

August 31, 1999
RC-99-0173

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Attention: Ms. K. R. Cotton

Gentlemen:

Subject: VIRGIL C. SUMMER NUCLEAR STATION (VCSNS)
DOCKET NO. 50/395
OPERATING LICENSE NO. NPF-12
RE: CLOSURE OF THE REVIEW OF THE RESPONSE TO
THE GENERIC LETTER 92-01, REVISION 1,
SUPPLEMENT 1, "REACTOR VESSEL STRUCTURAL
INTEGRITY"; - CORRECTIONS TO REACTOR VESSEL
INTEGRITY DATABASE (RVID), VERSION 2

Reference: 1) Karen Cotton, NRC, to SCE&G, dated July 22, 1999
2) Gary J. Taylor, SCE&G, to NRC, RC-98-0185, dated
October 9, 1998
3) Gary J. Taylor, SCE&G, to NRC, RC-95-0266, dated
November 7, 1995

South Carolina Electric and Gas Company (SCE&G) has reviewed RVID, Version 2, in accordance with the recommendation of Reference 1 and hereby requests that the information listed in the RVID, Version 2, be amended to reflect the data for VCSNS as marked in the attachment to this letter.

This information was provided previously within the report provided for the VCSNS Reactor Vessel Radiation Surveillance Program (Reference 2). The report identified changes due to recent surveillance capsule testing which revised the information provided by Reference 3 for the RVID. RVID, Version 2, needs to be amended to accurately reflect the current VCSNS reactor vessel information.

This information also supports a proposed Technical Specification Change Request to update the heat-up and cooldown curves submitted by SCE&G.

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PDR ADOCK 05000395
P PDR

NUCLEAR EXCELLENCE - A SUMMER TRADITION!

AB28/

Gary J. Taylor
Vice President
Nuclear Operations

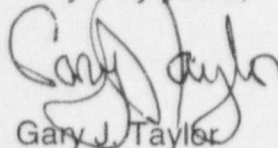
South Carolina Electric & Gas Co
Virgil C. Summer Nuclear Station
P. O. Box 88
Jenkinsville, South Carolina
29065

803.345.4344
803.345.5209
www.sceg.com

These statements and matters set forth herein are true and correct to the best of my knowledge, information, and belief.

Should you have questions, please call Mr. Jim Turkett at (803) 345-4047.

Very truly yours,


Gary J. Taylor

JT/GJT
Attachment

c: J. L. Skolds
W. F. Conway
R. R. Mahan (w/o Attachment)
R. J. White
L. A. Reyes

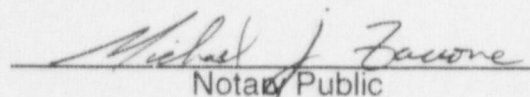
NRC Resident Inspector
J. B. Knotts, Jr.
RTS (LTR 920001, AVA 1001)
File (815.14)
DMS (RC-99-0173)

STATE OF SOUTH CAROLINA :
:
COUNTY OF FAIRFIELD :

TO WIT :

I hereby certify that on the 31ST day of August 1999, before me, the subscriber, a Notary Public of the State of South Carolina personally appeared Gary J. Taylor, being duly sworn, and states that he is Vice President, Nuclear Operations of the South Carolina Electric & Gas Company, a corporation of the State of South Carolina, that he provides the foregoing response for the purposes therein set forth, that the statements made are true and correct to the best of his knowledge, information, and belief, and that he was authorized to provide the response on behalf of said Corporation.

