NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Hope Creek Generating Station	0 5 0 0 0 3 5 4	1 OF 0 4

TITLE (4) PRIMARY CONTAINMENT ISOLATION SYSTEM ACTUATION DUE TO MOMENTARY LOSS OF POWER TO RADIATION MONITORING EQUIPMENT.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 1	2 3	8 7	8 7	- 0 0 5	- 0 0	0 2	1 9	8 7		0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	1 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
R.B. COWLES, LEAD ENGINEER, TECHNICAL STAFF	6 0 9 3 3 9 - 1 5 2 6 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A Primary Containment Isolation System (PCIS) actuation occurred when Reactor Building Exhaust (RBE) radiation monitoring channel "A" momentarily tripped at the same time that channel "C" was in a tripped condition. With channel "C" previously tripped, a "2 of 3 channels tripped" logic was satisfied when channel "A" tripped, actuating PCIS. After determining that the channel "A" trip was momentary and spurious, all associated systems and equipment were returned to normal service. Subsequent simulation by I&C Department determined that a momentary power loss to channel "A" caused the channel "A" trip. The root cause of the momentary power loss could not be determined, as such, a definite corrective action can not be identified. I&C Department is tracking this event for any repeat occurrences.

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1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Hope Creek Generating Station	0 5 0 0 0 3 5 4	8 7	— 0 0 5	— 0 0	0 2 OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor (BWR/4)
Primary Containment Isolation System (EIIS Designator: JM)
Process Radiation Monitoring (EIIS Designator: IL)
Radiation Monitoring System Channel "A" (EIIS Designator: CHA)

IDENTIFICATION OF OCCURRENCE

Primary Containment Isolation System (PCIS) Actuation Due to Momentary Loss of Power to Radiation Monitoring Equipment
Event Date: 01/23/87
Event Time: 1054
This LER was initiated by Incident Report No. 87-015.

CONDITIONS PRIOR TO OCCURRENCE

OPERATIONAL CONDITION 1 (Power Operation), Reactor Power 100%, unit load 1124 MWe.

DESCRIPTION OF OCCURRENCE

On January 23, 1987 at 1054, A Primary Containment Isolation (PCIS) Actuation occurred when Reactor Building Exhaust (RBE) Radiation Monitoring Channel "A" Momentarily tripped at the same time that channel "C" was in a tripped condition. After determining that the channel "A" trip was momentary and spurious, all associated systems and equipment were returned to normal service.

The following alarms and indication were observed during the course of this incident.

- RBE Radiation Alarm/Trouble Alarm
- FRVS Recirc Fan Trouble Alarm; FRVS Recirc Fan and "B" FRVS Vent Fan Start
- Reactor Building Ventilation Panel Alarm; Reactor Building Ventilation Isolation
- Service Water Supply Loop A&B Trouble; "B" Service Water Pump Start
- "A" SACS Loop Trouble; "B" SACS Pumps Start
- Instrument Gas System A&B Trouble; Instrument Gas System Isolation
- Unit Substation Feeder Breaker Trouble; Trip of Reactor Building Vent Fans

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Hope Creek Generating Station	0 5 0 0 0 3 5 4	8 7	— 0 0 5	— 0 0	0 3 OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF OCCURRENCE CONT'D

The following system/equipment actuations were observed during this incident:

- FRVS System Start
- Reactor Building Ventilation Isolation
- Service Water and SACS Auto-Start
- Instrument Gas System Isolation
- Torus Water Cleanup Isolation
- Drywell Floor and Equipment Drain System Isolation

APPARENT CAUSE OF OCCURRENCE

Equipment Malfunction - Momentary loss of power to radiation monitor channel "A". The root cause of the momentary power loss could not be determined.

ANALYSIS OF OCCURRENCE

On 1/19/87, the reactor building exhaust channel "C" radiation monitor failed a source check, causing it to be declared inoperable per technical specification 3.3.2, and was subsequently tripped. During channel "C" sensor calibration on Ø1/23/87 (channel "C" still tripped), a channel "A" trip was momentarily received, and the "2 of 3 channels tripped" logic was satisfied, actuating PCIS.

I&C department subsequently ran a simulation in an effort to determine exactly what caused the momentary channel "A" trip signal. The only simulation that repeated all previously listed system responses was a momentary loss of power to the Radiation Monitoring System RBE channel "A". No reason for the momentary loss of power could be determined, and such is being considered as an isolated occurrence.

No similar incidents of this nature have been reported. I&C department is monitoring the RMS panel to determine if a trend toward momentary power interruptions is occurring. An analysis of the safety impact of this event will be done should a trend be identified by I&C.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Hope Creek Generating Station	0 5 0 0 0 3 5 4	8 7	— 0 0 5	— 0 0	0 4 OF 0 4

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CORRECTIVE ACTIONS

Because the cause of the momentary loss of power to the Radiation Monitoring System Channel "A" can not be determined, I&C is monitoring the Radiation Monitoring System to determine if a trend toward momentary power interruptions is occurring.

Sincerely,

R. S. Salvesen
General Manager -
Hope Creek Operations

RBC:tlb
SORC Mtg. 87-023

PSEG

Public Service Electric and Gas Company P.O. Box L Hancocks Bridge, New Jersey 08038

Hope Creek Operations

February 18, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

HOPE CREEK GENERATING STATION
DOCKET NO. 50-354
UNIT NO. 1
LICENSEE EVENT REPORT 87-005-00

This Licensee Event Report is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(iv).

Sincerely,

R. S. Salvesen
General Manager -
Hope Creek Operations

RBC:tlb

Attachment
SORC Mtg. 87-023

C Distribution

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