INDIANA & MICHIGAN ELECTRIC COMPANY DONALD C. COOK NUCLEAR PLANT

PROCEDURE COVER SHEET

Procedure No. 12 PMP 4021

Revision No. 1

TITLE

REACTOR TRIP REVIEW

SCOPE OF REVISION :

Major revision. Re-organized to reduce redundancy and to comply with PMI 2010. Added sign-off sheets 6 and 16 in response to SOER 83-07. Added sign-off sheets 7, 8, 17 and 18 to assist in organizing procedural requirements and assignments prior to reactor restart

authorization. Changed the Reactor Trip Breaker Undervoltage response requirement from a time measurement to observed only. Incorporated PC's 1 & 2. Organized sign-off sheets

1-8 for Unit 1 and 11-18 for Unit 2.

SIGNATURES	REVISION NUMBER			
*****	Rev. 1			
PREPARED BY	Marden	TINFORMATION	RECORDS CENTER ROLLED	-
DEPARTMENT HEAD APPROVAL	Blims	SER	3 0 1986 CS	TH
INTERFACING DEPARTMENT HEAD CONCURRENCE	Operations Plandau	VOLUME	Debrin	3
QUALITY ASSURANCE SUPERVISOR APPROVAL	meta			
PLANT NUCLEAR SAFETY COMMITTEE	mtg. #1967			
PLANT MANAGER APPROVAL	Nigo-			
APPROVAL DATE	9/25/86			
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DONALD C. COOK NUCLEAR PLANT

1.0 TITLE Reactor Trip Review

2.0 OBJECTIVE

- 2.1 To determine the exact cause of a Reactor Trip.
- 2.2 To ensure all required automatic responses associated with a reactor trip have either functioned properly or have been investigated and resolved.
- 2.3 To establish requirements for a Reactor restart.

3.0 REFERENCES

- 3.1 References on which the Procedure is based.
 - 3.1.1 NUREG 0977 and NUREG 1000, Vol 1 describing the ATWS events at Salem Nuclear Generating Station.
 - 3.1.2 The NRC's "Salem Restart Status Report" of March 28, 1983 response.
 - 3.1.3 Generic letter 83-28, "Required Actions based on Generic Implications of SALEM ATWS Events", July 8, 1983.
 - 3.1.4 AEP:NRC:0838A response to Generic letter 83-28.
 - 3.1.5 INPO Good Practice OP-211, "Post Trip Reviews", 9/83.
 - 3.1.6 10 CFR 50.73 (IV).
 - 3.1.7 1-OHP 4021.001.002 Rev. 15 and 2-OHP 4021.001.002 Rev. 9.
 - 3.1.8 Oconee Nuclear Station directive, Investigation of Unit Trips, Revision of 10/23/75.
 - 3.1.9 Oconee Nuclear Station Performance Manual, Section 4.7, Support of Reactor Trips, Revision of 2/16/83.
 - 3.1.10 Memo, V. VanderBurg to W.G. Smith 6/30/83.
 - 3.1.11 SOER 83-7 'Inadvertent Reactor Trips Caused by Instrumentation and Control Problems.'

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- 3.2 References to assist in procedure implementation.
 - 3.2.1 Elementary Diagrams
 - 3.2.1.1 98501 98515, Reactor Protection and Safeguards Logic Diagrams.
 - 3.2.1.2 98361 98377, Elementary Diagrams for Protection System and Safeguards, Train A.
 - 3.2.1.3 98381 98397, Elementary Diagrams for Protection System and Safeguards, Train B.
 - 3.2.1.4 1-98101, 2-98101, 2-98102, Turbine Control.
 - 3.2.1.5 98211, Steam Generator Feedwater Turbine E. 98212, Steam Generator Feedwater Turbine W.
 - 3.2.1.6 98021, Generator and Transformer Differential.
 - 3.2.1.7 98120, Turbine Events Monitor and Miscellaneous Recorders.
 - 3.2.1.8 98655, 98656, 98657, Operations Sequence Monitor.
 - 3.2.1.9 98665, 98666, 98667, Oscillograph.
 - 3.2.2 1200-A,B,C,G, one Line Electrical Diagrams.
 - 3.2.3 Recorder Chart Index, 4/21/81 Revision.
 - 3.2.4 Hathaway H634 Manual, Issue #5, May 1975.
 - 3.2.5 Hathaway H-559 Manual, Issue #1, April 1970.
 - 3.2.6 Westinghouse DI11 P250 Manual, 1/68 Revision.
 - 3.2.7 Westinghouse P250 Continuous Monitoring System Manual S2G-08A, issued 11/68.
 - 3.2.8 S2G-09B Westinghouse Post Trip Review Program Description, original issue.
 - 3.2.9 D. C. Cook Plant operating records.
 - 3.2.10 Appendix A: Miscellaneous information relating to the Operation of Trip Monitoring Devices.

4.0 DETAILED PROCEDURE

4.1 Responsibilities

- 4.1.1 The Shift Supervisor (SS) has the overall responsibility for the safe and reliable operation of the plant. The Shift Technical Advisor Section (STA) is responsible for the coordination of this procedure, the thoroughness and completeness of the procedure. As directed by the STA Supervisor, the STA is responsible for the drafting of the review report.
- 4.1.2 The use of "Not Applicable" is allowed for the completion of the procedure. 'N/A' will be written in the sign off space with an explanation describing the reason for not being applicable. This will then be followed by the initials of the person making this decision and the date of this decision.

4.2 Definitions

- 4.2.1 The classification scheme differentiates between "minor" and "major" safety-related equipment. The classification is relative and no implication is intended that "minor" equipment is unimportant. Whether a failure has serious safety significance shall be determined by those classifying the event.
 - 4.2.1.1 A "Major Equipment Failure" is where failure of equipment could have a substantial adverse impact on a normal plant shutdown or an accident situation. Such failures merit examination by the PNSRC prior to plant start-up. Examples: Failure of an auxiliary feedpump to start when required, failure of a reactor trip breaker to open, failure of a turbine trip system to depressurize.
 - 4.2.1.2 A "Minor Equipment Failure" is where failure of equipment does not have a substantial adverse impact on a normal plant shutdown or accident situation. Examples of minor equipment failures: Failure of a single reactor trip instrumentation channel or feedwater isolation actuation slightly below the Technical Specification value of 554°.

4.3 SignOff Sheets

- 4.3.1 The Signoff Sheets are sorted into two sets.
 The first set, Signoff Sheets 1 thru 8 is for
 Unit One and the second set, Signoff Sheets 11
 thru 18 is for Unit Two. When directed by the
 detailed procedure to use a signoff sheet, the
 first number is associated with Unit One and
 the second number, in brackets (), is
 associated with Unit Two. To avoid confusion,
 discard the signoff sheets from the unaffected
 Unit.
- 4.2.2 Signoff Sheet #6 (16), 'Corrective Action Review', is required to be completed and reviewed prior to "Reactor Restart Authorization", (4.8). This Signoff sheet will be initiated by the STA section with input from all affected departments who will sign the applicable subsection. Procedural defects will be only those procedure related errors which are due to logic errors, misprints, or failure to address a situation and not those errors related to personnel noncompliance or misunderstandings. Noncompliance will be a personnel error. Section C on Signoff Sheet #6 (16) is to be completed for personnel errors or precedural defects only. Section F on Signoff Sheet #6 (16) will be signed by the Plant Manager or his designee prior to recommendation of Reactor Restart Authorization.
- 4.3.3 Signoff Sheet #7 (17) is to be used as a guide for the requirements of the different event conditions.
- 4.3.4 Signoff Sheet 8(18) is to be used by the A for tracking the status of any items requiring action prior to "Reactor Restart Authorization". Required action could be to repair malfunctional equipment, make procedural changes, install additional work controls, provide additional training, etc. These would be for items causing the Reactor Trip or problems as the result of the Reactor Trip. The status of Job Orders, Procedural Changes, etc., will be monitored and noted on signoff sheet 8(18) by the STA.

4.4 PREREQUISITES

4.4.1 E-O, "Reactor Trip or Safety Injection" and/or other Emergency Operating Procedures directed to be performed by E-O have been performed to where the plant is stable.

And

4.4.3 A reactor Trip has occurred that was not part of a preplanned event of a surveillance test procedure.

And

4.4.4 Appropriate notification has been made as specified in PMI 7030, "Condition Reports and Plant Reporting", Attachment No 2, "Immediate Notifications by the Shift Supervisor of significant events."

4.5 DATA COLLECTION

- The Operations Department is to check the relays on Panel G and the A panels. Use Signoff Sheet #1 (11) and place an X to indicate actuated status. Actuated status is indicated by targets being shown or indicating lights being unlit. Relays with indicating lights are shown with a * by the relay number on Signoff Sheet #1 (11). If the Reactor Trip occurred with the turbine reset, the Operations Department is to check the First Hit annunciator in the EHC Cabinet, Unit One only. Record the "First Hit" on Signoff Sheet #1.
- The I&C Section is responsible for collecting copies of the charts listed on Signoff Sheet 2 (12), "Chart Collections". These copies are to include portions of the trace prior to and following the Reactor Trip. Copies are to be labeled appropriately.
- 4.5.3 All personnel involved with the Reactor Trip are to complete Signoff Sheet 3 (13), "Personnel Interview Form". As a minimum, the Unit Supervisor, the Control Room Reactor Operators and the Shift Supervisor are to complete these forms. The STA is to review these forms with the individuals to ensure completeness.

- 4.5.4 Using data obtained from the OSM, complete Signoff Sheet 4 (14), "Trip Response Times".
- 4.5.5 With the assistance of a Licensed Control Room Operator, the STA is to complete Signoff Sheet 5 (15), "System Status Response".

4.6 Data Analysis

NOTE: This Section should be performed by all those classifying the event.

- *4.6.1 Review Signoff Sheet 1 (11) "Relay Targets" for abnormal items. These items are then to be listed on Signoff Sheets 6 (16) and/or 8 (18).
- *4.6.2 Review the charts obtained [see Signoff Sheet 2 (12)]. Any traces which are abnormal and appear to be the result of degraded equipment are to be noted on Signoff Sheets 6 (16) and/or 8 (18).
- *4.6.3 Review Signoff Sheet 3 (13), "Personnel Interview Form". Any items noted in these interviews pertaining to procedural defects, personnel noncompliance, misunderstandings or equipment problems are to be listed on Signoff Sheets 6 (16) and/or 8 (18).
- *4.6.4 Review Signoff Sheet 4 (14), "Response Times". The Trip Response Times are to be compared to those responses from past Reactor Trips. These Time Responses may differ from past trips because of sequential differences due to varying initiating events. Where Responses Times differ from past Time Responses, verify it is not due to degraded equipment. When degraded equipment has been identified, the equipment is to be listed on Signoff Sheets 6 (16) and/or 8 (18).
- *4.6.5 Review Signoff Sheet 5 (15), "System Status and Response". Include any equipment malfunctions on Signoff Sheets 6 (16) and/or 8 (18).

^{*} Requires Signoff

4.6.6 Determine and list the exact cause of the Reactor Trip from the following applicable information sources:

Source	Cause	of	Reactor	Trip
OSM P-250 Post Trip Review P-250 Table of Alarms Interviews Others				

4.7 EVENT CLASSIFICATION

- 4.7.1 A Reactor Trip is classified by one of the next 3 conditions.
 - 4.7.1.1 The reactor trip is classified a Condition I event if the cause of the event is known (4.6.6), no major discrepancies in time responses due to degraded equipment (4.6.4) and no indication of improperly functioning safety related equipment.
 - 4.7.1.2 The reactor trip is classified a
 Condition II event if the cause of the
 trip is positively known (4.6.6), and
 some minor safety related equipment did
 not function properly. However, the
 malfunction has been corrected and the
 status of events or equipment malfunction
 directly related to the trip do not
 prevent a restart.
 - 4.7.1.3 The Reactor Trip is classified a Condition III event if the cause of the trip is not positively known, or some minor safety related equipment malfunctioned during the event and has not been repaired, or some major safety related equipment malfunctioned during the event, whether or not repairs have been made.
- 4.7.2 Review of 4.6, "Data Analysis", which has been summarized on Signoff Sheets 6 (16) and 8 (18) and in step 4.6.6 has determined the Rector Trip Event is classified as follows:

4.7.2.1	Concurrence between the SS and STA classifies the event as a Condition I event.
	,
	SS
	STA
	There is a disagreement between the SS and STA on the RX Trip event being a Condition I event. The Operations Superintendent along with SS or STA have decided to classify the event as a Condition I event.
	OPG Curve
	OPS Supv
	SS or STA
4.7.2.2	The Operations Superintendent and STA concur that the event satisfies the criteria for Condition II events.
	OPS Supv
	STA
	There is a disagreement between the Operations Superintendent and STA on
	the Reactor Trip event being classified a
	Condition II event. The Assistant Plant
	Manager Production, along with the
	Operations Superintendent or STA have decided to classify the event as a
	Condition II event.
	Asst. PM/Prod.
	110001 211/22001
	OPS Supv or STA

4.7.2.3	The Operations Superintendent a	and STA
	concur the Reactor Trip event s	satisfies
	the criteria for Condition III	events.

OPS Supv

There is a disagreement between the Operations Superintendent and STA on the Reactor Trip event being classified a Condition III event. The Assistant Plant Manager-Production, along with the Operations Superintendent or STA have decided to classify the event as a Condition III event.

Asst. PM/Prod

OPS Supv or STA

4.8 REACTOR RESTART AUTHORIZATION

4.8.1 For Condition I events with concurrence between the SS and STA (4.7.2.1), the SS has the authority and recommends the restart of the effected unit to the Plant Manager.

SS /

4.8.2 For Condition I events without concurrence between the SS and STA (4.7.2.1), the Operations Superintendent has the authority and recommends the restart of the effected unit to the Plant Manager.

OPS Supv

4.8.3 For Condition II events with concurrence between the Operations Superintendent and STA (4.7.2.2), the Operations Superintendent has the authority and recommends the restart of the effected unit to the Plant Manager.

OPS Supv

Page 9 of 10 Rev. 1 4.8.4 For Condition II events without concurrence between the Operations Superintendent and STA (4.7.2.2) the Assistant Plant Manager-Production has the authority and recommends the restart of the effected unit to the Plant Manager.

Asst PM Prod

4.8.5 For Condition III events (4.7.2.3) all open items associated with the trip must be reviewed by the PNSRC with no unresolved safety questions. The PNSRC has the authority and recommends the restart of the effected unit to the Plant Manager.

