for 1-, 2-FCV-70-87, -89	**********					No longer an issue.
EQP 8603	Replace portion of PP711B	6627	Kimsey		N/A	C	
EQP 8604	Cable 1PL3241A not qualified	**********					No longer an issue.
EQP 8605	1-, 2-TS-74-43, -44, -45, -46 not qualified	6589	Branham		C	C	
EQP 8606	Undervoltage concern on feedwater isolation valve brakes	6611 6676	Rutledge		8/15	8/5	Holding for drawings
EQP 8607	Delete TB for 1-, 2-PT-1-2A, -27A	6626	Instrument Maint.	12014	C	7/2	Will work same time as EQP 8601.
EQP 8608	Enable/disable MOV brakes	6621 6622 6665	Rutledge		6/30	6/30	Holding for ECN 6686.
		6686			8/13	8/13	
EQP 8609	Replace cable 1V7973B	6618				N/A	EQP is reviewing ECN for technical correctness.
EQP 8611	JB has weephole in top	N/A	Amburn		N/A	C	

Date: 7/1/86

SCR No.	Description	ECN	Engineer	Workplan No.	Estimated Date of Completion U-1	Estimated Date of Completion U-2	*Comments
N/A	Move surge suppression network for PORV	5773	Kimsey	11883	C	C	U-2 complete. On U-1, 1-PCV-68-340 in hold. Holding for maintenance to complete.
EEB 8523	Penetration overcurrent	6606	Legg		7/20	7/20	During closure process, it was found that 6 circuits had not been addressed by ECNs. ECN 6606 addresses this.
N/A	Work FCR to delete 1-, 2-PS-3-160A, -160B, -165A, and -165B	5883	Hall		C	C	Need SI-166.
N/A	Replace 1-FT-1-3A, -3B, -10A, -10B, -21A, -21B, -28A, -28B	6347	Instrument Maint.	N/A	C	N/A	Transmitters changed need response test.
NEB 8510	Relocate LT-68-320, PT-68-323, -320	6439	Carrasquillo Peters	Various	C	C	
	Remount 63-71, 68-308	6496	Legg	11865	C	C	
	Replace LS-65-4, -5	6504	Legg	11865	N/A	C	
MEB 8410R3	Replace LS-77-127	6525	Legg	11865	N/A	C	
	Delete brakes FCV-62-61	6521	Branham	11905	C	C	
EEB 8517	Replace pressure transmitter PDT-65-80, -82, -90, and -97	6488	Branham	11931	C	C	
EEB 8631	Raychem splice						

Date: 7/1/86

SCR No.	Description	ECN	Engineer	Workplan No.	Estimated Date of Completion U-1	Estimated Date of Completion U-2	*Comments
EQP 8612	Replace zone switches	6669			N/A	C	
EQP 8614	Weepholes in JB	6652			C	C	
EQP 8615	Replace FT-72-13, -34						
EQP 8617	Junction box internal wiring						
EQP 8618	Relocate arc suppression network						
EQP 8619	Valve room submerged cables						
EQP 8620	Relocate steam generator level transmitters						

OFFSITE DOSE CALCULATION MANUAL CHANGES

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TABLE OF CONTENTS

TABLE OF CONTENTS (continued)

13

Description of Change

A typographical error in Table 3.2-1 "Maximum Values for the Lower Limits of Detection (LLD)" is to be corrected. The gross beta LLD value (1×10^{2}) for airborne particulate or gas should be 1×10^{-2}. No model changes are reflected in this change; therefore, no evaluations are necessary.

TABLE 3.2-1

MAXIMUM VALUES FOR THE LOWER LIMITS OF DETECTION (LLD)[a,c]

