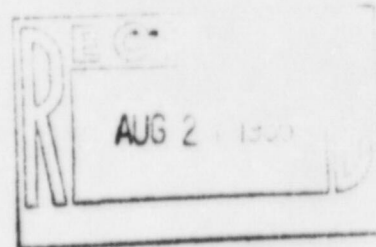


TEXAS UTILITIES GENERATING COMPANY
SKYWAY TOWER • 400 NORTH OLIVE STREET, L.B. 81 • DALLAS, TEXAS 75201

August 22, 1986

WILLIAM G. COUNCIL
EXECUTIVE VICE PRESIDENT



Mr. Eric H. Johnson, Director
Division of Reactor Safety and Projects
U. S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76012

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)
DOCKET NO. 50-445
STIFFNESS VALUES FOR CLASS 1 STRESS ANALYSIS
SDAR: CP-86-33 (INTERIM REPORT)

Dear Mr. Johnson:

On April 29, 1986, we verbally notified your Mr. T.F. Westerman of a deficiency involving the use of incorrect pipe support stiffness values in the Unit 1 Class 1 pipe stress analysis. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). We have submitted an interim report logged TXX-4831, dated May 28, 1986.

A reanalysis of all Class 1 stress problems in Unit 1 has been completed. No piping over-stresses were encountered due to support stiffness changes. The final evaluation of the effect of stiffness changes on valves, components, local stresses, penetrations, Reactor Coolant Loop nozzles, fatigue, and break locations is currently in progress. Stone & Webster Engineering Corporation (SWEC) is currently evaluating supports for the new loads resulting from the reanalysis.

We are continuing our evaluation and anticipate submitting our next report by October 17, 1986.

Very truly yours,

W.G. Council

W. G. Council

By: *G.S. Keeley*
G.S. Keeley
Manager, Nuclear Licensing

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