PNSRC

PANEL G RELAY TARGETS UNIT 1

	(HAA)				(HAA)	
T1 Turbine Valves	T2 Generator Relays	T3 Excitation Relays		T11 Turbine Hyd Trip Soleniod	T12 Stator Cooling Trip	T13 Trans. 1 Sudden Pressure
T4 Turbine Mech. Trip Solenoid	T5 Transformer Relays	T6 Emerg. P.B. Trip Gen. Panel	,	T12 Trans. 1AB Sudden Pressure	T15 Trans. ICD Sudden Pressure	T16 Trans 1 Tap Chgr Hndl in
UNIT DI	FFERENTIAL (HEA)			OVERALL DIF	FERENTIAL (HEA)	
X-U1	87X1-U1	87X2-U1	87	/X-OA1 87	X1-0A1	87X2-0A1

PANEL A RELAY TARGETS UNIT 1

	87-0A PH.3			87-T PH.3	87-G PH.3	
Panel A-4	87-0A FIL.2			87-T PH.2	87-G PH.2	
	87-0A PH.1			87-T PH.1	87-G PH.1	
Panel A-3		83B	25A		N 65	159N
		836	252			
.2		87-A1 PH.3		64-AF PH.1 PH.2		
Panel A-2		87-A1 PH.2			96-618	151-618
		87-A1 PH.1		64-GF РН.1 РН.2		

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PANEL A RELAY TARGETS UNIT 1

Panel A-8

Panel A-7

Panei A-6

125A

87T-1AB	PH.2
87T-1AB	PH. 1
87T-1CD	PH.3
87T-1CD	PH.2
87T-1CD	PH. 1

87T-1AB PH.3

27-4EP1 251TN-1AB 51TN-1AB

251TN-1CD

SITN-ICD

27-4EP3

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PANEL A RELAY TARGETS UNIT 1

				UNIT 1				
	Pane A-1			Panel A-10			Panel A-9	
51T-11A PH.1	51N- T11B	51T-11B PH. 1	51 12CD	25-TCD		51 12AB	25 TAB	
51T-11A PH.3		51T-11B PH.3	87-TCDLD PH.1	87-TCDLD PH.2	87-TCLD PH.3	87-TABLD PH.1	87-TABLD PH.2	87-TABLD PH.3
87-DGAB PH.1	87-DGAB PH.2	87-DGAB PH.3	87T-101CD PH.1	87T-101CD PH.2	87T-101CD PH.3	87T-101AB PH.1	87T-101AB PH.2	87T-101AB PH.3
37T-11A	87T-11A	87T-11A	TR101C) Voltage Low		TRIOIAE	Voltage Low	
РН.1	PH. 2	PH. 3	PH. 1	PH. 2	PH.3	PH.1 ★	PH. 2	PH. 3
87T-11B PH.1	87T-11B PH.2	87T-11B PH.3	51TN- 101CD		251TN- 101CD	51TN- 101AB		251TN- 101AB
51-DGAB	51N-DGAB	51-DGAB	4KV T1	ID Volt Low			A Voltage Lo	w
PH.1		PH.3	PH.1	PH.2	PH. 3	PH. 1	PH.2 ★	PH.3
						Page 4		

PANEL A RELAY TARGETS UNIT 1

	MII I				
	Panel A-13			Panel A-12	
51Т-11С РН.1	51N-T11C	51T-11D PH.1	87 RCP1 PH.1	87 RCP1 PH.2	87 RCP1 PH.3
51T-11C PH.3		51T-11D PH.3	87 RCP2 PH.1	87 RCP2 PH.2	87 RCP2 PH.3
87-DGCD PH.1	87-DGCD PH.2	87-DGCD PH.3	87-TBMC PH.1	87-TBMC PH.2	87-TBMC PH.3
87T-11C PH.1	87T-11C PH.2	87T-11C PH.3	87-TCMC PH.1	87-ТСМС РН.2	87-TCMC PH. 3
87T-11D · PH.1	87T-11D PH.2	87T-11D PH.3	87-RCP3 PH.1	87-RCP3 РН.2	87-RCP3 PH.3
51DGCD PH. 1	51N- DGCD	5 1DGCD PH. 3	87-RCP4 PH.1	87-RCP4 PH.2	87-RCP-4 PH.3

SWITCHGEAR COMPLEX RELAY TARGETS

DATE		PERFORMED BY:				
rgets actuated. Specify '	"1", "3", or "N" where	applicable. Note alarm l	ights lit			
Instrument No.	Name	Attempted	eset Successful			
	relays on back of breake	er panels (01,2,3 feeder	relays).			
ument No.	Name	Attempted R	eset Successful			
	s 4KV Switchgear: rgets actuated. Specify .). Instrument No.	s 4KV Switchgear: rgets actuated. Specify "1", "3", or "N" where a .). Instrument No. Name tchgear rgets actuated. Include relays on back of breake	Instrument No. Name Attempted Attempted Atchgear rgets actuated. Include relays on back of breaker panels (01,2,3 feeder			

UNIT ONE

EHC FIRST HIT ANNUNCIATOR

BACKUP & FAST CLST +30VDC OVERSPEED RESET LOST	TRIP	CUST.
RESET CLST		
	P B O	MA TRIP BACKUP BUS OVERSPEED ENFRGIZER TRIP
		2 (2

UNIT 1

CHART COLLECTION

The I & C Section is to obtain a copy of the following charts for all Reactor Trips:

Pressurizer Water Level
Pressurizer Pressure
Operations Sequence Monitor
P-250 Trend (Post Trip Review)
P-250 Alarm (Sequence of Events)
Control Room Log
Wide Range Temperature, All Four Loops.

As specified by the a copy of any char causing the trip.	ne cognizant STA, et associated wit	the I & C section is to the the specific transient	obtain
As specified by the associated with an	ne cognizant STA, ny equipment which	obtain a copy of charts the has malfunctioned:	•
Charts Reviewed By	':		

UNIT ONE

PERSONNEL INTERVIEW FORM

Person interview	red I	Role in Event	
Description of t	the event:		
Include the plant indications that result of those malfunctions or procedure deficit	a problem exi indications, i inadequacies,	isted, your act noted equipment	ion as a
INTERVIEWED BY			
INTERVIEWED BY REVIEWED BY			

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REACTOR TRIP EXPECTED RESPONSE (U1)

				A	Acceptance
Expected Response		Data Type	Requirement	Time	Criteria
Point #	(Initial Event)	Meas	All	0	0
44 Reactor Trip Breaker	A	Meas	All		*
45 Reactor Trip Breaker	В	Meas	A11		*
46 Reactor Trip Breaker	Undervoltage coil A	Meas	Obsv		Yes
47 Reactor Trip Breaker	Undervoltage coil B	Meas	Obsv		Yes
Control Rods Bottom		Obsv	All		Yes
Mechnical Trip Operated	(Turb)	Obsv	All		*
112 WMFPT Vacuum Trip		Neas	WMFPT Vac Trip Reset		, h
98 EMFPT Vacuum Trip		Meas	EMFPT Vac Trip Reset		*
118 WMFPT Hydr Pr Low		Meas	WMFPT Reset		*
104 EMFPT Hydr Pr Low		Meas	EMFPT Reset		*
Feedwater Isolation		Obsv	On Main Feed		Yes
WMDAFP Start		Obsv	On Main Feed, Aux		
			feed on standby		Yes
EMDAFP Start		Obsv	On Main Feed, Aux		
			feed on standby		Yes
TDAFP Start		Obsv	On Main Feed, Aux		Yes
			feed on standby		If start signal required

^{* -} To be compared to past trips as per 4.6.4

TURBINE TRIP EXPECTED RESPONSE (U1)

Expecied Response	Data Type	Requirement	Time	Acceptance Criteria
(Initial Event)	Meas		0	0
006 RT-Turbine Trip + P7	Meas	Turbine Reset & above F	-7	*
140 Mech Trip Oper	Meas	Turbine Reset		*
141 Emerg Gov OVSP Trip	Meas	Turb Reset		*
127 MT Reheat VA CL	Meas	Turb Reset		*
126 MT Stop VA CL	Meas	Turb Reset		*
157 Overall Diff Oper	Meas	Turb Reset		*
137 EHC Trip Sys Trip	Meas	Turb Reset		*
145 DC BRG Oil PP Run	Meas	Turb Reset		*
147 TRB Oil Buppp Run	Meas	Turb Reset		*
153 STM Seal PR Low	Meas	Turb Reset		*
Steam Dump Actuation	Obsv	Tavg Mode		Yes

^{* -} To be compared to past trips as per 4.6.4

GENERATOR TRIP EXPECTED RESPONSE (U1)

Expected Response	Data Type	Requirement	Time	Acceptance Criteria
(Initial Event)			0	0
Mech Trip Oper	Meas	Generator Exciter		*
157 Overall Diff Oper	Meas	Generator Exciter		*
159 Alterx Diff Oper	Meas	Generator Exciter		*
156 Generat Motoring	Meas	Generator Exciter		ķ
164 Unit/Sys Freq Hi Lo	Meas	Generator Exciter		*
151 Gen Cool Flow Lo	Meas	Generator Exciter		*
162 Gen Cool Trip Oper	Meas	Generator Exciter		*
Trans To Normal Reserve	Obsv	Gen Supplies Auxiliaries		Yes
Generator Field Breakers Open	Obsv	Field Breaker Closed		Yes
Generator Output Breakers Open	Obsv	Paralleled		Yes

*	_	To	be	compared	to	past	trips	as	per	4.6.
		40	87.6	compared		Bres or c		-	1	

REVIEWED	BY	1
		1
		,

UNIT ONE

SYSTEM STATUS AND RESPONSE

Nuclear Instrumentation

List all NI drawer "negative rate" trip status lights which are energized.

Source Range instrument power restored automatically as designed (both channels)

Major equipment out of service:

Describe any abnormal conditions (such as 'loss of detector voltage' or 'channel in test') or behavior noticed during the transient.

Reactor Coolant Pumps

Describe circumstances, including place in sequence of events, of any RCP trips (manual or automatic) which occurred during the transient.

Describe effect of reactor transient on leakoff flows.

Describe any behavioral abnormalities noticed during the transient.

UNIT ONE
RCS Pressure Control
Control statu; prior to trip (man/auto)
Backup heaters "on" banks
"auto" banks
Pressure channels selected (control/bistables)/
Pressurizer PORV's blocked prior to transient:
Spray valves in "auto"
Did pressurizer PORV's or safeties lift during transient? Describe circumstances.
Major equipment out of service:
Describe any behavioral abnormalities noticed during the transient
Pressurizer Level Control
Control Status prior to trip (man/auto)
Charging pump in service
Charging flow controller (QRV-251 or Recip speed controller) status prior to trip (man/auto)
Level channels selected (control/bistables)/

Describe any behavioral abnormalities noticed during the transient.

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CTT	CC	Ma	100	115
CA	60	ria	ve	up

E	rior	to	tr	ansient,	blender	line	contained
(acid,	pr	i.	water,	blend)		

Makeup in auto with 'start' signal? (yes/no)

Control and Shutdown Rods

Full insertion of all control and shutdown rods verified.

Rods that did not insert:

Rod control status prior to trip (auto/man)

Was the rod control status changed during the transient? If so, when in the sequence of events?

Major equipment out of service:

Describe any behavioral abnormalities noticed during the transient.

Main Feedwater/Steam Generator Level

Operating main feed pumps tripped on reactor trip (yes/no/OOS)

EMFP	
WMFP	

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<pre>< Feed water isolation received when Tavg < 554°F. Valves closed (yes/no)</pre>
FRV210 FRV220 FRV230 FRV240 FM0201 FM0202 FM0203 FM0204
Feedpump control status prior to trip (auto/man)
East feedpump speed control West feedpump speed control DP controller
Feedpump steam supply prior to trip
East feedpump (reheat/main/aux)
West feedpump (reheat/main/aux)
Feedwater regulating valve status prior to trip (auto/man)
FRV210 FRV220 FRV230 FRV240
Condensate bypass valve status prior to trip (auto/man) CRV224
Feedwater Heater level controls in manual prior to trip:

Major equipment out of service:

Describe any behavioral abnormalities noticed during the transient.

Auxiliary Feedwater

MDAFP's started automatically on trip of main feedpumps

EMDAFP		_
WMDAFP		

TDAFP opearated? Describe circumstances and approximate duration of operation.

Did operator intervene in the TDAFP response (place in manual or trip)? If so, when in the sequence of events?

Describe the response of the S/G levels.

Describe any behavioral abnormalities noticed during the transient.

Steam Dump/Steam Generator Pressure Relief

Control status prior to trip (Tavg/Steam Pressure)

If in Tavg mode, steam dump valves modulated to maintain Tavg ~547. (yes/no)

If Tavg dropped below 541°F, steam dump blocked (yes/no)

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UNIT ONE

Did steam dump valve open prior to trip (yes/no)? If yes desribe.

If vacuum trip, did steam dump valves stay closed (yes/no)?

Control status of atmospheric steam dumps prior to trip (auto/manual)

MRV213 MRV223 MRV233 MRV243

Did atmospheric steam dumps operate during transient (yes/no)? If yes, describe (automatically, manually, circumstances).

Was the control status of atmospheric steam dumps changed during the course of the transient? If so, when in the sequence of events?

Did steam generator safeties lift during transient (yes/no)? If yes describe.

Major equipment out of service.

Describe any other behavioral abnormalities noticed during transient.

Main Turbine/MSR

ine/MSK
Was there a turbine runback (yes/no)?
If yes, provide best estimate of power change and the time in sequence of events when it occurred.
Did the operator intervene in the turbine runback? If so, when in the sequence of events?
Turbine trip: Main turbine stop, control, and intercept valves closed
Controlling device prior to trip (load limiter/operating device)
Set rates loading
unloading
Were valve or misc turbine tests in progress when the trip occurred? Which one?
Did exhaust hood sprays actuate (yes/no)?
Vacuum breakers opened following trip? (yes/no)
'Hogging' SJAE's in service following trip? (yes/no)

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MSR	in	se.	rvi	ce	prior	to	trip?
(no,							

Did any MSR safety valves lift during the transient (yes/no/identify)?

Major equipment out of service:

Describe any behavioral abnormalities noticed during the transient.

Generator/Electrical

Generator trip. This will occur 30 seconds after the reactor trip unless it results from a generator event.

Output breakers open (yes/no)

К _____

K1

Exciter breaker open (yes/no)

If auxiliaries were suppled by the generator, were auxiliaries automatically transferred to normal reserve (yes/no/NA)? Describe any failures.

Diesel generators operated? Describe curcumstances.

UNIT ONE

Any CRID or CRP bus switched to alternate supply? Describe circumstances. (OSM point 154, white light on G panel, ann 19 drops 20 + 40, ann 20 drop 80 + 100).

Major equipment out of service:

Describe any behavioral abnormalities noticed during transient.

Safety Injection

Was the event associated with an SI (yes/no)?

If yes, did verification of automatic actuations indicate failure of any required response (yes/no)?

If yes, list actuations not received, reason if known, and whether the response was successfully initiated manually.

List major equipment out of service.

Describe any abnormalities noticed in the response of the safety injection system.

UNIT ONE

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Was there a phase B (yes/no)?

If yes, did verification of automatic actuations indicate failure of any required response (yes/no)?

If yes, list actuations not received, reason if known, and whether the response was successfully initiated manually.

List major equipment out of service.

Describe any abnormalities noticed in the response of the CTS System.

Miscellaneous

List any major plant equipment not covered above which is out of service and could have had an effect on the progression of the transient.

Describe any behavioral abnormalities of systems not covered above which may affect plant safety.

/
/

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UNIT ONE

CORRECTIVE ACTION REVIEW

The	deter	mined	cause	of the	trip	signal	was:	(che	ck one)	
	1.	Equip	ment F	ailure	-					
		b.	Failed Mode o Spurio Is thi	f fail us:	ure:	e failu	re?			
	2.	Proce	dural	defect	(see	proced	ure st	ep 4.	3)	
	3.	Perso	nnel E	rror (see p	rocedur	e step	4.3)		
STA:					_ S.:	s				
Depa	rtmen	ts Aff	ected:							
Imme	diate	actio	ns req	uired	iewed	by:	the fa	Dept	's Affector erro	cte
Imme	diate	actio	ns req	uired	iewed to mit	by: igate	the fa	Dept ilure	's Affector erro	cte
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STA S	diate r to signa ons p	actio restar ture:	ns requ	Dep	t's Ai	fected fected	the fa	ilure	's Affector erro	cte

Unit One

This checklist is to assist in organizing the requirements for reactor restart. The event condition determined by 4.7 "Event Classification" should be circled below and the other two columns N/A'd. The space provided is to be initialed and dated by the STA when the associated requirement has been completed.