Analysis (pCi/l)	Water (pCi/l)	Airborne Particulate or Gas (pCi/m^3)	Fish (pCi/kg,wet)	Milk (pCi/l)	Food Products (pCi/kg,wet)	Sediment (pCi/kg,dry)
gross beta	4	1×10^{-2}	N.A.	N.A.	N.A.	N.A.
H-3	2000	N.A.	N.A.	N.A.	N.A.	N.A.
Mn-54	15	N.A.	130	N.A.	N.A.	N.A.
Fe-59	30	N.A.	260	N.A.	N.A.	N.A.
Co-58,60	15	N.A.	130	N.A.	N.A.	N.A.
Zn-65	30	N.A.	260	N.A.	N.A.	N.A.
Zr-95	30	N.A.	N.A.	N.A.	N.A.	N.A.
Nb-95	15	N.A.	N.A.	N.A.	N.A.	N.A.
I-131	1[b]	7×10^{-2}	N.A.	1	60	N.A.
Cs-134	15	5×10^{-2}	130	15	60	150
Cs-137	18	6×10^{-2}	150	18	80	180
Ba-140	60	N.A.	N.A.	60	N.A.	N.A.
La-140	15	N.A.	N.A.	15	N.A.	N.A.

Description of Change

Table 3.1-2 of the SQN ODCM is to be revised by deleting Farm SU, 3.3 miles SSE. This farm is to be deleted because milk samples are no longer available at this location. This change is also reflected in Figure 3.1-6.

Evaluation and Justification for Change

The change is necessary to accurately reflect the milk sampling program conducted at SQN.

Effect on the Accuracy and Reliability of Dose Calculations and Setpoint Determinations

This change has no effect on the accuracy or reliability of dose calculations or setpoint determinations.