WITNESS my Hand and Notarial Seal


Notary Public

My Commission Expires

My Commission Expires July 13, 2005

Date

NRC – Reactor Vessel Integrity Database

Excerpted Pages

For

V. C. Summer Nuclear Station

PTS Summary Report

Surveillance Data Summary

Upper Shelf Energy Summary Report

**NRC - Reactor Vessel Integrity Database
PTS Summary Report**

Printed 08/10/1999 10:26.4
Page 1

Docket No: 50-395
EOL Date: 08/06/2022

SUMMER

Beltline Identification			Neutron Fluence		RTpis @EOL		Fluence		Chemistry Factor		Margin		Margin Method		Cu %		Ni %		P %		S %	
Type	Heat ID						RTndt(u)	RTndt(u) METHOD	RTndt(u)	Chem Factor	Chemistry Method	Margin	Margin Method	Cu %	Ni %	P %	S %					
PLATE	INTERMEDIATE SHELL		92.2	4.840 3.340	-20.0	PLANT SPECIFIC	78.1 78.1		58.00	TABLE	34.0	POSITION 1.1 (NO S DATA)	0.090	0.450	0.006	0.016						
	A9153-2																					
PLATE	INTERMEDIATE SHELL		95 95	4.840 3.840	30.0	PLANT SPECIFIC	44.0 47.7		34.00 35.4	SURVEILLANCE NON-RATIO	17.0	POSITION 2.1 (S DATA)	0.100	0.510	0.009	0.015						
	A9154-1																					
PLATE	LOWER SHELL		112.8	4.840 3.840	10.0	PLANT SPECIFIC	68.7 68.7		51.00	TABLE	34.0	POSITION 1.1 (NO S DATA)	0.080	0.410	0.005	0.014						
	C9923-1																					
PLATE	LOWER SHELL		112.8	4.840 3.840	10.0	PLANT SPECIFIC	68.7 68.7		51.00	TABLE	34.0	POSITION 1.1 (NO S DATA)	0.080	0.410	0.005	0.015						
	C9923-2																					
WELD	WELDS		33 33	4.840 3.840	-44.0	PLANT SPECIFIC	49.2 49.2		28.00 36.5	SURVEILLANCE NON-RATIO	28.0	POSITION 1.1 (S DATA)	0.050	0.910	0.013	0.012						
	4P4784																					

Plant References and Beltline Material Notes

Fluence, chemical composition, and RTndt(u) are from the June 30, 1992, letter from J.F. Skolds (SCE&G) to the USNRC Document Control Desk, subject: Response to GL 92-01, Revision 1, Reactor Vessel Structural Integrity. Actual datapoints used for determining the best-estimate chemistry value for the only beltline weld were documented in the licensee's response to GL 92-01, Revision 1, Supplement 1 dated November 7, 1995.

UUSE data are from Table A-1 of WCAP-10814.

There are six surveillance data points for the plate fabricated from heat number A9154-1. The staff considered the surveillance data credible because only one plate data point exceeded the credibility criterion by 1.6 degrees F.

The UUSE value for the beltline weld (heat number 4P4784) was revised from 93 ft-lb to 84 ft-lb according to the information in the licensee's response to GL 92-01, Supplement 1 dated November 7, 1995.

The response to GL 92-01, Revision 1, Supplement 1 dated November 7, 1995 indicates that the ratio procedure does not apply to the beltline weld because "the copper and nickel content of the surveillance weld does not differ from that of the vessel weld."