	Condition One Event	Condition Two Event	Condition Three Event
Signoff Sheet 1			
Signoff Sheet 2			
Signoff Sheet 3			
Signoff Sheet 4			
Signoff Sheet 5			
Signoff Sheet 6			
Signoff Sheet 8			
PNSRC Approval	N/A	N/A	/

UNIT ONE

Items Requiring A		Assignments	
Status			
Time and Date	Status of Item	Requiring Action	STA
			-
			-

PANEL G RELAY TARGETS UNIT 2

	(HAA)			(HAA)			(HAA)
T17 Main Transf Sudden Press. PH.1	T18 Main Transf Sudden Press. PH.2	T19 Main Transf Sudden Press. PH.3	Tl Turbine Valves	T2 Generator Relays	T3 Excitation Relays	T11 Turbine Tripping (Right Sys)	T12 Stator Cooling Trip
T20 Trans. 2AB Sudden Press	T21 Trans 2CD Sudden Press		T4 Turbine Tripping (Left Syst)	T5 Transformer Relays	T6 Emerg. P. B. Trip Gen. Panel		
	UNIT DIFFERENTI	AL (HEA)		OVERALL	DIFFERENTIAL (H	EA)	
87X-U2	87X1-U2	87X2-U2	8	7X-0A2 871	X1-0A2	87X2-0A2	
	_						

12 PMP 4021 TRP.001 REACTOR TRIP REVIEW PROCEDURE SIGNOFF SHEET 11

PANEL A RELAY TARGETS UNIT 2

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			87-0A PH.1	87-0A PH.2	87-0A PH.3	
87-A 87-A 87-A PH.1 PH.2 PH.3	836	838				
	252	25A				
64-GF 64-AF			87-T PH. 1	87-T PH.2	87-T PH.3	
89-96		N 65	87-6 PH. 1	87-G PH.2	87-G PH.3	
151-65		159N				

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PANEL A RELAY TARGETS UNIT 2

	Pan A-				anel A-8		Panel A-9	Ţ
				125A			25 TAB	
87T-AB PH.1	87T-AB PH.2	87T-AB PH.3	87T-CD PH.1	87T-CD PH. 2	87T-CD PH.3	87T-201AB PH.1	87T-201AB PH.2	87T201AB PH.3
SITN-AB		251TN- AB	51TN-CD		251TN- CD	51TN- 201AB	TR-201AB Volt Low .1* PH.2* PH.	251TN 201AB 3*
						T2 PH.1*	1A Volt Low PH.2* PH	.3*

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PANEL A RELAY TARGETS UNIT 2

Panel A-10		Panel A-11			Panel A-12	
25-TCD	51T-21A PH.1	51N-T21B	51T-21B PH.1	87-RCP1 PH.1	87-RCP1 PH.2	87-RCP1 PH.3
	51T-21A PH.3		51T-21B PH.3	87-RCP2 PH.1	87-RCP2 РН.2	87-RCP2 PH.3
	87-DGAB PH.1	87-DGAB PH.2	87-DGAB PH.3	87-TBMC PH.1	87-TBMC PH.2	87-TBMC PH.3
87T-201CD 87T-201CD 87T-201CD PH.1 PH.2 PH.3	87T-21A PH.1	87T-21A PH.2	87T-21A PH.3	87-TCMC PH . 1	87-TCMC PH.2	87-TCMC PH.3
51TN- TR-201CD 251TN- 201CD Volt Low 201CD PH.1* PH.2* PH.3*	87T21B PH.1	87T21B PH.2	87T21B PH.3	87-RCP3 PH.1	87-RCP3 РН.2	87-RCP3 PH.3
T21D Volt Low PH.1* PH.2* PH.3*	51DGAB PH.1	51NDGAB	51DGAB PH.3	87-RCP4 PH.1	87-RCP4 PH.2	87-RCP4 PH.3

^{*} Relay is Actuated when white light is unlit.

PANEL A RELAY TARGETS UNIT 2

A-13		
51NT-21C	51T-21D PH.1	
	51T-21D PH.3	
87-DGCD PH.2	87-DGCD PH.3	
87-T21C PH.2	87-T21C PH.3	
87T-21D PH.2	87T-21D PH.3	
51N-DGCD	51-DGCD PH.3	
	87-DGCD PH.2 87-T21C PH.2 87-T21D PH.2	51NT-21C 51T-21D PH.1 51T-21D PH.3 87-DGCD 87-DGCD PH.2 87-T21C PH.2 87-T21C PH.3 87T-21D 87T-21D PH.3 51N-DGCD 51-DGCD

UNIT TWO

SWITCHGEAR COMPLEX RELAY TARGETS

TIME		DATE	PERFORMED BY:	
RC Pump Bus and T-l List all relay ("GF", "CT", PL, et	largets actuated.	Specify "1", "3", or "N" where ap	plicable. Note alarm lights	lít
Breaker	Instrument	No. Name	Reset Attempted	Successful
List all relay		Include relays on back of breaker Name	panels (feeder relays). Reset Attempted	Successful
	argets actuated.		Reset	Successful

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UNIT 2

CHART COLLECTION

The I & C Section is to obtain a copy of the following charts for all Reactor Trips:

Pressurizer Water Level
Pressurizer Pressure
Operations Sequence Monitor
P-250 Trend (Post Trip Review)
P-250 Alarm (Sequence of Events)
Control Room Log
Wide Range Temperature, All Four Loops.

	hart		the I & C section is to the specific transient	
			obtain a copy of charts has malfunctioned:	
Charts Reviewed	By:	/		

PERSONNEL INTERVIEW FORM

Person inter	viewed	1	Role in E	vent	
Description	of the eve	ent:			
Include the indications result of the malfunctions procedure de	that a propose indicator inadeq	blem exitions, ruacies,	sted, you	ir action ipment	as a
INTERVIEWED	ВҮ				
REVIEWED BY					
REVIEWED BY					

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UNIT TWO
REACTOR TRIP EXPECTED RESPONSE (U2)

Expected Response	Data Type	Requirement	Time	Criteria
(Initial Event)	Meas	All	0	0
44 Reactor Trip Breaker A	Meas	All		*
45 Reactor Trip Breaker B	Meas	All		*
46 Reactor Trip Breaker Undervoltage coil A	Obsv	All		Yes
47 Reactor Trip Breaker Undervoltage coil B	0bsv	All		Yes
Control Rods Bottom	0bsv	All		Yes
131 Main Turbine Left System Trip	Meas	All		*
132 Main Turbine Right System Trip	Meas	All		*
111 WMFPT Vacuum Trip	Meas	WMFPT Vac Trip Reset		*
99 EMFPT Vacuum Trip	Meas	EMFPT Vac Trip Reset		*
114 WMFPT Emergency System Trip	Meas	WMFPT Reset		*
102 EMFPT Emergency System Trip	Meas	EMFPT Reset		*
Feedwater Isolation (<554°F)	Obsv	On Main Feed		Yes
WMDAFP Start	Obsv	On Main Feed, Aux		Yes
EMDAFP Start	Obsv	On Main Feed, Aux		165
		feed on standby		Yes
TDAFP Start	0bsv	On Main Feed, Aux		Yes
		feed on standby		If start

^{* -} To be compared to past trips as per 4.6.4

UNIT TWO TURBINE TRIP EXPECTED RESPONSE (U2)

Expected Response	Data Type	Requirement	Time	Acceptance Criteria
(Initial Event)	Meas	Turb Reset	0	0
131 Main Turbine Left System Trip	Meas	Turb Reset		*
132 Main Turbine Right System Trip	Meas	Turb Reset		*
151 Left Emergency Circuit Trip (<8.5 psig)	Meas	Turb Reset		*
152 Right Emergency Circuit Trip (<8.5 psig)	Meas	Turb Reset		*
137 Control Fluid Safety Circuit Trip	Meas	Turb Reset		*
139 Auxilliary Lube Oil Pump	Meas	Turb Reset		*
140 West Emergency Lube Oil Pump Running	Meas	Turb Keset		*
141 East Emergency Lube Oil Pump Running	Meas	Turb Reset		*
121 Main Turbine Stop Valves Closed	Meas	Turb Reset		*
122 Main Turbine Intercept Valves Closed	Meas	Turb Reset		*
6 Reactor Trip from Turbine Trip	Meas	Above P-7		*
165 Overall Differential Generator Trip				
from Turbine Stop Valves Closed	Meas	Turb Reset		*
Steam Dump Actuation	0bs	Tavg Mode		Yes

^{* -} To be compared to past trips as per 4.6.4

UNIT TWO GENERATOR TRIP EXPECTED RESPONSE (U2)

Expected Response	Data Type	Requirement	Time	Acceptance Criteria
(Initial Event)	Heas	Generator Exciter	0	0
131 Main Turbine Left System Trip	Meas	Generator Exciter		*
132 Main Tubine Right System Trip	Meas	Generator Exciter		*
Transfer to Normal Reserve	0bs	Gen Supplies Auxiliaries		Yes
Generator Field Breakers Open	Obs	Field Breaker Closed		Yes
Generator Output Breakers Open	0bs	Paralleled		Yes

* - To be compared to past trips as per 4.6.4

Reviewed	by	1	
		1	
		1	

UNIT TWO

SYSTEM STATUS AND RESPONSE

Nuclear Instrumentation

List all NI drawer "negative rate" trip status lights which are energized.

Source Range instrument power restored automatically as designed (both channels)

Major equipment out of service:

Describe any abnormal conditions (such as 'loss of detector voltage' or 'channel in test') or behavior noticed during the transient.

Reactor Coolant Pumps

Describe circumstances, including place in sequence of events, of any RCP trips (manual or automatic) which occurred during the transient.

Describe effect of reactor transient on leakoff flows.

Describe any behavioral abnormalities noticed during the transient.

	UNIT TWO
RCS Press	ure Control
	Control status prior to trip (man/auto)
	Backup heaters "on" banks
	"auto" banks
	Pressure channels selected (control/bistables)
	Pressurizer PORV's blocked prior to transient:
	Spray valves in "auto"
	Did pressurizer PORV's or safeties lift during transient? Describe circumstances.
	Major equipment out of service:
	Describe any behavioral abnormalities noticed during the transient
Pressuriz	er Level Control
	Control Status prior to trip (man/auto)
	Charging pump in service
	Charging flow controller (QRV-251 or Recip speed controller) status prior to trip (man/auto)
	Level channels selected (control/bistables)
	Describe any behavioral abnormalities noticed during the transient.

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CVCS Makeup

Prior to transient, blender line contained (acid, pri. water, blend)

Makeup in auto with 'start' signal? (yes/no)

Control and Shutdown Rods

Full insertion of all control and shutdown rods verified.

Rods that did not insert:

Rod control status prior to trip (auto/man)

Was the rod control status changed during the transient? If so, when in the sequence of events?

Major equipment out of service:

Describe any behavioral abnormalities noticed during the transient.

Main Feedwater/Steam Generator Level

Operating main feed pumps tripped on reactor trip (yes/no/OOS)

EMFP					
WMPD					
WMFP	-	 	-	 	

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Feed water isolation received when Tavg <554°F. Valves closed (yes/no)
FRV210 FRV220 FRV230 FRV240 FM0201 FM0202 FM0203 FM0204
Feedpump control status prior to trip (auto/man)
East feedpump speed control West feedpump speed control DP controller
Feedpump steam supply prior to trip
East feedpump (reheat/main/aux)
West feedpump (reheat/main/aux)
Feedwater regulating valve status prior to trip (auto/man)
FRV210 FRV220 FRV230 FRV240
Condensate bypass valve status prior to trip (auto/man) CRV224
Feedwater Heater level controls in manual prior to trip:

Major equipment out of service:

Describe any behavioral abnormalities noticed during the transient.

Auxiliary Feedwater

MDAFP's started automatically on trip of main feedpumps

EMDAFP	
WMDAFP	

TDAFP opearated? Describe circumstances and approximate duration of operation.

Did operator intervene in the TDAFP response (place in manual or trip)? If so, when in the sequence of events?

Decribe the response of the S/G levels.

Describe any behavioral abnormalities noticed during the transient.

Steam Dump/Steam Generator Pressure Relief

Control status prior to trip (Tavg/Steam Pressure)

If in Tavg mode, steam dump valves modulated to maintain Tavg ~547. (yes/no)

If Tavg dropped below 541°F, steam dump blocked (yes/no)

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Did steam dump valve open prior to trip (yes/no)? If yes desribe.

If vacuum trip, did steam dump valves stay closed (yes/no)?

Control status of atmospheric steam dumps prior to trip (auto/manual)

MRV213 MRV223 MRV233 MRV243

Did atmospheric steam dumps operate during transient (yes/no)? If yes, describe (automatically, manually, circumstances).

Was the control status of atmospheric steam dumps changed during the course of the transient? If so, when in the sequence of events?

Did steam generator safeties lift during transient (yes/no)? If yes describe.

Major equipment out of service.

Describe any other behavioral abnormalities noticed during transient.

Main Turbine/MSR

Was there a turbine runback (yes/no)?

If yes, provide best estimate of power change and the time in sequence of events when it occurred.

Did the operator intervene in the turbine runback? If so, when in the sequence of events?

Turbine trip: Main turbine stop, control, and intercept valves closed

Controlling device prior to trip (load limiter/operating device)

Set rates loading _____

unloading

Were valve or misc turbine tests in progress when the trip occurred?
Which one?

Did exhaust hood sprays actuate (yes/no)?

Vacuum breakers opened following trip? (yes/no)

'Hogging' SJAE's in service following trip? (yes/no)

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	UNII IWO
	MSR in service prior to trip? (no/partial/full)
	Did any MSR safety valves lift during the transient (yes/no/identify)?
	Major equipment out of service:
	Describe any behavioral abnormalities noticed during the transient.
Generator	/Electrical
	Generator trip. This will occur 30 seconds after the reactor trip unless it results from a generator event.
	output breakers open (yes/no)
	A1
	A2
	Exciter breaker open (yes/no)
	If auxiliaries were suppled by the generator, were auxiliaries automatically transferred to normal reserve (yes/no/NA)? Describe any failures.
	Diesel generators opérated? Describe curcumstances.

UNIT TWO

Any CRID or CRP bus switched to alternate supply? Describe circumstances. (OSM Point 156, white light on G panel, ann 20 drops 80+100, ann 19 drops 20+40

Major equipment out of service:

Describe any behavioral abnormalities noticed during transient.

Safety Injection

Was the event associated with an SI (yes/no)?

If yes, did verification of automatic actuations indicate failure of any required response (yes/no)?

If yes, list actuations not received, reason if known, and whether the response was successfully initiated manually.

List major equipment out of service.

Describe any abnormalities noticed in the response of the safety injection system.

Phase !	-						_	_	*	-
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Was there a phase B (yes/no)?

If yes, did verification of automatic actuations indicate failure of any required response (yes/no)?

If yes, list actuations not received, reason if known, and whether the response was successfully initiated manually.

List major equipment out of service.

Describe any abnormalities noticed in the response of the CTS System.

Miscellaneous

List any major plant equipment not covered above which is out of service and could have had an effect on the progression of the transient.

Describe any behavioral abnormalities of systems not covered above which may affect plant safety.

COMPLETED B	Y _				
REVIEWED BY	_			/	
	_			/	
P	age lev.	10	of	10	

CORRECTIVE ACTION REVIEW

	gnal which initiated RPS:
The dete	ermined cause of the trip signal was: (check one)
1.	Equipment Failure -
	a. Failed component:b. Mode of failure:c. Spurious:d. Is this a repetive failure?
2.	Procedural defect (see procedure step 4.3)
3.	Personnel Error (see procedure step 4.3)
STA:	S.S
Departme	ents Affected:
Immediat	nature: Reviewed by: Dept's Affected: te actions required to mitigate the failure or error or restart of Unit:
-	
	nature: Dept's Affected: proposed to prevent recurrence:
	proposed to prevent recurrence:
Actions	proposed to prevent recurrence:

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Unit Two

This checklist is to assist in organizing the requirements for reactor restart. The event condition determined by 4.7 "Event Classification" should be circled below and the other two columns N/A'd. The space provided is to be initialed and dated by the STA when the associated requirement has been completed.

	Condition One Event	Condition Two Event	Condition Three Event
Signoff Sheet 11			
Signoff Sheet 12			
Signoff Sheet 13			
Signoff Sheet 14			
Signoff Sheet 15			
Signoff Sheet 16			
Signoff Sheet 18			
PNSRC Approval	N/A	N/A	/

Items Requiring Action		Assignments		
Status				
Time and Date	Status of Item	Requiring Action	STA	
			_	
			_	
	-		-	
	-			

Page 1 of 1 Rev. 1 APPENDIX A: Miscellaneous Information

Relating to the Operation of Trip Monitoring Devices.