SQN

TABLE 3.1-2

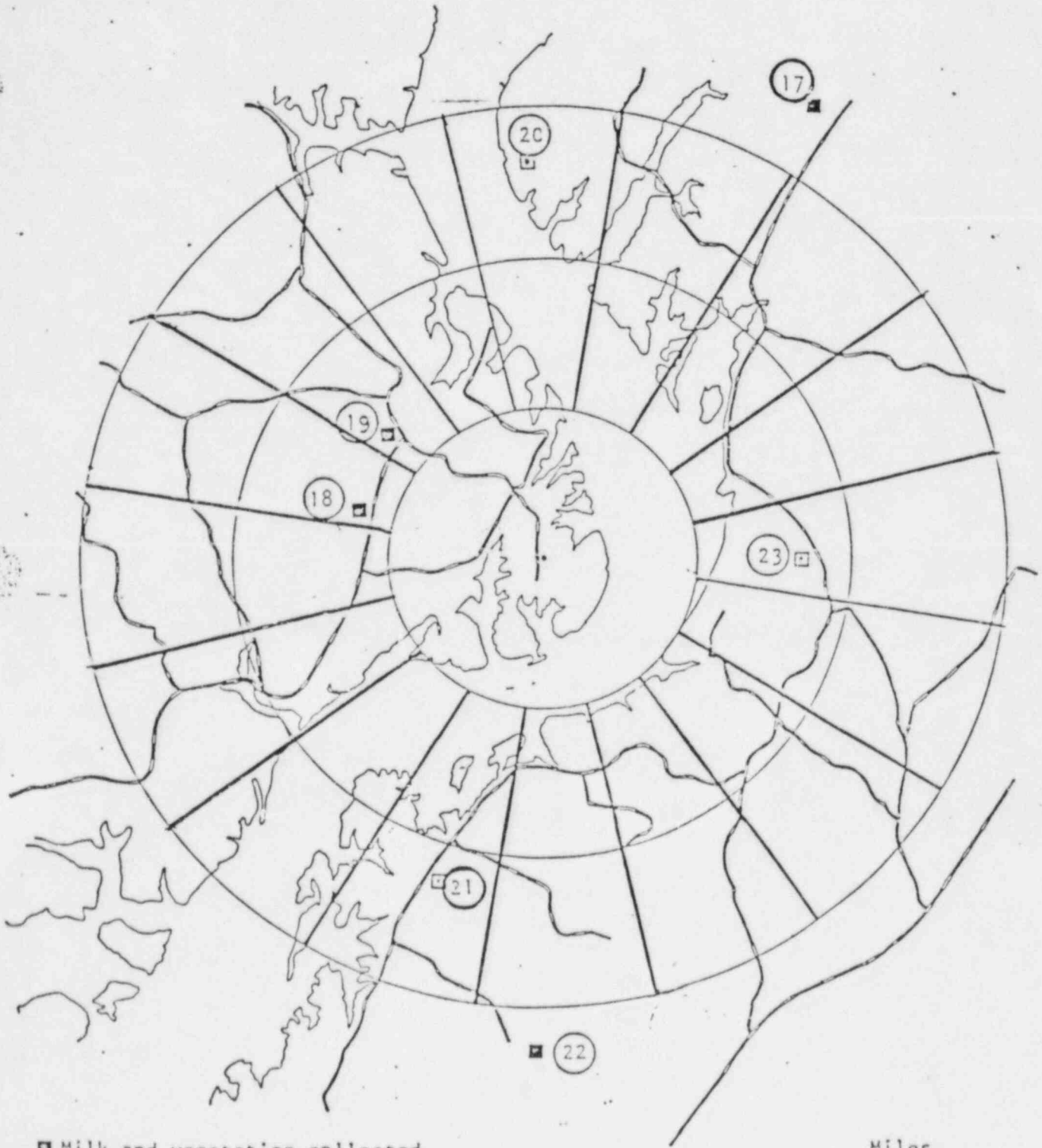
ATMOSPHERIC AND TERRESTRIAL MONITORING STATION LOCATIONS

SEQUOYAH NUCLEAR PLANT

Reference Number	Sample Station	Location Approximate Distance and Direction from Plant
2	LM-2 SQ	0.8 mile N
3	LM-3 SQ	1.3 miles SSW
4	LM-4 SQ	1.5 miles NE
5	LM-5 SQ	1.7 miles NNE
7	PM-2 SQ (Chester Frost Park)	3.8 miles SW
8	PM-3 SQ (Daisy)	5.6 miles W
9	PM-8 SQ (Harrison)	8.7 miles SSW
10	PM-9 SQ (Lakeside)	2.7 miles WSW
11	RM-1 SQ (Chattanooga, Riverside)	16.7 miles SW
12	RM-2 SQ	17.5 miles NNE
13	RM-3 (Cleveland)	11.3 miles ESE
14	RM-4 (Dunlap)	19.5 miles WNW
17	Farm M	3.5 miles NNE
18	Farm J	1.3 miles NW
19	Farm HW	1.3 miles W
20	Farm EM	2.5 miles N
21	Farm BR	2.3 miles SSW
22	Farm LE	3.5 miles S
23	Farm GO	1.7 miles E
25	Farm B (control)	43.0 miles NE
26	Farm C (control)	16.0 miles NE
27	Farm S (control)	12.0 miles NNE

Figure 3.1-6

MILK AND VEGETATION SAMPLING LOCATIONS



■ Milk and vegetation collected

⊡ Vegetation only collected

Miles

0 1 2

Note: Vegetation is also collected at each air monitoring station. See Figure 3.1-1

Revision 14

Description of Change

Figure 3.1-7 "Milk Sampling Locations" is to be deleted. The milk sampling locations are identified in Figure 3.1-6 with the vegetation sampling locations. All references to Figure 3.1-7 in the text of the ODCM have been changed to Figure 3.1-6. This deletion removes redundant information from the SQN ODCM and has no effect on the methodology or sampling program outlined in the manual.

Evaluation and Justification of Change

The change will remove redundant information concerning milk sampling locations.

Effect on the Accuracy and Reliability of Dose Calculations and Setpoint Determinations

This change has no effect on the accuracy or reliability of dose calculations or setpoint determinations.

Figure 3.1-7
Milk Sampling Locations

Figure 3.1-7 Deleted by
Revision 14

Description of Change

The word "land" is to be inserted in the phrase ". . . nearest site boundary . . ." to clarify the location descriptor as ". . . nearest land site boundary . . ." on pages 2, 5, and 13. No model changes are reflected in this change; therefore, no evaluations are necessary.

7. Raw meteorological data consist of wind speed and direction measurements at 10m and temperature measurements at 9m and 46m.