**NRC - Reactor Vessel Integrity Database
Surveillance Data Summary**

Printed 08/10/1999 10:27:1
Page 1

Docket No: 50-395
EOL Date: 08/06/2022

SUMMER

Type	Direction	ID	Capsule	Fluence Factor	Neutron Fluence	Fluence Factor	Group CF	Used in CF Calcs	Predicted ΔRTndt	Measured ΔRTndt	Predicted - Measured ΔRTndt	Credible RG1.99 Scatter	σ of Pred-Meas ΔRTndt	Unir USE	Meas USE	%Drop in USE	%Drop in USE Line Offset	Cu %	Ni %	P %	S %
PLATE	LONGITUDINAL	A9154-1	U, 3.11	0.65	0.65	0.88	35.4	YES	31.2	36.0	-4.8	YES	12.5	132.0	131.0	0.76	-0.07458	0.100	0.510	0.009	0.015
PLATE	TRANSVERSE	A9154-1	U, 3.11	0.65	0.65	0.88	35.4	YES	31.2	46.0	14.8	YES	12.5	75.0	73.0	0.00	-3.95401	0.100	0.510	0.009	0.015
PLATE	LONGITUDINAL	A9154-1	V, 3.11	1.54	1.54	1.12	35.4	YES	39.6	52.6	-13.0	YES	12.5	132.0	122.0	7.58	0.83986	0.100	0.510	0.009	0.015
PLATE	TRANSVERSE	A9154-1	V, 3.11	1.54	1.54	1.12	35.4	YES	39.6	52.6	-13.0	YES	12.5	75.0	76.0	0.00	-4.03057	0.100	0.510	0.009	0.015
PLATE	LONGITUDINAL	A9154-1	X, 3.72	2.54	2.54	1.25	35.4	YES	44.3	37.7	6.6	YES	12.5	132.0	125.0	5.30	0.63208	0.100	0.510	0.009	0.015
PLATE	TRANSVERSE	A9154-1	X, 3.72	2.54	2.54	1.25	35.4	YES	44.3	37.7	6.6	YES	12.5	75.0	73.0	2.67	0.33352	0.100	0.510	0.009	0.015
WELD	N/A	4P4784	U, 3.11	0.65	0.65	0.88	36.5	YES	32.1	28.0	4.1	YES	11.0	91.0	87.0	4.40	0.68901	0.050	0.910	0.013	0.012
WELD	N/A	4P4784	V, 3.11	1.54	1.54	1.12	36.5	YES	40.9	58.6	-17.7	YES	11.0	91.0	85.0	6.59	0.77954	0.050	0.910	0.013	0.012
WELD	N/A	4P4784	X, 3.72	2.54	2.54	1.25	36.5	YES	45.6	28.3	17.3	YES	11.0	91.0	85.0	6.59	0.72666	0.050	0.910	0.013	0.012
PLATE	LONGITUDINAL	A9154-1	W	4.66	4.66	1.39	35.4	YES	49.2	65.7	-16.5	YES	12.5	132.0	126	4.5		0.100	0.500	0.009	0.015
PLATE	TRANSVERSE	A9154-1	W	4.66	4.66	1.39	35.4	YES	49.2	52.8	-3.6	YES	12.5	75.0	74.0	1.3		0.100	0.500	0.009	0.015
WELD	N/A	4P4784	W	4.66	4.66	1.39	36.5	YES	50.7	54.4	-3.7	YES	11.0	91.0	87.0	4.4		0.050	0.910	0.013	0.012

NRC - Reactor Vessel Integrity Database
Upper Shelf Energy Summary Report
SUMMER

Docket No: 50-395
EOL Date: 08/06/2022

Beltline Identification		Material Type	USE @ EOL @ 1/4 T	1/4 T Neutron Fluence @ EOL	Unirradiated USE	Unirradiated USE Method	%Drop in USE @ EOL @ 1/4 T	%Drop in USE Method	Cu %
Type	Heat ID								
PLATE	INTERMEDIATE SHELL A8153-2	A 533B	81.07	2.40 2.41	107.00	DIRECT	23.39	POSITION 1.2 (NO S DATA)	0.080
PLATE	INTERMEDIATE SHELL A8154-1	A 533B	74.09	2.40 2.41	81.00	DIRECT	23.39 8.4	POSITION 2.2 (S DATA)	0.100
PLATE	LOWER SHELL C9923-1	A 533B	81.21	2.40 2.41	106.00	DIRECT	23.39	POSITION 1.2 (NO S DATA)	0.080
PLATE	LOWER SHELL C9923-2	A 533B	70.48	2.40 2.41	92.00	DIRECT	23.39	POSITION 1.2 (NO S DATA)	0.080
WELD	WELDS 4P4784	LINDE 124	77.76	2.40 2.41	84.00	DIRECT	7.3 7.3	POSITION 2.2 (S DATA)	0.050

Plant References and Beltline Material Notes

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