I. HATHAWAY OPERATIONS SEQUENCE MONITOR

The sequence monitor provides a printed record of the operation of certain selected events. It has the capacity to monitor 192 on-off points and produces a line item output on a printer located in the control room when any one of the monitored points indicates an abnormal condition. Forty five points are used to monitor events related to reactor trip initiation or reactor trip circuit breaker position, 6 points monitor condensate or hotwell pumps, 13 points monitor feedwater heater extreme high level events, 6 points monitor onsite power diesel generators, 22 points monitor the main feedwater pumps, 40 points monitor the main turbine-generator, and 8 points monitor the step-up and auxiliary transformers and miscellaneous items.

The operations sequence monitor permits discrimination for contact closures which occur more than 2 milliseconds apart.

A contact closure will result in a line item printout on a dedicated printer located in the control room. The line item contains a 3 digit number for the day of the year, a 4 digit number for the hours a minute, a 2 digit number for the second, a 3 digit number for the milliseconds, an "A" indicating an off normal condition, a 3 digit number to identify the event to the operator. A sample output is attached to the end of this section.

The following information elaborates on the meaning of particular alarms that commonly appear as a result of reactor trips.

A. UNIT I

019 Reactor Trip, S.G. Lo Lo Water Level, Loop 4.

This alarm is a typical example of a signal from the RPS initiating a trip.

044 Reactor Trip Breaker Tripped, Train A.

This alarm originates from breaker position limit switches. It indicates the breaker is actually open.

046 Reactor Breaker Undervoltage, Train A.

This alarm indicates the undervoltage trip attachments have operated. They report late in spite of the fact UVTA's actuate the trip breakers for reactor trips other than manual trips. This situation results from the fact that the auxiliary relay the reports the event is part of an RL circuit. The delay is ~200 msec.

112 WMFP Vaccuum Trip

As indicated on elementary, Dia 1-98212-5 coordinate H-3, this alarm indicates a vacuum trip has occurred.

1118 WMFP HYDR. Press Low.

This alarm results when the MFP Hydr. Oil pressure reaches 130 psi dec.

140 Mechanical Trip Operated.

This alarm indicates a turbine trip has been initiated. As indicated on elementary diagram 1-98101, the following trip signals will initiate this alarm:

1) Overall differential

- 2) Unit differential
- 3) Thrust bearing wear & low bearing oil trip
- 4) MSR level
- 5) Turbine high vibration trip
- 6) Turbine solenoid trip
- 7) Loss of stator cooling turbine trip
- 8) Turbine low vacuum trip
- 9) EHC low hydraulic press trip
- 10) EHC trip system press trip
- 11) Turbine shaft P.P. low oil press trip
- 12) EHC master trip
- 13) Rx trip train A + SG HI HI or S.I.
- 156 Generator Motoring

As indicated on elementary Dia 1-98021-2 coordinates Cl, this alarm results from: (All valves closed or control valves at no load position) and (generator output breakers closed).

B. UNIT II

- Ol9 Reactor Trip, S.G. Lo Lo Water Level, Loop 4

 This alarm is a typical example of a signal from the RPS initiating a trip.
- 044 Reactor Trip Breaker Tripped, Train A

This alarm originates from breaker position limit switches. It indicates the breaker is actually open. (verbal comment by T. King)

046 Reactor Breaker Undervoltage, Train A

This alarm indicates the undervoltage trip attachments (UVTA's) have operated. They report late in spite of the fact UVTA's actuate the trip breakers for reactor trips other than manual trips. This situation results from the fact that the auxiliary relay that reports the event is part of an RL circuit. The delay is typically ~200 msec.

131 Main Turbine Left System Trip

This alarm indicates a turbine trip has been initiated. As indicated on Elementary Diagram 2-98101, coordinate D-2, the following trip signals will initiate this alarm:

- 1) Turbine solenoid trip control switch
- 2) Unit overall differential
- 3) Transformer and generator unit differential
- 4) Thrust bearing position trip
- 5) Moisture separator reheater high level trip
- 6) Turbine vibration
- 7) Lube oil pressure low
- 8) Loss of stator cooling
- 9) Reactor trip, SI, or steam generator hi-hi
- 10) Turbine low vacuum trip
- 11) High exhaust hood temperature

As indicated on Elementary Diagram 2-98101,

coordinate H1, this signal results from pressure switch
63X TSP indicating low pressure in the safety fluid

circuit. It is operated by pressure switch 2515 on the

Turbine Control Diagram, Figure PCS-4B-11 in the

training manuals.

151 Left Emergency Circuit Tripped

As indicated on Elementary Diagram 2-98102, coordinate D-7, this alarm results from pressure switch 63 ECL indicating that emergency circuit pressure dropped below 8.5 psig. It is operated by pressure switch 4591 on the Turbine Control Diagram, Figure PGS-4B-11 in the training manual.

NOTE: 63 indicates a pressure switch on an elementary diagram.

158 Generator Motoring

As indicated on Elementary Diagram 2-98021-2, coordinates D2, this alarm results from: (all valves closed OR control valves at no load position) AND (generator output breakers closed)

165 Turbine valve Trip overall Differential

As indicated on Elementary Diagram 2-98021-2, this signal results from a trip of the unit overall differential by turbine valves closed and required delays.

111 FPTW Vacuum Trip

As indicated on Elementary, Diagram 98217-3, coordinate D-4, this alarm indicates a vacuum trip has occurred. It is operated by limit switch 33X BVTW on a mechanical linkage. This mechanical linkage is operated by all trips (verbal coment by T. King) and is expected for FPT trips initiated with the vacuum trip reset.

114 FPTW Emergency System Trip

As indicated on Elementary Diagram 2-98217, coordinate H-3, this signal results from pressure switch 63X BESTW indicating low pressure in the emergency circuit.

NOTE: 33 indicates a limit switch on an elementary diagram.

NOTE: 63 indicates a pressure switch on an elementary diagram.

SAMPLE OSM OUTPUT

```
0 25 1630 02 401 T 300 CCCCOR 2 TIST PT
  026 1730 00 400 T 000 DCCOOK 2 TEST PT
                                               12 PMP 4021.TRP.001
  0 26 1830 00 400 T 000 DCCOOK 2 TEST PT
                                               APPENDIX A
  026 1930 00 401 T 000 DCCOOK 2 TEST PT
  026 2030 00 401 T 000 DCCOOK 2 TEST PT
  026 2130 00 401 T 200 DCCOOK 2 TEST PT
  026 2215 08 218 A 058 DG2AB HEA OPER
  026 2226 03 820 A 060 CG 2AB START
  026 2230 00 401 T 000 DCCOOK 2 TEST PT
 0 26 2330 30 400 T 000 DCCOOK 2 TEST PT
 027 0030 00 400 T 000 DCCOOK 2 TEST PT
 0 27 0 130 00 401 T 000 DCCCOK 2 TEST PT
 027 0230 00 401 T 000 DCCCOK 2 TEST PT
                                              -0425 1-27-83
Lip
 027 0332 00 401 T 000 DCCOOK 2 TEST PT
 027 0350 53 105 A 075 HTR IC LEVEL HI
 027 0424 58 082 A 149 MT VAC TRIP BLOCKED -
 827 8424 59 819 A 149 MT VAC TRIP BLOCKED
 027 0425 23 307 A 158 GENERAT MOTORING
 027 0425 25 128 A 313 RT LP3FDMT FL LO
 027 0425 25 217 A 045 REACT BKR TRIP B
 027 0425 25 237 A 044 PEACT BKR TRIP A
 027 0425 25 261 A 131 MT L SYSTEM TRIP
 027. 0425 25 282 A 132 MT R SYSTEM TRIP
     0425 25 436 A 028 PT LPIFDUT FL LO
 0 - 0425 25 461 A 151 LEFT EMERG CKT TRIP
 027 0425 25 463 A 006 RT TURB TRIP & PT
 027 0425 25 489 A 111 FPTW VACUUM TELP
 027 0425 25 535 A 099 FPTE VACUUM TRIP
 0 27 0425 25 610 A 329 RT PURG PNRATE TR
 027 C425 25 635 A 046 PEACT BKT. 'W A
 027 0425 25 639 A 047 FEACT SKR UV B
 027 0425 25 654 A 010 FT LP3FDWT FL LO
227 3425 25 656 A 137 MT CONFL SAFCKT TP
 027 0425 26 224 A 102 FPTE EMERGSYS TR
 027 0425 26 242 A 075 HTR IC LEVEL HI
 027 0425 26 298 A 102 FPTE EMERGSYS TR
 027 0425 26 669 A 114 FPTW EMERCSYS TR
 027 0425 27 480 A 016 3T SGI LET EX LO
027 0425 27 756 A 015 RT SGI LEY EX LO
0 27 0425 27 324 A 013 FT SG3 LEV EX LO
027 0425 28 389 A 017 ST SG2 LEV EX LC
227 8425 28 595 A 799 RT LP2FDVT FL LO
027 0425 28 644 A 318 RT SG3 LEV EX LO
227 0425 28 895 A 210 PT LP3FDVT FL LO
027 0425 28 924 A 009 PT LP2FDUT FL LO
027 0425 29 070 A 017 9T SG2 LEV EX LO
027 0425 29 174 A 008 37 LPIFD"T FL LO
027 0425 29 448 A 011 FT LP4FOWT FL LO
027 0425 29 856 A 011 PT LP4FDMT FL LO
027 0425 30 220 A 319 FT SG4 LEW EX LO
227 2425 34 467 A 113 FPT! CONTOIL FR LO
227 0425 34 845 A 121 FPTE COLTCIL PR LC
027 0425 35 271 A 101 FPTE CONTOIL PR LO
027 0425 36 217 A 311 RT LP4FDVT FL LO
027 0425 36 389 A 113 FPT! CONTOIL PP LO
027 0425 36 661 A 009 PT LP2FDVT FL LO
227 0425 36 868 A 131 FPTE CONTOIL PF LO
227 2425 38 227 A BII PT LP4FDVT FL LO
027 0425 38 622 A 209 RT LP2FDMT FL LO
027 0425 38 763 A 010 FT LP3FDVT FL LO
027 0425 38 799 A 011 RT LP4FDWT FL LO
027 0425 38 816 A 308 RT LPIFDET FL LO
                                               Page 9 of 40
027 0425 39 248 A 011 PT LP4FDVT FL LO
                                              Rev. 1
027 0425 39 264 A 008 RT LPIFOUT FL L3
```

027 0425 39 598 A 311 PT LP4FOUT FL LO

II. ESTERLINE ANGUS TURBINE EVENT MONITOR.

The turbine event monitor is a dual unit strip chart recorder. Each of the 2 charts has 20 on-off points. The speed of the continuously moving charts is changed after a trip initiation so that 24 hours of chart are advanced through the recorder in 24 seconds. Fast speed on Unit I is 3 inches/sec. Fast speed on Unit II is 1.5 inches/sec. The chart speed then returns to normal and a trip initiation event recurs. Two points, one on each chart, are used to monitor the Train A and Train B reactor trip circuit breakers, 2 points monitor electrical lockout relays which indicate an electrical system level trip, 16 points monitor the position of turbine emergency and pre-emergency valves

(stop and interceptor valves). The remaining points monitor various turbine trip initiating events.

The time discrimination between events is approximately 20 milliseconds when the chart is in high speed operation.

The data is displayed on 2 strip charts. Each point operates a heat pen which leaves a continuous trace on the thermally sensitized chart. The pens trace a printed line on the chart to indicate a normal condition. The pen moves off the printed line to a position approximately midway between the printed lines for 2 adjacent points to indicate an off normal condition. A sample strip chart is attached to the end of this section.

UNIT I PEN IDENTIFICATIONS

Stylus number Chart	Equipment or Device Being Monitored
1.	* Unit differential
2.	* Overall differential
3.	
4.	* Reactor trip TR-A
5.	* Reactor trip TR-B
	* Mechanical trip
6.	* AEP to master trip
7.	* EHC master trip
8.	* Back-up overspeed trip
9.	* Loss of speed
10.	* Loss of station battery
11.	* Trip system pressure EHC
12.	* Mechanical overspeed trip
	operated
13.	* Mechanical trip operated
14.	* Power load unbalance
15.	* AEP EHC trip system
16.	* Stop walves closed
17.	* Stop valves closed
1/.	* Reheat and Intercept valves
10	closed
18.	* Vibration trip operated
19.	* Trip system pressure HFA
20.	* Time
Stylus Number, Chart 2	Equipment or Device
21.	Equipment or Device Stop valve No. 1 closed
THE RESERVE OF THE PERSON NAMED IN THE PERSON	The state of the s
21.	Stop valve No. 1 closed
21. 22.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed
21. 22. 23.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed
21. 22. 23. 24.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed
21. 22. 23. 24. 25. 26.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed
21. 22. 23. 24. 25. 26. 27.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed
21. 22. 23. 24. 25. 26. 27. 28.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed
21. 22. 23. 24. 25. 26. 27. 28. 29.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed
21. 22. 23. 24. 25. 26. 27. 28. 29.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 6 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 6 closed Intercept valve No. 1 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 5 closed Intercept valve No. 1 closed Intercept valve No. 2 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 5 closed Intercept valve No. 1 closed Intercept valve No. 1 closed Intercept valve No. 2 closed Intercept valve No. 3 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 6 closed Intercept valve No. 1 closed Intercept valve No. 2 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 4 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33. 34. 35.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 5 closed Reheat valve No. 6 closed Reheat valve No. 6 closed Intercept valve No. 1 closed Intercept valve No. 2 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 4 closed Intercept valve No. 5 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33. 34. 35. 36.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 6 closed Intercept valve No. 1 closed Intercept valve No. 2 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 4 closed Intercept valve No. 5 closed Intercept valve No. 5 closed Intercept valve No. 6 closed Intercept valve No. 6 closed Intercept valve No. 6 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33. 34. 35.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 5 closed Reheat valve No. 6 closed Reheat valve No. 6 closed Intercept valve No. 1 closed Intercept valve No. 2 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 4 closed Intercept valve No. 5 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33. 34. 35. 36. 37.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 6 closed Intercept valve No. 1 closed Intercept valve No. 2 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 4 closed Intercept valve No. 5 closed Intercept valve No. 5 closed Intercept valve No. 6 closed Intercept valve No. 6 closed Intercept valve No. 6 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33. 34. 35. 36.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 5 closed Intercept valve No. 1 closed Intercept valve No. 1 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 4 closed Intercept valve No. 5 closed Intercept valve No. 6 closed
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33. 34. 35. 36. 37.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 6 closed Intercept valve No. 1 closed Intercept valve No. 2 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 5 closed Intercept valve No. 6 closed * Thrust bearing wear or low bearing oil trip operated * Low vacuum trip operated
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33. 34. 35. 36. 37.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 5 closed Reheat valve No. 6 closed Reheat valve No. 6 closed Intercept valve No. 1 closed Intercept valve No. 2 closed Intercept valve No. 3 closed Intercept valve No. 5 closed Intercept valve No. 6 closed * Thrust bearing wear or low bearing oil trip operated * Low vacuum trip operated * Moisture separator trip
21. 22. 23. 24. 25. 26. 27. 28. 29. 30. 31. 32. 33. 34. 35. 36. 37.	Stop valve No. 1 closed Stop valve No. 2 closed Stop valve No. 3 closed Stop valve No. 4 closed Reheat valve No. 1 closed Reheat valve No. 2 closed Reheat valve No. 3 closed Reheat valve No. 4 closed Reheat valve No. 5 closed Reheat valve No. 6 closed Intercept valve No. 1 closed Intercept valve No. 2 closed Intercept valve No. 3 closed Intercept valve No. 3 closed Intercept valve No. 5 closed Intercept valve No. 6 closed * Thrust bearing wear or low bearing oil trip operated * Low vacuum trip operated

*Those devices that will automatically activate the turbine sequence of events monitor hi-speed drive when operated.