8. Dose is to be evaluated at the offsite exposure point where maximum concentrations are expected to exist.

9. Potential maximum-exposure points considered are the nearest land site boundary points (Table 1.4) in each sector. |14

10. A semi-infinite cloud model is used.

11. No credit is taken for shielding by residence.

12. Plume depletion and radioactive decay are considered.

13. Building wake effects on effluent dispersion are considered.

14. A sector-average dispersion equation is used.

15. The wind speed classes that are used are as follows:

Number	Range (m/s)	Midpoint (m/s)
1	<0.3	0.13
2	0.3-0.6	0.45
3	0.7-1.5	1.10
4	1.6-2.4	1.99
5	2.5-3.3	2.88
6	3.4-5.5	4.45
7	5.6-8.2	6.91
8	8.3-10.9	9.59
9	>10.9	10.95

| 6

16. The stability classes that will be used are the standard A through G classifications. The stability classes 1-7 will correspond to A=1, B=2, ..., G=7.

17. Terrain effects are not considered.

18. Environmental transfer data is consistent with NUREG/CR-1004.

Equations

To calculate the dose for any one of the 16 potential maximum-exposure points, the following equations are used.

For determining the air concentration of any radionuclide:

10. Potential maximum exposure points considered are the nearest land site boundary points (Table 1.4) in each sector. |14

11. Terrain effects are not considered.

12. Building wake effects on effluent dispersion are considered.

13. Plume depletion and radioactive decay are considered for air-concentration calculations.

14. Radioactive decay ~~is~~ considered for ground-concentration calculations.

15. Deposition is calculated based on the curves given in Figure 1.2.

16. A milk cow obtains 100 percent of her food from pasture grass.

17. No credit is taken for shielding by residence.

Equations

To calculate the dose for any one of the potential maximum-exposure points, the following equations are used:

1. Inhalation

Equation for calculating air concentration, X, is the same as in the Noble Gas Section, 1.1.1.A.

For determining the thyroid dose rate:

$$D_{THI} = 1 \times 10^{-6} \sum_i X_i \, DFI_i \qquad (1.4)$$ |3

where:

D_{THI} = thyroid dose rate due to inhalation, mrem/y.

X_i = air concentration of radionuclide i, $\mu Ci/m^3$.

DFI_i = infant inhalation dose factor, mrem/yr per $\mu Ci/cm^3$, (Table 1.7). |1

1×10^{-6} = m^3/cm^3 conversion factor.

Step 2

This methodology is to be used if the calculations in Step 1 yield doses that exceed 50 percent of the applicable limits. | 13

Equations and assumptions for calculating doses to air from releases of noble gases are as follows:

Assumptions

1. Doses to be calculated are gamma and beta air doses.

2. Dose is to be evaluated at the nearest land site boundary point in each sector. | 14

3. Historical onsite meteorological data from the period 1972-1975 will be used. | 9

4. All measured radionuclide releases are considered.

5. A semi-infinite cloud model is used.

6. Radioactive decay is considered.

7. Building wake effects on effluent dispersion are considered.

8. Dose factors are calculated using data from TVA's radionuclide library.

Equations

Equations for calculating air concentration, X, is the same as in Section 1.1.1, step 1, part A. Air concentrations are calculated for the site boundary in each sector.

For determining the gamma dose to air

$$D_{\gamma n} = t_m \sum_i X_{ni} DF_{\gamma i} \qquad (1.16)$$

where:

$D_{\gamma n}$ = gamma dose to air for sector n, mrad.

X_{ni} = air concentration of radionuclide i in sector n, $\mu Ci/m^3$

$DF_{\gamma i}$ = gamma-to-air dose factor for radionuclide i, mrad/yr per $\mu Ci/m^3$ (Table 1.5).

t_m = time period considered, yr

Description of Change

ODCM page 10 under Step 2 has been revised to include methodology for Release Mix specific setpoint determination.

Evlauation and Justification for Change

This addition is needed in the ODCM to illustrate how setpoints (gaseous) are determined for releases of a known nuclide mixture. Methodologies are given for determining setpoints from the RETS dose rate limits or from the 10 CFR 50.73 concentration limits for reporting.

Effect of Change on Release Rate Limits or Calculated Doses

These changes will not affect calculated doses. Changes in gaseous release rate limits calculations have been documented in the memorandum from E. A. Belvin to H. L. Abercrombie, dated October 30, 1985 (L61 851030 800).