UNIT II PEN IDENTIFICATIONS

Stylus number	Equipment or Device Being Monitored
1.	* Unit differential
2.	* Overall differential
3.	* Reactor bkr tripped TR-A
4.	* Reactor bkr tripped TR-B
5.	* Turbine trip left system
6.	* Loc Vacuum trip operated
7.	* Condenser A low Vacuum trip
1. 2. 3. 4. 5. 6. 7. 8.	* Condenser B low Vacuum trip
9	* Condenser C low Vacuum trip
10.	Spare
11.	Spare
12.	
	* Left emergency ckt tripped
13.	* Right emergency ckt tripped
14.	* Feed pump turbine "E" & "W"
	emergency trip
15.	* Turbine trip right system
16.	* Stop valves closed
17.	* Reheat stop and intercept
	valves closed
18.	* Vibration trip operated
19.	* Cont. fluid safety circ.
	tripped
20.	Time

Recorder Points 21-40

Stylus Number	Equipment or Device
21.	No. 1 stop valve closed
22.	No. 2 stop valve closed
23.	No. 3 stop valve closed
24.	No. 4 stop valve closed
25.	No. 1 reheat stop valve closed
26.	No. 4 reheat stop valve closed
27.	No. 2 reheat stop valve closed
28.	No. 5 reheat stop valve closed
29.	No. 3 reheat stop valve closed
30.	No. 6 reheat stop valve closed
31.	No. 1 intercept valve closed
32.	No. 4 intercept valve closed

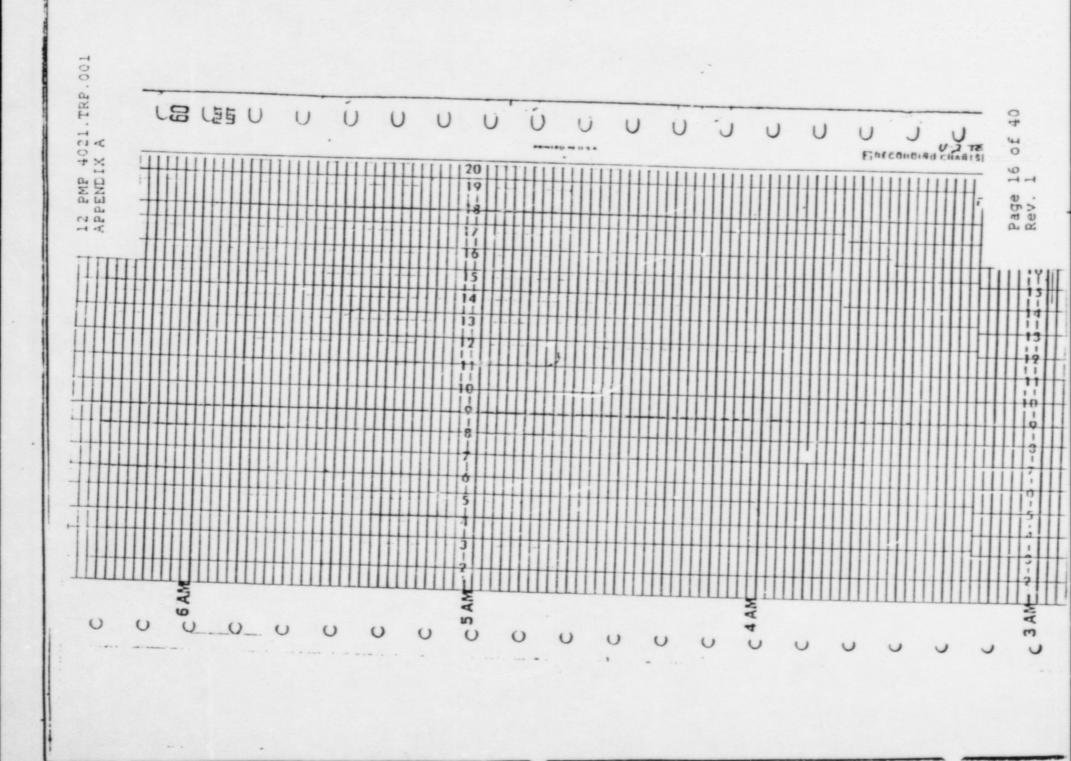
*Those devices that will automatically activate the turbine sequence of events monitor hi-speed drive when operated.

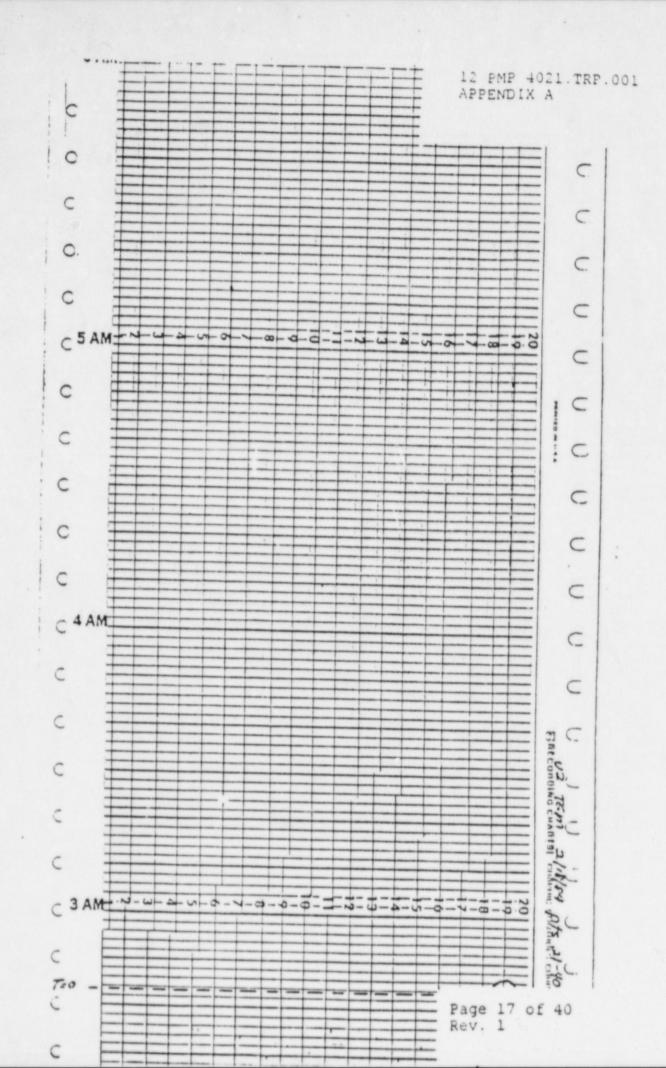
Recorder Points 21-40 (cont.)

Stylus Number	Equipment or Device
33. 34.	No. 2 intercept valve closed No. 5 intercept valve closed
35.	No. 3 intercept valve closed
36.	No. 6 intercept valve closed
37.	Feed pump turbine "E" emergency trip
38.	Feed pump turbine "W" emergency trip
39.	* Moisture separator Hi level trip
40.	Time

*Those devices that will automatically activate the turbine sequence of events monitor hi-speed drive when operated.

SAMPLE TEM STRIP CHARTS





III. UNIT I EHC FIRST HIT MONITOR PANEL

PS 100 A & B Are the Emergency Trip Pressure Switches which signal the electrical trip system logic that the Emergency Trip System has depressurized.

SPD SIG LOST Activated by concurrent loss of primary and secondary speed signals with turbine speed greater than 200 RPM.

CUST. TRIP Customer trips are the following: Thrust bearing wear & low bearing oil trip Steam Generator High Level Overall differential Unit differential MSR high level Reactor trip (P-7) Turbine high vibration (1 right plus 1 left) Solenoid trip (Control Switch) Loss of stator cooling Low condensor vacuum EHC hydraulic pressure low: 1100 PSIG EHC system pressure trip: 800 PSIG Shaft pump oil pressure low > 1300 RPM Safety injection

MA TRIP BUS ENERGIZER	Indicates that a turbine trip has occured, and the master trip bus has been energized. Also indicates trip is sealed in.
NO EHC DC INPUT POWER	Loss of 24 V DC @1800 RPM or 250 V DC if < 1800 RPM. Verify "No Station Battery" Annunciator in Miscellaneous Turbine Test Cabinet.
BACKUP OVER- SPEED TRIP	Activated by excessive turbine speed.
POWER LOAD UNBALANCE	Initiates rapid control and intercept valve closure on greater than 40% power/load mismatch.
FAST CLST	Rapid closure of intercept valves demanded by turbine supervisory instruments.
-22 VDC LOST OR +30 VDC LOST	DC supply for electrical control lost. Verify indication on lambda power supplies to the left of the First Hit Panel.

IV. HATHAWAY OSCILLOGRAPH

The unit oscillograph has 32 galvanometers. Each galvanometer will record one analog channel or, if properly modified, 4 on-off functions. Eight galvanometers have been converted to on-off functions and the remaining galvanometers are reserved for electrical analog quantities. The unit has a prefault recording feature where all input quantities are continuously recorded on a magnetic disc. Under normal conditions, the data are erased and current recordings written over the old space after approximately 100 milliseconds. If one of a specific set of events occurs, the data are recorded on ultra-violet sensitive photographic paper such that the information recorded prior to the event is recorded followed by additional data resulting from the event. The recording is continued for a fixed time period following the event. Recording chart speed may be selected to be either 12" or 3" per second, the usual practice being to record the initial portion of the event at the higher chart speed followed by additional recording at the slower chart speed.

Six points are used to monitor the A and B train reactor trip circuit breaker positions, undervoltage trip initiation, and safety injection actuation, 2 points monitor the start of onsite power diesel generators, 1 point monitors the trip of the feedwater pumps, 10 points monitor turbine initiated events, 7 points monitor generator and excitation events, and 4 traces are used for references to assist in identification of trace

locations. The analog traces record generator phase currents, phase and ground voltages, and field current.

The display provided by the developed photographic paper is a reproduction of the amplitude and wave shapes of the analog electrical quantities. The on-off events are indicated by a continuous straight line trace for a normal condition or the absence of the trace at that location signifying an off normal event. The photographic paper is developed by exposure to ultra-violet light (fluorescent lights are adequate sources) and no wet chemical processes are required. A sample strip chart is included at the end of this section.

The time descrimination between events during higher chart speed is better than 5 milliseconds between events and better than 10 milliseconds during slower chart speed.

DONALD C. COOK NUCLEAR PLANT

	INTT 11 OSCILLOGRAPH 1/84		
TRACE NO.	TRACE ASSIGNMENT	BATIO	CALLBRATION
		7000/1	17.960 A/In.
1 1 1	Generator Current 61		
11	Spare		
1111 3	Generator Current #3	7000/1	37,960 A/In.
	Spare		
	Spare		
- 6	Spare		
,	Gen. Grounding Trans. Voltage	41.5/1	220 V/tn.
	Source		
(9)	Gen. Field Current Zero Mirror		
9	Gen. Field Current (Shunt-4000 A/100 M.V.)	160 M.V./Ia.	9,465 A/10
- 10	SPACE.		
M. "	Gen. Metering Pot. Voltage	220/1	227 V/La
12	Spare		
W 13	345 KV Pot. Timing Trace	1800/1	213 V/1a
14	Spare (Corrent)		
- 15	Spare		
	Spare (Current)		
17	OMA Traces: 1-Reference: 2-Reactor Straker Tripped 'A'; 3-Reactor Steaker Tripped 'B'; 4-Reactor Steaker Under- volt Trip 'A'		on/off
1.8	ON6 Traces: L-Safety Injection 'A'; 2-Safety Injection 'B'; 3-Olesel Gen. 'AB'Start; 4-Stesel Gen. 'CD' Start.		on/off
19	OMA Traces: 1-Reference; 2-Feed PP. Turb. 'E'. 'Y' Trip: 3-Spare; 4-Spare.		on/off
20	OM4 Traces: 1-Main Stop Valves Closed: 2-Main Turb. Mech Trace: 3-Emerg. Gov. Overspeed Trip; 4-dack-up Overspeed		On/Off
=======================================	ONG Traces: 1-Reference: 2-FRC System Trip: 1-Reactor Bkr. Undervolt Trip '3'; 4-Thrust Searing Trip.		On/Off
12	OHA Traces: 1-Moisture Separator Hi. Level Trip; 2- Vacuum Trip Operacing; 3-Main Turo, Bigh Vib. Trip; 4-Lube Oil Press. Low Trip.		on/off
::	Temp. High; J-Stator Cool Turb. Trip: 4-Stator Cooling Water Trip:		Un/Off
=======================================	OM4 Traces: 1-Generator Motoring; 1-Unit HEA Operated; 3-Jueral: HEA operated; 4-Alterrex & Excitation Trip.		on/off

OSCILLOGRAFY STARTING SENSOR CALIBRATION

1.-38 Undervoltage - - 109 /AC Note: Overcurrent - (Not Used)

2.-30 Overvoltage - - 130 VAC

3.-18 Overvoltage - - 50 VAC

Page 2

*--Any on/off operation will start oscillograph

Rev. 1

U-2	A 52 14 1	10621PH
0	03515	

TN	O. Trace Assignment	APPEND	9/11/84
	- 1 race Assignment		
1	Generator potential E pl-2 volts		
1		220/1	130 V/i
	Generator potential E #2-3 volts	220/1	120 11/1
1			128 V/i
, ,	Cenerator potential E #1-3 volts		
	Volts	220/1	124 V:/i
4	Spare		
10	Spare	1	
7	Generator nutral voltage		
	The state of the s		126 V/I
8	Generator current I gl		
-	denotator current I pl	7,000/1	36,119
9	Generator current I #2	7,000/1	36,119 ;
10	Generator current I p3	7,000/1	43,914 A
-11	Spare		
12	Generator field current		
	1 200 6	==	
13	TSPAFS.		
16	765 KV timing trace		
			128 V/in
	OM4 Traces: 1-Reference; 2-Reactor Breaker Tripped "A"; 3-Reactor Breaker Tripped "B";		
17	Tribacor Breaker Undervoltage Trip "A"		On/Off
17	OM4 Traces: 1-Safety Injection "A"; 2-Safety Injection "B"' 1-Diesel Generator AB Start; 4-Diesel OM4 Traces: 1-Patterness 2-Patterness	 	On/Off
	OM4 Traces: 1-Safety Injection "A"; 2-Safety Injection "B" 1-Diesel Generator AB Start; 4-Diesel OM4 Traces: 1-Perference; 2-Feed Pump Turbine "E" and "W" Trip; 1-Feed Pump Turbine "E" and "W" Trip (Emergency): 4-Source		-
18	OM4 Traces: 1-Safety Injection "A"; 2-Safety Injection "B" J-Diesel Generator AB Start; 4-Diesel OM4 Traces: 1-Refurence; 2-Feed Pump Turbine "E" and "W" Trip; 3-Feed Pump Turbine "E" and "W" Trip (Emergency): 4-Spare OM4 Traces: 1-Vacuum Trip Operated; 2-Nain Stop Valve Closed; 3-Main Turbine Left System Trip; 4-Main Turbine Right System Trip;		On/Off
18	OM4 Traces: 1-Safety Injection "A"; 2-Safety Injection "B"' 1-Diesel Generator AB Start; 4-Diesel OM4 Traces: 1-Reference; 2-Feed Pump Turbine "E" and "W" Trip; 1-Feed Pump Turbine "E" and "W" Trip (Emergency): 4-Spare OM4 Traces: 1-Vacuum Trip Operated; 2-Nain Stop Valve Closed; 1-Main Turbine Left System Trip; 4-Main Turbine Right System Trip. OM4 Traces: 1-Reference; 2-Control Fluid Safety Circ. Tripped; 1-Pactor Breaker Undervoltage		On/Off On/Off
18	OM4 Traces: 1-Safety Injection "A"; 2-Safety Injection "B"' 3-Diesel Generator AB Start; 4-Diesel Generator CD Current "W" Trip; 3-feed Pump Turbine "E" and "W" Trip; 3-feed Pump Turbine "E" and "W" Trip (Emergency): 4-Spare OM4 Traces: 1-Vacuum Trip Operated; 2-Nain Stop Valve Closed; 3-Main Turbine Left System Trip; 4-Main Turbine Right System Trip. OM4 Traces: 1-Reference; 2-Control Fluid Safety Circ. Tripped; 3-Paactor Breaker Undervoltage Trip "B"; 4-Thrust Paaring Trip OM4 Traces: 1-Moisture Separator Hu-Level Trip; 2-Air-		On/Off On/Off On/Off On/Off
18	OM4 Traces: 1-Sifety Injection "A"; 2-Sifety Injection "B"' 1-Diesel Generator AB Start; 4-Diesel OM4 Traces: 1-Reference; 2-Feed Pump Turbine "E" and "W" Trip; 1-Feed Pump Turbine "E" and "W" Trip (Emergency): 4-Spare OM4 Traces: 1-Vacuum Trip Operated; 2-Nain Stop Valve Closed; 1-Main Turbine Left System Trip; 4-Main Turbine Right System Trip; OM4 Traces: 1-Reference; 2-Control Fluid Safety Circ. Tripped; 1-Peactor Breaker Undervoltage Trip "B"; 4-Thrust Rearing Trip Oil, H2 Diff. Press. Low Trip; 1-Main Turbine High Vib. Trip; 4-Lube Oil Press Low Tr		On/Off On/Off On/Off
18	OM4 Traces: 1-Safety Injection "A"; 2-Safety Injection "B"' 3-Diesel Generator AB Start; 4-Diesel Generator CD Current "W" Trip; 3-feed Pump Turbine "E" and "W" Trip; 3-feed Pump Turbine "E" and "W" Trip (Emergency): 4-Spare OM4 Traces: 1-Vacuum Trip Operated; 2-Nain Stop Valve Closed; 3-Main Turbine Left System Trip; 4-Main Turbine Right System Trip. OM4 Traces: 1-Reference; 2-Control Fluid Safety Circ. Tripped; 3-Paactor Breaker Undervoltage Trip "B"; 4-Thrust Paaring Trip OM4 Traces: 1-Moisture Separator Hu-Level Trip; 2-Air-		On/Off On/Off On/Off On/Off