Step 2a (Initial Setpoints)

The dose rate limits of interest (Specificatien 3.11.2.1) are:

Total Body = 500 mrem/yr
Skin = 3000 mrem/yr
Maximum Organ = 1500 mrem/yr

For the determination of initial setpoints, the above limits are divided by the appropriate dose calculated in step 1 yielding,

$$\frac{\text{Dose limit}}{\text{Dose step 1}} = R$$

This ratio, R, represents how far above or below the guidelines this step 1 calculation was. Multiplying the original source term by R will give release rates that should correspond to the dose limits given above.

Appropriate release rate limits in μCi/ sec for each nuclide and release point will be provided to plant personnel for use in establishing monitor setpoints. The setpoint for each gaseous effluent monitor will be established using plant instructions.

The general equation used by plant personnel in establishing setpoints in cpm from release rate limits in μCi/sec is as follows:

$$C = \frac{Q \times e \times s \times 60}{v \times 28320}$$

where:

C = monitor setpoint, cpm
Q = release rate limit, uCi/sec
e = detector efficiency, cpm/μCi/cc
s = safety factor; 0.2 for systems without automatic isolation; 0.5 for systems with automatic isolation
60 = sec/min
28320 = cc/ft^3
V = flow rate past the detector, cfm

Step 2b (Release Mix Specific Setpoints)

When release mixes are known, setpoints are based on the dose methodology given in Step 1, disregrding the design source term mix. Using a normalized source term (q_i) for each nuclide, nuclide specific dose rates (D_i) are determined independently for each nuclide using Step 1 methodology. Dividing the appropriate dose rate limit above by the nuclide specific dose rate, D_i, yields.

$$\frac{\text{Dose rate limit}}{D_i} = r_i$$

This ratio, r_i represents how far above or below the guidelines the nuclide specific dose rate (D_i) is. Multiplying the normalized source term (q_i) by r_i will give the maximum allowable release rate R_i for nuclide i.

R_i for each nuclide is also calculated by an alternative methodology using the reporting requirements of 10 CFR 50.73 (2 times the 10 CFR 20 Appendix B, Table II air concentrations). Release rate limits, R_i for each nuclide are calculated using this methodology as given below:

$$R_i = (2)(MPC_i)(5.12 \times 10^{-6})(10^{-6})$$
$$= 1.02 \times 10^{-11} MPC_i$$

where

MPC_i = the 10 CFR 20, Appendix B. Table II, air concentration, $\mu Ci/cc$.
5.12×10^{-6} = worst land site boundary X/Q, s/m^3 (Table 1.4).
10^{-6} = conversion factor, m^3/cc.

The release rate limit, R_i, for each nuclide will be the most restrictive one calculated using both methodologies. 14

For a known mixture of n nuclides the release rates must be such that:

$$\sum_{i=1}^{n} \frac{Qi}{R_i} < 1$$

Appropriate release rate limits in $\mu Ci/s$ for each nuclide and release point will be provided to plant personnel for use in establishing monitor setpoints. The setpoint in counts per minute for each gaseous effluent monitor will be established using plant instructions. The general equation used by plant personnel in establishing setpoints in cpm from release rate limits in $\mu Ci/sec$ is the same as that used in Step 2a.

1.2 Monthly Dose Calculations

Dose calculations will be performed monthly to determine compliance with specifications 3.11.2.2 and 3.11.2.3. These specifications require that the dose rate in unrestricted areas due to gaseous effluents from each reactor at the site shall be limited to the following:

For noble gases,

1. During any calendar quarter, 5 mrad to air for gamma radiation and 10 mrad to air for beta radiation.

2. During any calendar year, 10 mrad to air for gamma radiation and 20 mrad to air for beta radiation.

For iodines and particulates,

1. During any calendar quarter, 7.5 mrem to any organ.

2. During any calendar year, 15 mrem to any organ.

L61 851030 800

H. L. Abercrombie, Site Director, NUC PR, Sequoyah Nuclear Plant

E. A. Belvin, Manager, Radiological Health, 401 UBB-C

OCT 30 1985

SEQUOYAH NUCLEAR PLANT - GASEOUS RELEASE RATE LIMITS

References:

1. Memorandum from G. F. Stone to H. J. Green, "Sequoyah Nuclear Plant - Revision of Acceptance Criteria for Sequoyah Nuclear Plant Gaseous Releases," dated March 11, 1981 (L00 810313 228)

2. Memorandum from E. A. Belvin to H. J. Green, "Sequoyah Nuclear Plant Environmental Technical Specifications," dated November 1, 1982 (L00 821104 763)

Radiological Health Technical Assistance Section has revised the nuclide specific gaseous release rate limits for Sequoyah Nuclear Plant (SQN). These updated release rate limits (attached) are mix independent and are based on the dose rate limits given in Radiological Effluent Technical Specification (RETS) 3.11.2.1.