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SAMPLE OSCILLOGRAPH STRIP CHART

DONALD C. COOK STUCLEAR PLANT

UNIT NO. ? OSCILLOCRAPH

TRACE	NO.	TRACE ASSIGNMENT	RATIO	CVLIBEATI
M	1	Generator Potential Edi-2 Voics	220/1	130 7/tn
	2	Generator Potential E#2-3 Volta	220/1	128 V/ta
	3	Ganaracor Potential E03-1 Volts	220/1	124 V/ta
M	٠	Generator Current [3]	7,000/1	36,119 A/in
-	5	Spare		
	6	Generator Neutral Voliage		126 7/10
		Scare		
M	8	Generator Current 102	7,000/1	36,119 A/in
V.V.	9	Generator Current [9]	7,000/1	43,914 A/LD
-	10	Space		
-	-	Spare		
-	12	Spare		-
i A AC	-	3 Pare		
	14	765 KV Timing Trace		125 V/Lm.
	:5	OM4 Traces: 1-Reference; 2-Reactor Breaker Tripped 'A"; J-Reactor Breaker Tripped "B"; 4-Reactor Breaker Undervolose Trip "A"; OM4 Traces: 1-Safety Injection 'A"; 1-Safety Injection "B"		Oa/Off
	16	Just dederator Ab Start; Diesel Gener-		00/000
	1,7	OM4 Traces: 1-Reference; 2-Feed Pump Turbine "E" and """ Trip; 1-Feed Pump Turbine "E" and """ Trip (Emergency); 4-Space		On/Off
	18	OM Traces: Livacyum Trip Coerated: Laft so scop Trip:		Ca/Off
	19	OM4 Traces: 1-Reference; 2-Control Fluid Sefety Circ. Tripped; 3-Reactor Breaker Undervoltage Trip "B"; 4-Thrust Bearing Trip		On/Ott
	20	OM4 Traces: 1-Moisture Separator Hi-Level Trip; 2-Air- Oil, H ₂ Diff. Press. Low Trip; 1-Main Tur- bine High Vib. Trip; 4-Lube Oil Press. Low Trip.		On/Off
	21	OM4 Truces: 1-Reference; 2-Stator Outlet Cooting Water Temp High: 3-Stator Cooting Turbine Trup; 4- Stator Cooling Generator Trip;		On/Off
	22	1994 Traces: t-denerator Potoring: 1-Unit Heat Operated; 1-Overall Heat Operated: 6-Alterrex and Excitation Trip		00/068
		OSCILLOGRAPH STARTING SENSOR CALIBRATION 1. 102 VAC 1. 102 VAC 1. 103 OVERVOLENCE E81-2 - 1 1. 103 VAC 1. 104 OVERVOLENCE E81-1 - 1 1. 105 VAC 1. 107 VAC 1. 108 OVERVOLENCE E81-1 - 1 1. 108 OVERVOLENCE E81-1	19 VAC	e 25 of

J. 14 Overvoltage 52 VAC

^{4.} May safeff operation will start Rev. 1

V. P250 SEQUENCE OF EVENTS RECORDING PROGRAM

The Sequence of Events Recording program records the sequence of operation of a number of monitored contacts to a high time resolution. When one of the monitored contacts changes state, an interrupt is initiated which causes the P250 to scan each monitored contact for any change from its previous state. The program stores such changes and the cycle count since the first event. A cycle is approximately 20 milliseconds in length. Due to a dead time of 2 milliseconds in the interrupt process, an automatic rebid of the program is programmed for the cycle following each interrupt bid. This is done to avoid loss of contact changes during the dead time. The Sequence of Events Recording program is terminated when either the cycle count reaches 3600 or 25 contact changes have been recorded.

When the program is terminated, an output routine is called. All collected data are first moved to the output program buffers to free the Sequence of Events Recording program buffers for continued monitoring. The output routine prints the time of the first event in hours, minutes, and seconds. Following this message, the alpha-numeric address, a 36 character contact description, and cycle count from the first event are printed for each contact change. The first event will always have a cycle count of zero. A sample output is included at the end of this section.

SEQUENCE OF EVENT ADDRESSES (Reference P250 Manual TPS129) F0403D RCL LO F ABOVE P-8 CAUS RE F0423D RCL LO F ABOVE P-7 CAUS REF F0493D STM LINE HI F SI CAUS RE STM GEN A LO LO L CAUS RE L0406D L0426D STM GEN B LO LO L CAUS RE L0446D STM GEN C LO LO L CAUS RE L0466D STM GEN D LO LO L CAUS RE PRESSURIZER HI 1 CAUS RE L0483D PWR RNG CHAN HI Q CAUS RE N0005D PWR RNG CHAN LO Q CAUS RE NOOLOD N0024D INTERM RNG HI Q CAUS RE PWR RNG CHAN HI Q RATE CAUS RE N0029D N0036D SOURCE RNG HI Q CAUSE RE P0407D STM LINE A HI DP SI CAUS RE P0427D STM LINE B HI DP SI CAUS RE P0447D STM LINE C HI DP SI CAUS RE STM LINE D HI DP SI CAUS RE P0467D P0483D PRESSURIZER HI P CAUS RE P0488D PRESSURIZER LO P CAUS RE P1003D CONTAINM HI P SI CAUS RE RCL OVERTEMP DI CAUS RE T0498D T0499D RCL OVERPWR DT CAUS RE V0324D RCP BUS UNDER VOLT &P7 CAUSE RE Y0004D REAC MANUAL TR 1 CAUS RE Y0005D REAC MANUAL TR 2 CAUS RE Y0006D REAC MAIN TR BKR A Y0007D REAC MAIN TR BKR B Y0026D REAC AUX TR BKR A Y0027D REAC AUX TR BKR B RCP BUS UNDER FREQ PART RE Y0320D Y0321D RCP BUS UNDER FREQ PART RE Y0322D RCP BUS UNDER FREQ PART RE Y0323D RCP BUS UNDER FREQ PART RE RCP BUS UNDER FREQ CAUS RE Y0324D Y0335D UNIT ON LINE TIE OCB A1 BKR Y0335D UNIT ON LINE TIE OCB A2 BKR Y0337D UNIT ON LINE TIE OCB B1 BKR Y0390D TB TRIP CAUSE RE TB STOP VLV A CI PART RE Y0391D Y0392D TB STOP VLV B CI PART RE Y0393D TB STOP VLV C CI PART RE Y0394D TB STOP VLV D CI PART RE Y0400D RCPA BKR OP CAUS RE Y0401D STM GEN A LO L & FW F CAUS RE Y0420D RCPB BKR OP CAUS RE Y0421D STM GEN B LO L & FW F CAUS RE Y0440D RCPC BKR OP CAUS RE STM GEN C LO L & FW F CAUS RE Y0441D Y0460D RCPD BKR OP CAUS RE Y0461D STM GEN D LO L & FW F CAUS RE Y0480D PRESUZER LO P&L SI CAUS RE Y0920D SFTY INJ SET MANUAL 1 CAUS RE Y0921D SFTY INJ SET MANUAL 2 CAUS RE

SAMPLE SEQUENCE OF EVENTS OUTPUT

	ALAKE HI TUATUA AT KELA OVERTERY E	1 1 62		178.5	H 152.0	DEGF
025	MEIRN OR TOUSON AT BULB OVERTEAP OF 1 SP	159.4	DEOF			
			Dt 66			
6257	ALIAN ON TOUGHT AT ALLE OVERTERS OF 1 SP	140.3	DEGF			
0257	REIRN OR 10470A AT RELE OVERTEAP DE 1 SP REIRN OR 10470A AT RELO OVERTEAP DE 1 SP / ALAKM LO TUAUDA AT RELA 1 TAVE			546.7	L 552.0	DLUF
025	ALAKA LO IDADDA AL RELA I INVO					
0257	SEQUENCE OF EVENTS RECORD. FIRST EVENT AT H 2 856 540			547.5	L 553.0	DEBF
025	ALAKA LO TUAZUA AL KELB I TAVU			347.3	r 222.0	
	103900 IS INTO CAUSE RE IN C O					
6758	ALTEN LO SUSADA AL UNIT SEMERATOR GROSS ME	3.0	L -2.0 AW		H 152.0	DEGF
0250	ALARM MI TUASOA AL RELE OVERTENP L	1 1 86		152.4	H 132.0	Dr. Gr
	TORDED ALAE MAIN IN EXK B				L 552.0	DEGE
025	THE TENTH AT WILL T TAUL			545.6		PC
025	ALANA HI UUAUB CV HILA OVERPUR BE	DEU FR	COMPUTED	104.2	11 4.0	rL
	THUUCD LIAC MAIN IN BER A IN C 5					
0251	ALAKA LO TUSCUA AL KELD I TAVE			545.2	L 552.0	DEUI
	PROTES AND BUC CHAN HE D BALL CAUS OF 18 (21					
	THE WAR THE CH WITT OUT KINK ST	DEU FR	COMPUTED	104.7	H 6.0	1.5
025	ALARM HE UU468 CV RCLD OVERPUR SI	DEO FR	CONFUIED	104.1	H 4.0	r.c
	ALANA III TUAZUA AL KELD OVEKTENT L	1 1 SF		150.1	H 150.0	DEGI
0250	ALAKH II 1047DA AI KELD OVENTEMP DISCH	HOR P		2145.0	L 2200.0	P511
0230	TOUTH TO STOP VILL DEL PART ME THE C 28					
	DELTA HEUX PROGRAM IN LOW POWER CUTOFF MODE					
0228	103930 18 ELOP VIV C CL PART RE 16 C 30					
	103910 TO STOP VIVA CL PART RE TR C 37					
	103910 18 STOP VIV A EL PART RE					
	TOGGED SIN GEN C LO L & IN I LAUS AE IK C 226					
	LOSZED SIN GEN B LO LO L CAUS RE IR C 257					
	TOCHBO THE SSUKIZER TO F LAUS FE THE E 266		COMPULED	104 0	H 4.0	PE
025	ALAKE HI UU428 CU KELE OVERTAK ST	DEO LK	CONFUIED			
	TOTORD SIN PEN W TO TO F THIS ME					
	LOSEED SIN CEN DIGIGIL LAUS ME 16 C 272			152.2	1 152.0	DEUI
025	INCH HI TUATUA AT KELD OVERTENT L) 1 1 br		132.2		
	LUCALD SIN GEN C LO LO L CAUS RE IN C 2/8					
	TUAZID SIN LIN B LO L & FU I LAUS ME IN C 2/8					
	TOLOTO SIN GEN A LO L A IN F CAUS RE IR C 30/					
	TUSATO SIR GEN D LO L & SW I LAUS ME IN E 307					
0.100	3/18/1984 DC COOK UNIT 2					
0300	THESID SIR LIN D LO L & IN F LAUS ME HI IK C 317					
	TUGATO SIN DEN O LO L & IN F CAUS RE IR C 323					
	104210 SIR GIN B IO L & IN I LAUS KE NI IK C 640					
	ALAKE HI TUASUA AL KILB OVEKTERP (1 1 1		150.4	H 150.0	DEOL
030	TOGGED SIN GEN D LO L & FW F CAUS RE HI IN C 479					
	TUSSID SIN LEN E LO L & IN I CAUS NE NI IN E 746					
	TOTAL SECTION AS A P CAUS RE NI IN (108)					
	THER HI TUAJUA AL KELB OVERTENT L	11 1 60		152.0	1 152.0	DEGI
070	TOURS OF LINE ILE OLS AZ BER OF C 1474					
	TOURS OF CHAPT ON THE SEE OF SEE OF C 1459					
	INTUIN ON THE LIE OF ME BEN OL C 1440					
0301	END SEQUENCE OF EVENIS RECORD					
6361	SERUTHEE OF EVENIS KELOKD. TERST EVENT AT H 2 859 631					
6301	103790 IS TRIP CAUSE RE MI I.					
	TOTALD SIN BLE C LO L & IN F LAUS AL IK					
	104410 SIN GEN C LO L & FW F CAUS RE NI IR & 2250					
630.5	IND SIMULALE OF EVENTS ALCOAD					
6101	DIGI ANALOG INING-DIVICE 2 SIGFFED			-		
	DIDI ARMIND INTRO DIVILL & SIDIFE					

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0303 ANALOG THEND-DEVICE 2 STOFFED

VI. POST TRIP REVIEW PROGRAM.

The Post Trip Review program periodically records a number of pre-selected inputs. These inputs are stored on a disc in a circular buffer, with newer sets of data replacing the older sets. When a trip occurs, either automatically (Post Trip) or manually (Test Trip), the pre trip data being entered into the circular buffer are frozen and the data are thereafter stored in a post trip buffer. When this buffer is filled, both sets of data (pre and post) are printed out on the typewriter.

The parameters monitored are analog in nature. At the present time, they include selected RPI indication (for unit 2 only), steam generator feed water flow and steam flow, steam generator narrow range and wide range level, pressurizer level, pressurizer livel setpoint, source range detector output,

intermediate range detector output, power range detector output, first stage turbine pressure, steam generator pressure, pressurizer pressure, containment pressure, unit gross electrical output, Taverage, delta T power, overtemperature delta T setpoint, overpower delta T setpoint, wide range cold leg temperatures, pressurizer steam temperatures, T-reference, auctioneered delta T, and auctioneered Tavg. These parameters remain as selected by the computor vendor, Westinghouse Electric Corporation, except for certain RPI indications on Unit 2. The RPI indications were substituted for the four channels of total power range NIS power in order to obtain data on the anomalous response of RPI H-8 upon trip. The individual power range upper

and lower detector outputs remain in the Post Trip Review output for both units to monitor NIS power range indications.

Eight of the parameters in the previous paragraph are sampled at 2 second intervals 6 seconds before and after the trip. These are total NIS power range power on unit 1 and RPI indications on unit 2, turbine first stage pressure, unit gross electrical output, and auctioneered Taverage. All parameters are sampled at 8 second intervals for 2 minutes before and 3 minutes after the trip. These sampling times remain as set by the computor vendor.

The Post Trip Print program first outputs a start message on the appropriate typewriter. It then outputs a line of headings for the values which will be printed in columnar form. The headings consist of the six-character name of the point. The values are printed below their names starting with the oldest set of data on the first line, the next oldest on the next line and so on until the most recent pre-trip data are printed. Included in each row of data is a column indicating the time. When all the pre-trip data for this set of points are printed, the message POST TRIP DATA - TRIP TIME XXXX is printed. All the post-trip data for these points are printed in the same format as described above.

After all the post-trip data for these points are finished, the program starts over with another set of data in the same format: 6 character names, pre-trip values, trip message, and post-trip values. When all the points have been printed, the program outputs a finished message, unblocks the collection

programs, and exits. A sample output is included at the end of this section.

```
POST TRIP REVIEW ADDRESSES
         Cont. Rod Bank B Group 1 Pos MO8 (Unit 2 Only)
C0010A
C0027A
          Cont. Rod Bank D Group 2 Pos Plo (Unit 2 Only)
          Cont. Rod Bank D Group 2 Pos H08 (Unit 2 Only)
C0029A
C0075A
          SD Rod Bank D Group 1 Pos F10
                                              (Unit 2 Only)
F0403A Stm Gen A Feed Wtr in 1 F
F0404A Stm Gen A Feed Wtr in 2 F
F0405A Stm Gen A Stm Out 1 F
F0406A Stm Gen A Stm Out 2 F
F0423A Stm Gen B Feed Wtr in 1 F
F0424A Stm Gen B Feed Wtr in 2 F
F0425A Stm Gen B Stm Out 1 F
FO426A Stm Gen B Stm Out 2 F
F0443A Stm Gen C Feed Wtr in 1 F
F0444A Stm Gen C Feed Wtr in 2 F
F0445A Stm Gen C Stm Out 1 F
F0446A Stm Gen C Stm Out 2 F
F0463A Stm Gen D Feed Wtr in 1 F
F0464A Stm Gen D Feed Wtr in 2 F
F0465A Stm Gen D Stm Out 1 F
F0466A Stm Gen D Stm Out 2 F
LO400A Stm Gen A Nar Rng 1 L
L0401A Stm Gen A Nar Rng 2 L
L0402A Stm Gen A Nar Rng 3 L
LO403A Stm Gen A Wide Rng L
LO420A Stm Gen B Nar Rng 1 L
LO421A Stm Gen B Nar Rng 2 L
L0422A Stm Gen B Nar Rng 3 L
L0423A Stm Gen B Wide Rng L
LO440A Stm Gen C Nar Rng 1 L
L0441A
         Stm Gen C Nar Rng 2 L
L0442A Stm Gen C Nar Rng 3 L
L0443A Stm Gen C Wide Rng L
LO460A Stm Gen D Nar Rng 1 L
L0461A
         Stm Gen D Nar Rng 2 L
L0462A Stm Gen D Nar Rng 3 L
L0463A Stm Gen D Wide Rng L
L0480A
          Pressurizer 1 L
L0481A
          Pressurizer 2 L
L0482A Pressurizer 3 L
LO483A Pressurizer Lvl Control S.P.
NOO31A Source Rng Detector 1 Log Q
N0032A Source Rng Detector 2 Log Q
N0035A Source Rng Detector 1 Log Q
N0036A
          Interm Rng Detector 2 Log Q
N0041A
          PWR Rng 1 Top Detector Q
N0042A
          PWR Rng 1 Bottom Detector O
N0043A
           PWR Rng 2 Top Detector Q
N0044A
          PWR Rng 2 Bottom Detector Q
N0045A
          PWR Rng 3 Top Detector Q
NOO46A PWR Rng 3 Bottom Detector Q
N0047A PWR Rng 4 Top Detector Q
N0048A PWR Rng 4 Bottom Detector Q
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POST TRIP REVIEW ADDRESSES
N0049A
          PWR Rng Channel 1 Q (Unit 1 Only)
N0050A
           PWR Rng Channel 2 Q (Unit 1 Only)
           PWR Rng Channel 3 Q (Unit 1 Only)
N0051A
           PWR Rng Channel 4 Q (Unit 1 Only)
N0052A
             Tb First Stage 1 P
P0398A
P0399A Tb First Stage 2 P
P0400A
           Stm Gen A Stm Out 1 P
PO401A Stm Gen A Stm Out 2 P
P0402A Stm Gen A Stm Out 3 P
P0420A Stm Gen B Stm Out 1 P
PO421A Stm Gen B Stm Out 2 P
P0422A
           Stm Gen B Stm Out 3 P
           Stm Gen C Stm Out 1 P
P0440A
P0441A Stm Gen C Stm Out 2 P
P0442A Stm Gen C Stm Out 3 P
PO460A Stm Gen D Stm Out 1 P
P0461A Stm Gen D Stm Out 2 P
P0462A Stm Gen D Stm Out 3 P
P0480A Pressurizer 1 P
PO481A Pressurizer 2 P
P0482A
           Pressurizer 3 P
P0483A Pressurizer 4 P
P1000A Containment 1 P
P1001A Containment 2 P
P1002A Containment 3 P
P1003A Containment 4 P
Q0340A Unit Generator Gross
T0400A RCL A 1 T-Avg.
T0403A
           RCL A 1 DT
T0406A
           RCL A 1 Cold T
T0407A RCL Overpwr DT 1 SP
T0410A RCL A Overtemperature AT Setpoint
TO420A RCL B 1 T-AVg.
T0423A RCL B 1 DT
T0426A RCL B Cold T
T0427A RCL B Overpwr DT 1 SP
T0430A RCL B Overtemp DT 1 SP
T0440A
           RCL C 1 T-Avg.
T0443A RCL C 1 DT
T0446A RCL C Cold T
TO447A RCL C Overpwr DT 1 SP
T0450A RCL C Overtemp DT 1 SP
T0460A RCL D 1 T-Avg.
T0463A RCL D 1 DT
T0466A
           RCL D Cold T
           RCL D Overpwr DT 1 SP
T0467A
T0470A RCL D Overpwr DT 1 SP
T0481A Pressurizer Stm T
TO496A RC T-Ref.
T0497A RCL Auct. DT
T0499A RCL Auct. T-Avg.
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SAMPLE POST TRIP REVIEW OUTPUT

116.9 62.5 116.9 116.9 62.5 116.9 62.5 116.9 62.5 116.9 62.			116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9 116.9	0.0 116.9 1 0.0 1 0.0 1	0.0 0.0 116.9 116.9 10.0 116.9 10.0 10.0 10.0 10.0 10.0 10.0 10.0 10	0.0 116.9 1 0.0 1 0.0 116.9 1 0.0 1
		6.0 6.0 6.0 6.0 6.0 6.0 6.0 6.0 6.0 6.0		9.0 116.9 6.0 9.0 42.5 6.0 9.0 116.9 6.0 9.0 116	0.0 0.0 116.9 0.0 0.	7-34.4
		9.0 9.0 9.0 9.0 9.0 9.0 9.0 9.0 9.0 9.0	182.2 0.0 42.5 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.7 0.0 184.8 0.0 184.8 0.0 184.9 0.0 184.9 0.0 184.9 0.0 184.9 0.0 184.9 0.0 184.9 0.0 184.9 0.0 184.9 0.0 184.0 0.0	0.0 0.0	6.0 6.0 182.2 6.0 0.0 6.0 6.0 6.0 6.0 6.0 6.0 6.0 6.0	229.4 0.0 0.0 192.2 0.0 229.4 0.0 0.0 42.2 0.0 209.3 0.0 0.0 42.2 0.0 209.3 0.0 0.0 116.9 0.0 209.4 0.0 0.0 116.9 0.0 229.4 0.0 0.0 116.9 0.0 229.4 0.0 0.0 116.9 0.0 229.4 0.0 0.0 116.9 0.0 229.4 0.0 0.0 116.9 0.0 229.4 0.0 0.0 116.9 0.0 229.4 0.0 0.0 116.9 0.0 229.4 0.0 0.0 116.9 0.0 229.4 0.0 0.0 116.9 0.0 229.4 0.0 0.0 116.9 0.0 229.6 0.0 0.0 116.9 0.0 229.6 0.0 0.0 116.9 0.0
		9.0 9.0 9.0 9.0 9.0 9.0 9.0 9.0 9.0 9.0	114.7 1 0.0 114.7	6.0 42.5 1 0.0 6.0 116.7 1 0.0	6.4 6.4 6.4 6.4 6.4 6.4 6.4 6.4 6.4 6.4	207.4 0.0 0.0 116.7 0.0 207.5
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Section 100, 100, 100, 100, 100, 100, 100, 10	March Marc			17.5	2.7.2		27.3	17.6			17.3		17.7	17.5	17.4	17.4	17.8	17.8		17.9				18.3	18.5	18.7	18.9	19.0	19.0	19.2	14.1	16.31	17.1	14.3	16.1	19.4	14.5	19.2	16.1	0.41	104204	91.16	6.0	50.9	# · · · · ·	4.0°	* 17.	. O. A.	511.4	20.00	34.5	54.4	343.4
Column	Column																			2.4.5			20.1	20.1	20.2	20.3	24.5	20.5	20.4	20. 3	20.00	20.9	20.9	20.9	20.9	70.9	20.7	20.7	20.7	30.4	MEUTO !	45.5	4	43.4	60.0	45.2	1 59	1	45.2	65.7	65.3	4:.1	45.3
March Marc	March Marc			17.6	17.4	17.5	17.5	17.3	17.5	17.5	17.5	17.5	17.4	17.4	17.8	17.9	17.9	18.0																																			
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Sec. 2	Sec. 2																		129.3	129.1	179.4	129.3	174.4	2.47.5	130 3	124.2	129.1	129.0	127.0	176.9	129.6	179.0	179.0	179.11	124.1	138.9	174.0	177.2	179.5	104444	544.2	543.8	544.2	8.63.8	343.8	543.4	341.8	8.5.5	373.0	2.7.2	7 175	541.8	9 17.
Section Sect	Section Sect		107.7	107.4	4./61	10/.4		107.4	107.3	107.5	*****		102 .	107.4	107	107.4	167.6																																				
Section	Section																																																				
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Sept. Sept	Sept. Sept														_		_	* ***	131 1	127.8	127.7	127.7	127.1	137.7	137.7	177.4	177.7	1717	177.3	177.4	127.3	1:7:1	177.2	177.4	177.4	177.5	137.1	8.77	1.7.4	3485A				23.5	23.0	9.14	24.0	24.5	24.5	24.5	0.4	5.5	0.47
March Marc	March Marc	107.4	107.4	107.4	167.3	187.4	167.2	147.3	167.3	167.1	107.2	147.2	107.2	107.3	107.3	107.4	111.04.3	107	147.4	167.4	1117.4	167.4	107.4	107.4	1117.4	107.4	107.4	1417	107.2	107.1	107.3	147.2	107.2	107.4	107 4	107.1	107.4	107.4	107.3	0487A	7.1.1.	9 587	935.9	215.9	255.4	234.4	7.36.9	234.9	7.36.9	2 16.9	84.9	277.5	137.7
Sec.	Sec.	A .	-3.	C+		.2	7.	3	.7.								ir line		-2.5	1.1.	2.5	. 2.4	5.4					. 2.5	.7.5	-2.5	-2.5	-2.5	.7.				-2.4	.7.3		91450	9.56	223.0	232.5	221.0	200	223.5	0.577	0.4.6	174.8	5.4.3			
		244.2	548.3	.44.		.48.2	.48.1	2.418.2	2.4B.7		7.85.7	2.00.				* * 87.	Unia . Is	3.48.4	7.68.4	548.5		2.4H.A	9			7.83	48.	1.69.7	2.48.7	1.83.	3.48.8	H . H .	# H	. 46. 8		1.85	3.418.4		**************************************	1187.7	2:31.7	1111.1	201.2	1.11.1	1.83.2	2.81.	7.38.7	1 14.6	7.88.	1.16 7 2	2 10 2	.40.7	C 071
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200	73.4	2560.2	2775.0	27.31.7	2774.0	2.48.5	345.1	4.03	* 177						
	7 16	UAIR - IN	WILL AND	015847						5.7600	***	4	47.3		
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	23.5	2746.7	2276.0	27.18 3	0.4555	537.1	345.1	541.7	541.8	5858.2	45.1	* 57			
	11.5	. ~	22.24. 0	3356	2000	2.48.	****	542.2	544.7	2.804.2	45.7				
	21.5		3353 6	2.00	4.629.9	219.9	245.1	542.2	544.2	5914.2	45.5	44.0	****		
	24.5		23.34 5	3210 3	1111.4	5.44.9	6.030	543.0	545.1	2.422.2	4.8	* **			
	24.5	2.41.2	2236.0	27.18.7	3554 0	4.77.	245.3	247.4	5.446.2	59 10.2	45.7	46.3	*		
	****	_	11.4.4	27.18.7	2124 4			4.74	2.44.4	5438.2	45.9	4.6.4	47.8		
	24.5		2005.5	27.38.7	2226.5			342.2	344.3	5946.3	45.7	44.2	47.4		
	11.5		0.3555	2218.7	2226.9	540.4		247.7	244.2	5954.2	4:.9	46.4	47.8		
	20.00		2026.6	2738.2	2776.9				344.6	2.3	45.7	46.5	47.9		
	11.5		2115.B	2738.2	2224.0	0 000	*****	9.745	244.4	10.2	45.9	46.3	1.87		
	23.4		2554.5	2733.2	2772.5	. 14.	2.5.5.	9.200	544.2	18.2	45.9	44.5	47.9		
	. 1.	*	1225.0	210.1	2125.5	5 22 4	****	2.1.1	143.8	26.3	45.1	46.3	47.9		
	23.55		2225.0	27.36.9	2.75.0	. 44.		2000	344.2	34.2	44.1	47.0	48.5		
	11.4	~	2211.5	2234.4	2724.5	5 14 5	2.575	****	244.7	47.5	44.6	4.7.4	48.7		
	28.50		2223.4	2255.9	25.24 %	* 10 %		341.7	341.8	10.3	47.1	41.7	1.69		
	31.5	2	2221.0	2235.9	2724.0	2.19.1	2.000	2.7.7	244.2	58.2	46.5	47.4	48.7		
	23.5	~	2777.5	2735.4	2273.5			245.2	244.2	1116.2	44.8	47.4	48.9		
	11.5	~	2332.5	2256.9	2771.0	2.00.0		247.7	.43.8	114.7	47.7	47.8	44.3		
	***		0.2225	27.54.9	2223.0	. 44.		2000	542.5	133.2	47.0	47.8	49.2		
	1.1.4	~	2222.4	2214.9	2223.0	2.89.5	* * * * *	2000	E	130.2	47.7	47.4	44.3		
	31.4		975572	2734.9	2000	4 35 5		*****	2.44.5	1 18.2	47.3	49.0	49.5		
	31.4	_	2131.5	2234.4	2222.0	5.14.5	* 575	2,77.6		344.2	47.5	48.2	4.44		
	1114.14		104238	. Ditting	16441A	1044.70	104432	9.555.0	346.6	154.3	47.8	48.3	4.9.9		
	0		47.4	46.9	46.2	17.7	W W	the same	MISSIA	10467A	1 11443A	11818	£ 1000A		
			4.14	46.11	. 1.99	41.3	6:3	* **	***	45.7	66.3	5642.3	9.0.0		
			4.74	1.49	179	47.4	45.9	45.9	4.11		1.44	\$4:0.4	D. 1660		
		*	61.3	44.7	\$.33	1.11	4.5.4		7 77		2.00	2424.3	0.408		
		***	3.74	44.0	44.2	47.3	83.8	45.7	***	***		1.00.4	900.0		
	* 15		1.1.	4	44.1	47.1	6:.3	4:55	44.8		44.0		0.000		
	* 15		47.4		4.5.4	47.0	8	45.7	44.5	45.4	44.1	\$7.00 4	0.000		
	21.5	. 10	47.7		1.00	46.5	45.7	4:	14.3	45.3	44.1		-0 Pure		
	21.5	21.6	47.1			44.14	47.4	4.5.4	44.2	45.1	46.1	5744.4	0.00		
	21.5	31.0	67.1	45.2	* 55	30.00	4	****	44.7	44.9	94.6	\$754.4	0.000		
	31.4	50.9	67.1	45.1	4	***		***	1.11	45.1	64.0	58112.4	-0.008		