The revised release rate limits, which are the result of revisions to the Gaseous Effluent Licensing Code (GELC), are intended to supersede the limits given in reference 1. Gaseous release rate limits for initial trip/set-point determinations have been previously provided (reference 2) and are based on the GELC revisions.

A comparison of the revised release rate limits with those contained in reference 1 indicates that the limits for noble gases have not been significantly affected, except for the short-lived Kr-89 and Xe-137. Limits for these two nuclides decreased by about a factor of 3.5. Release rate limits for particulates are generally lower by a factor of 1 to 11. However, the revised limits for iodines are higher by a factor of 1 to 5.

In addition, mix independent release rate limits have been determined based on the reporting requirements of 10 CFR 50.73 (2 times the 10 CFR 20 Appendix B, Table II concentrations in unrestricted areas). These limits are also included in the attachments. In general, the 10 CFR 50.73 release rate limits for noble gases are more restrictive than the RETS limits by a factor of about 1 to 5.6, except for Xe-137. Its 10 CFR 50.73 limit is more restrictive by a factor of about 27. Further, the 10 CFR 50.73 limits for iodines and particulates are, in general, significantly less restrictive than the RETS limits. Exceptions are Br-84, Br-85, Rb-88, and Pr-144.

H. L. Abercrombie

SEQUOYAH NUCLEAR PLANT - GASEOUS RELEASE RATE LIMITS

It should be noted that the 10 CFR 50.73 release rate limits are not instantaneous limits as are the RETS limits. The 10 CFR 50.73 limits are limits for reportable events and are to be averaged over a one-hour time period.

If we can be of further assistance, please let me know. Any questions may be directed to Regis Nicoll at extension 2532, Chattanooga.

E. A. Belvin

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Attachments
cc (Attachments):
NUC PR RIMS, 1520 CST2-C
Don Amos, NUC PR, Sequoyah
RARC Files (c/o B. M. Eiford, 401 UBB-C)
bc (Attachments):
C. E. Kent, 401 UBB-C

ATTACHMENT 1

SEQUOYAH NUCLEAR PLANT - NOBLE GAS RELEASE RATE LIMITS

Nuclide	RETS Limit (μCi/s)	10CFR20 MPC (μCi/cc)	10CFR50.73 Limit* (μCi/s)
Ar-41	1.3 (+4)	4(-8)	1.6 (+4)
Kr-85m	8.7 (+4)	1(-7)	3.9 (+4)
Kr-85	4.3 (+5)	3(-7)	1.2 (+5)
Kr-87	2.0 (+4)	2(-8)	7.8 (+3)
Kr-88	7.2 (+3)	2(-8)	7.8 (+3)
Kr-89	6.2 (+4)	3(-8)	1.2 (+4)
Xe-131m	9.0 (+5)	4(-7)	1.6 (+5)
Xe-133m	3.9 (+5)	3(-7)	1.2 (+5)
Xe-133	3.3 (+5)	3(-7)	1.2 (+5)
Xe-135m	4.1 (+4)	3(-8)	1.2 (+4)
Xe-135	5.2 (+4)	1(-7)	3.9 (+4)
Xe-137	3.2 (+5)	3(-8)	1.2 (+4)
Xe-138	2.1 (+4)	3(-8)	1.2 (+4)

*Limit (μCi/s) = 2 x MPC (μCi/cc) / $[5.12 \times 10^{-6}\ s/m^3][10^{-6}\ m^3/cc]$

Where $5.12 \times 10^{-6}\ s/m^3$ = worst site boundary X/Q listed in the SQN - ODCM.

ATTACHMENT 2

SEQUOYAH NUCLEAR PLANT - IODINE/PARTICULATE RELEASE RATE LIMITS

Nuclide	RETS Limit (μCi/s)	10CFR20 MPC (μCi/cc)	10CFR50.73 Limit* (μCi/s)
H-3	3.3 (+4)	2(-7)	7.8 (+4)
C-14	4.0 (+1)	1(-7)	3.9 (+4)
Na-24	1.1 (+3)	5(-9)	2.0 (+3)
Cr-51	1.4 (+3)	8(-8)	3.1 (+4)
Mn-54	1.1 (+1)	1(-9)	3.9 (+2)
Fe-59	1.4 (+1)	2(-9)	7.8 (+2)
Co-58	2.4 (+1)	2(-9)	7.8 (+2)
Co-60	1.7 (+0)	3(-10)	1.2 (+2)
Zn-65	2.9 (+0)	2(-9)	7.8 (+2)
Br-84	5.2 (+5)	3(-8)	1.2 (+4)
Br-85	9.1 (+7)	3(-8)	1.2 (+4)
Rb-88	3.5 (+5)	3(-8)	1.2 (+4)
Sr-89	4.7 (-1)	3(-10)	1.2 (+2)
Sr-90	8.3 (-3)	3(-11)	1.2 (+1)
Sr-91	1.1 (+3)	9(-9)	3.5 (+3)
Y-90	1.7 (+2)	3(-9)	1.2 (+3)
Y-91m	7.8 (+4)	6(-7)	2.3 (+5)
Y-91	5.7 (+0)	1(-9)	3.9 (+2)
Y-93	5.3 (+2)	5(-9)	2.0 (+3)
Zr-95	1.2 (+1)	1(-9)	3.9 (+2)
Nb-95	1.2 (+1)	3(-9)	1.2 (+3)
Mo-99	1.6 (+2)	7(-9)	2.7 (+3)
Tc-99m	3.6 (+4)	5(-7)	2.0 (+5)
Ru-106	3.2 (-1)	2(-10)	7.8 (+1)
Sb-124	7.7 (+0)	7(-10)	2.7 (+2)
Te-132	1.3 (+2)	4(-9)	1.6 (+3)
I-131	1.6 (-1)	1(-10)	3.9 (+1)
MI-131	1.2 (+1)	1(-10)	3.9 (+1)
I-132	1.1 (+3)	3(-9)	1.2 (+3)
MI-132	1.1 (+3)	3(-9)	1.2 (+3)
I-133	7.1 (+0)	4(-10)	1.6 (+2)
MI-133	5.2 (+1)	4(-10)	1.6 (+2)
I-134	5.4 (+3)	6(-9)	2.3 (+3)
MI-134	5.0 (+3)	6(-9)	2.3 (+3)
I-135	2.4 (+2)	1(-9)	3.9 (+2)
MI-135	2.6 (+2)	1(-9)	3.2 (+2)
Cs-134	3.7 (-1)	4(-10)	1.6 (+2)
Cs-136	1.6 (+1)	6(-9)	2.3 (+3)
Cs-137	3.6 (-1)	5(-10)	2.0 (+2)
Ba-140	4.3 (+1)	1(-9)	3.9 (+2)
La-140	2.7 (+2)	4(-9)	1.6 (+3)
Ce-141	3.6 (+1)	5(-9)	2.0 (+3)
Ce-144	9.9 (-1)	2(-10)	7.8 (+1)
Pr-143	6.3 (+1)	6(-9)	2.3 (+3)
Pr-144	7.6 (+4)	3(-8)	1.2 (+4)
Np-239	8.1 (+2)	2(-8)	7.8 (+3)

*Limit (μCi/s) = 2 x MPC (μCi/cc) / [5.12×10^{-6} s/m^3][10^{-6} m^3/cc]

TENNESSEE VALLEY AUTHORITY
Sequoyah Nuclear Plant
P. O. Box 2000
Soddy-Daisy, Tennessee 37379

July 14, 1986

Nuclear Regulatory Commission
Office of Management Information
and Program Control
Washington, DC 20555

Gentlemen:

SEQUOYAH NUCLEAR PLANT - MONTHLY OPERATING REPORT - JUNE 1986

Enclosed is the June 1986 Monthly Operating Report to NRC for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

P. R. Wallace

P. R. Wallace
Plant Manager

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