	31.4	51.0	41.2	45.1	65.3	***			43.9	6.19	4	5816.5	-0.00R		
	\$1.4	56.9	61.7	45.1	45.3	44.4			2.15	62.0	66.0	5818.4	0.000		
	***	51.0	41.2	12.1	15.1	4.6.4	44		43.7		44.0	2474.3	B. DUR		
	3.1.4	6.0.	17.19	45.0	45.2	66.3				44.4	44.13	2834.4	0.018		
	Post Inti o	AIA - IRIS	(IN. 01	2847					41.0	44.8	42.4	2047.5	0.000	0.084	0.000
	2.1.5	¥.0.4	6/.1	44.8	4:.0	1.95	65.50	45.1	44.6	4 11	*				
	7 17		***	44.9	42.4	44.2	4.1.4	45.1	44.0	0 11	45.8	20.00	900	0.070	-0.03
	4.1.6	21.00		44.4	1.04	***	2	45.3	44.7	44.9	1	. vib. 4	Birth o	0.070	Birli tr
	21.7	* 15	47. 3		2.5	4.6.4	4:.7	45.2	44.0	8.44	45.9	3914.	0 000	400.00	Bull.
	8.1.	31.4	47.3	45. 3	****	46.4	4	45.4	44.7	43.0	0.44	\$422.4	0 001	0 000	Mitte to
	27.8	27.45	47.4	6. 2			4	42.4	44.2	43.1	6.6.0	39 10.4	0.071	E 484	o dan
	3.2.0	31.5	47.4	1 19		***		4	44.5	45.3	45.4	3.914.	0.068	0 084	to defect
	27.1	31.7	61.7	45.5			2	43.7	44.5	45.3	9.64	3944.5	0.0414	0.0114	-11 11413
	52.40	51.6	87.4	65.5	45.7	2 4			4.4	45.4	1.11	3.954.4	0.073	0.68A	- fl. fatale
	0.00	3.1.6	47.9	45.7	6	47.0			64.3	43.4	64.2	2.5	600.0	0.046	11.190
		21.7	86.11	45.5	45.7	44.11	66.1	6.53		***	4.6.7	10.4	D. 00.0	D. DRA	0.000
		4.1.	4.19	45.5	45.4	1.94		4:.8		43.3		19.4	0.000	0.1186	B1+11- f1-
							*			****	1.10	1.07	0.000	D.047	-

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52.6	53.4	6.1.		24.3	16036	0.0	0.0	2 4	0.0	6.0	8.8	0.0	4.0	9.0	11.11	0.0	6.0	I Inir	6.6	0.0	0.9	0.0	9.0	0.0	9.0	9.0	0.0	0.0	- 4	0.0	9.0	2.3	0.0	6.6	11.11	9.0	11111
333	33.		31.6	4.8.	11 M F W	2,642.5	5450.5	34. H. G	5/16.5	4.77.6	3,7 50.6	:746.3	2.7.4.3	5810.7	.HIH. 5	454.		UAIA - I	2.620.7	. H. H. S	2914.5	9	. W 10. 6	.946.	34:45	4.4	18.5	34.4	500	5.0.7	4.4.	146.5	17.6	1 50.5	114.5	1.6.6	PENIN !
4 4 8	4.8		4 4	48.	104814	457.3	454.	427.	434.	6:6.4	8.95.8	656.	4:4.4	6.6.8	676.8	65.6.4	4.4.	RIF LINE	4:4.8	4.4.4	457.9	6:7:3	456.8	677.3	657.1	A.6.8	434.4	4.4.4	4.4.4	454.4	6:6.8	456.4	676.8	4.959	454.4	457.1	MISIMP
65.3					-	E 1/2	. 5455.	5742.	3/18	\$777.1	8.34.8	5750.8	\$7:4.9	1.6116.1	5.823.0	.630.9	2419.											31.0		54.8	107.9	110.7	1.97	134.8	141.1	158.7	
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***	:::	4.6.5	44.9	6.0	47.4																																
9.00	55.5	45.7	45.8	44.7	44.3																																
45.9	200	46.0	46.7	46.3	47.0																																
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-0.673	-0.673	-0.00%	0.073	0.60%	0.4146																																

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PROTECTION AND SAFEGUARDS SETPOINTS

TRIP DESCRIPTION	UNIT 1 SETPOINTS		UNIT 2 SETPOINTS	
	Actual	Tech. Spec.	Actual	Tech. Spec.
1) Manual	N/A	N/A	N/A	N/A
2) Pwr. Range Neutron Flux				
a) Low b) High	25% 109%	≦25% ≦109%	25% 109%	≦25% ≦109%
3) Pwr. Range Flux Rate a) Positive b) Negative	5% In 2 sec 5% In 2 sec	≤5% In ≥ 2 sec ≤5% In ≥ 2 sec	5% In 2 sec 5% In 2 sec	≦5% In ≧ 2 sec ≦5% In ≧ 2 sec
4) Intermediate Range Neutron Flux	25% (Current Equival.) (9.6 x 10-5amps)	≦25% (Current Equival.)	25% (Current Equival.) (8.1 x 10-5amps)	≤25% (Current Equival.)
5) Source Range Neutron Flux	9 x 10 ⁴ cps	≦10 ⁵ cps	9 x 10 ⁴ cps	≦10 ⁵ cps
6) <u>ΟΤΔΤ</u>	As Per Tech Spec		As Per Tech Spec	
7) <u>OPAT</u>	As Per Tech Spec		As Per Tech Spec	
8) Low PZR. Press	1872 psig	≧1865 psig	1966 psig	≧1950 psig
9) High PZR Press	2378 psig	≦2385 psig	2378 psig	≦2385 psig
10)High PZR Lv1.	91%	≦92%	91%	≦92%
11)Loss of Flow	90%	≧90%	93%	≧90°2

Page 1 of 2 Rev. 1

PROTECTION AND SAFEGUARDS SETPOINTS

TRIP DESCRIPTION	UNIT I SETPOINTS		UNIT 2 SETPOINTS	
DECORIT 110H	Actual	Tech. Spec.	Actual	Tech. Spec.
12) S/G Wtr. Lvl.Low-Low	17%(N.R.)	≧17%(N.R.)	21%(N.R.)	≧21%(N.R.)
13) Stm./Feed Flow Mismatch W/Low S/G Wtr Lvl	0.6 x 10 ⁶ pph Coincident with 26% (N.R.)	≦0.71 x 10 ⁶ pph Coincident with ≧25% (N.R.)	0.6 x 10 ⁶ pph Coincident with 26% (N.R.)	≤1.47 x 10 ⁶ pph Coincident with ≥25% (N.R.)
14) RCP Undervoltage	3150 Volts	≧2750 Volts	3150 Volts	≧2905 Volts
15) RCP Underfrequency	58.2 HZ	≥57.5 HZ	58.2 HZ	≧57.5 IIZ
16) RCP Bkr. Position	1/4 Open Bkrs Above P- 2/4 Open Bkrs Between P-7 & P-8	8 NA	1/4 Open Bkrs. Above P-8, 2/4 Open Bkrs. between P-7 & P-8	NA
17) Turbine Trip a) Low Sys. Press. b) Stop Vlv. Position	800 psig 1% open	≧800 psig ≧1% open	62 psig 1% open	≥58 psig ≥1% open
18) Safety Injection a) Manual b) High Containment Press c) Low PZR Press. d) High Stm. Line	NA 1.1 psig 1837 psig 100 psid	NA ≤1.1 psig ≥1815 psig ≤100 psid	NA 1.1 psig 1908 psig 100 psid	NA ≦1.1 psig ≧1900 psig ≦100 psid
Diff. Press. e) High Stm. Line Flow Coincident with	1.42 x 10 ⁶ pph for 0+20% Pwr. Ramped to 3.88 x 10 ⁶ pph at 100%	≦1.42 x 10 ⁶ pph for 0→20% Pwr. Ramped to 3.88 x 10 ⁶ pph at 100%	NA	NA
Lo-Lo Tavg or Low Stm. Line Press.	541°F 600 psig	≥541°F ≥600 psig	NA 600 psig	NA ≧600 psig

Page 2 of 2 Rev. 1 ATTACHMENT 2 TO AEP:NRC:0838S

DONALD C. COOK NUCLEAR PLANT

TABLE OF PARAMETERS IDENTIFIED AS

DESIRABLE FOR POST-TRIP REVIEW

Attachment 2 to AEP:NRC:0838S

Page 1

FROM SAIC REPORT TABLE Desirable PWR Parameters for Post-Trip Review AEP Response for D. C. Cook Nuclear Plant on Parameters listed in the TER as Not Recorded

SOE RECORDER	TIME HISTORY RECORDER '	PARAMETER/SIGNAL	SOURCE
1. X		Containment Isolation	 Emergency Operating Procedure E-O requires operator action to verify Containment isolation valves are closed and manually close any containment isolation valves which are open. We believe these actions provide the necessary verification of closed valves.
2. X		Control Rod Position	 Emergency Operating Procedure E-O requires immediate operator action to verify all rods are inserted. Operators are required to take specific actions when automatic control rod insertion does not occur, and the cause would be evaluated during a post-trip review.
3. X		Neutron Flux Power	3. Hathaway Operations Sequence Monitor 3.1 - Point 025 Reactor Trip NIS - Source Range Level Trip 3.2 - Point 026 Reactor Trip NIS - Intermediate Range Hi Level Trip 3.3 - Point 027 Reactor Trip NIS - Power Range Overpower Trip Low Range 3.4 - Point 028 Reactor Trip NIS - Power Range Overpower Trip High Range 3.5 - Point 029 Reactor Trip NIS - Power Range Pos-Neg Rate Trip
4. X		Containment Pressure	4. Hathaway Operation Sequence Monitors 4.1 - Point 034 Reactor Trip - High Containment Pressure Safety Injection
5. (2) Parameter m either an S	ay be recorded by OE or time history	Containment Radiation	5. The Eberline radiation monitoring system prints the level when an alarm limit is exceeded and can provide 10 minute averages for the past 4 hours. The control terminal/printer is run from an uninterruptible power source
6.	х	Containment Sump Level	6. We have redundant containment sump level indicators. We do not have a recorder.

SOE TIME HISTORY RECORDER '	PARAMETER/SIGNAL	SOURCE
7. X (1) Trip Parameter	Primary System Pressure	7. Hathaway Operations Sequence Monitor 7.1 - Point 030 Reactor Trip - Pressurizer Safety Injection 7.2 - Point 031 Reactor Trip - Pressurizer Low Pressure 7.3 - Point 032 Reactor Trip - Pressurizer High Pressure
8. X	Primary System Temperature	8. Hathaway Operations Sequence Monitor 8.1 - Point 001 Reactor Trip - Overtemperature DT 8.2 - Point 002 Reactor Trip - Overpower DT
9. X (1) Trip Paramter	Pressurizer Level	9. Hathaway Operations Sequence Monitor 9.1 - Point 033 Reactor Trip - Pressurizer High Water Level
10. X (1) Trip Parameter	Reactor Coolant Pump Status	10. Hathaway Operations Sequence Monitor 10.1 - Point 048 Reactor Coolant Pump No. 1 Trip 10.2 - Point 049 Reactor Coolant Pump No. 2 Trip 10.3 - Point 050 Reactor Coolant Pump No. 3 Trip 10.4 - Point 051 Reactor Coolant Pump No. 4 Trip
11. X X (1) Trip Parameter	Primary System Flow	11. Hathaway Operations Sequence Monitor 11.1 - Point 003 Reactor Trip Undervoltage RCP Bus 11.2 - Point 004 Reactor Trip - Low Flow RCP/P7 Permissive 11.3 - Point 005 Reactor Trip - Low Flow RCP/P8 Permissive 11.4 - Point 007 Reacator Trip Underfrequency RCP Bus 11.5 - We have redundant Primary System Flow indicators and believe that information essential to a post-trip review could be provided from information recorded by the operators.
12. (3) Acceptable recorder options are: system flow on a SOE, system flow on a THR, or equipment status recorder in an SOE.	Safety Inj: Flow, Pump/Valves status	12. We have flow indicators on all Safety Injection Pumps and limit switch indicators on key BCCS valves. Emergency Operating Procedure E-O requires operator actions for validation which indicates the pumps are operating and the valves are properly positioned. Any anomalies are annotated for later use in a post-trip review.
13. X	MSIV Position	 Hathaway Operations Sequence Monitor 13.1 - Point 020 Reactor Trip Safety Injection steamline pressure/flow steamline isolation gives indication of initiation of signal. Limit switches are available to determine valve closure and are subject to operator verification.

20 X

PORV Position

SOE RECORDER	TIME HISTORY RECORDER *	PARAMETER/SIGNAL	SOURCE
14. X		Steam Generator Pressure	14. Hathaway Operations Sequence Monitor 14.1 - Point 020 Reactor Trip - Safety Injection Steam Line Pressure/Flow Steam Line Isolation
15. X (1) Trip	Parameter	Steam Generator Level	15. Hathaway Operations Sequence Monitor 15.1 - Point 016 Reactor Trip - Steam Generator Low/Low Water Level Loop 1 15.2 - Point 017 Reactor Trip - Steam Generator Low/Low Water Level Loop 2 15.3 - Point 018 Reactor Trip - Steam Generator Low/Low Water Level Loop 3 15.4 - Point 019 Reactor Trip - Steam Generator Low/Low Water Level Loop 4
16. X (1) Trip	Parameter	Feedwater Flow	16. Hathaway Operations Sequence Monitor 16.1 - Point 008 Reactor Trip - Low Feed Water Flow Loop 1 16.2 - Point 009 Reactor Trip - Low Feed Water Flow Loop 2 16.3 - Point 010 Reactor Trip - Low Feed Water Flow Loop 3 16.4 - Point 011 Reactor Trip - Low Feed Water Flow Loop 4
17. X (1) Trip	Parameter	Steam Flow	17. Hathaway Operations Sequence Monitor 17.1 - Point 020 Reactor Trip Safety Injection Steamline pressure/flow steamline isolation
are sys	eptable Recorder options : system flow on an SOE, tem flow on a THR, or ipment status recorder an SOE.	Auxiliary Feedwater System; Flow, Pump/ Valve Status	18. Flow indication is available on all auxiliary feedwater pumps. Emergency Operating Procedure E-O requires operator action to verify auxiliary feedwater system status. Any anomalies of the AFW system would be recorded in the post-trip review procedure and investigated as needed.
19. X		AC and DC System Status (Bus Voltage)	19. Hathaway Operating Sequence Monitor 19.1 - Point 003 Reacator Trip Undervoltage RCP Bus 19.2 - Point 007 Reactor Trip Underfrequency RCP Bus 19.3 - Point 046 Reactor Breaker Undervoltage Train "A" 19.4 - Point 047 Reactor Breaker Undervoltage Train "B" 19.5 - Point 060 Diesel Generator 2 AB Start 19.6 - Point 061 Diesel Generator 2 CD Start 19.7 - Point 156 CRP on Backup (Unit 2) Point 154 CRP on Backup (Unit 1)

and investigated as needed.

20. Limit switches are available on PORVs and block valves. Emergency Operating Procedure E-O requires operator action to verify the PORV position. Any anomalies of the pressure relief system would be recorded in the post-trip review procedure