LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

OCTOBER 1986

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

IE24
1/1

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

October 1-31

October 1, 0001 Hours.	The Unit entered October with the Reactor Critical and on-line at 55.9% Power (615 MWe).
October 3, 2200 Hours.	Began Decreasing Power 25 MWe/hr. to 27.3% (300 MWe) due to drywell entry.
October 4, 1052 Hours.	Began Increasing Power 16 MWe/hr. to 80.9% (890 MWe).
October 6, 0000 Hours.	Began Decreasing Power to 53.6% (590 MWe) for Control Rod Manipulations.
October 6, 0600 Hours.	Began Increasing Power at 7.7 MWe/hr to 90% (990 MWe).
October 10, 0200 Hours.	Began Decreasing Power 50 MWe for Control Rod Manipulations.
October 10, 0245 Hours.	Began Increasing Power at 15 MWe/hr to 90% (990 MWe).
October 15, 1600 Hours.	Began Decreasing Power 50 MWe to 85.5% (940 MWe) due to Control Rod Manipulations.
October 15, 1710 Hours.	Began Increasing Power at 10 MWe/hr to 90% (990 MWe).
October 18, 0115 Hours.	Began Decreasing Power 120 MWe to 79.1% (870 MWe) for MSIV and TCV Surveillance Testing.
October 18, 0900 Hours.	Began Decreasing Power 260 MWe to 55.5% (610 MWe) due to maintenance needed on MSIV Limit Switch.
October 18, 1245 Hours.	Began Increasing Power at 10 MWe/hr to 90.1% (995 MWe).
October 24, 0550 Hours.	Began Decreasing Power 20 MWe to 75% (835 MWe) to swap FW 1B TDRFP with MDRFP.
October 24, 0600 Hours.	Began Increasing Power at 10 MWe/hr to 82.8% (911 MWe).
October 25, 0140 Hours.	Began Increasing Power at 7.7 MWe/hr to 96.5% (1062 MWe).
October 31, 2400 Hours.	Reactor and Generator on-line and holding at 96% (1060 MWe).

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

   Amendment 46 - Incorporate changes to reporting requirements for Iodine Spiking from a short term report to an item included in Annual Report, for Report Requirements on Primary Coolant Iodine Spikes.

2. Facility or procedure changes requiring NRC approval.

   There were no procedure changes requiring NRC approval during this reporting period.

   The following facility changes which required NRC Approval were made up to this reporting period.

MODIFICATION NUMBER	DESCRIPTION
1-1-82-263	Redundant Vent and Drain Valves and diverse and redundant scram instrumentation.
1-1-82-290	Redundant Fault protection in power circuits of electrical penetration circuits.

3. Tests and Experiments requiring NRC approval.

   There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

   The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L61873	DG 1A Compressor Start/Stop Pressure Switch 1PS-DG043A	Pressure Switch out of calibration.	The Compressor does not cycle.	Recalibrated Pressure Switch.
L61894	Post Loca Primary Containment O_2 Recorder 1AIR-CM048	Pen Bound up.	Red Pen for O_2 Recorder was sticking.	Cleared and Lubricated Recorder.
L61946	E-APRM 1C51-K605GS	APRM E HI set out of tolerance.	APRM E generates APRM Hi and Rod out block alarms when not bypassed.	Replaced and recalibrated trip unit.
L62026	Suppression Pool Temperature Recorder 1TR-CM037	Bad circuit card and connector.	Failure of LIS-CM-104	Replaced circuit card and connector.
L62340	DIV-II, Suppression Pool Temperature Recorder 1TR-CM038	Bad Amplifier Assembly	All points read approximately 30 degrees low	Replaced Amplifier Assembly.
L62561	1B21-F028D Outbd Limit Switch #4	Limit Switch was out of adjustment.	Upon re-opening, Limit Switch would not make (K3G would not re-energize).	Readjusted Limit Switch #4.
L62562	1B21-F028A Outbd. Limit Switch #4	Limit Switch was out of adjustment.	Upon slow closing, limit switch would not reopen (K3A does not de-energize).	Readjusted Limit Switch #4.
L62772	DIV-I Post Loca O_2 Monitor 1AIR-CM047	Found bad regulators, O_2 Cell, Vacuum Pump, and various leaks.	Division I read about 2-3 times less than DIV.II.	Replaced regulators R1, R2 and R4, O_2 cell, Vacuum Pump and Repair leaks at panel.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, October 1, through October 31, 1986. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
86-039-00	10/16/86	Residual Heat Removal Shutdown Cooling Suction High Flow Isolation Switch was lock wired Closed, instead of Open during LIS-RH-312 due to personnel error.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE November 10, 1986
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: October, 1986 GROSS HOURS IN REPORTING PERIOD: 745
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 1035
4. REASONS FOR RESTRICTION (IF ANY): Administrative

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	745	984.65	13023.65
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	1642.0
7.	HOURS GENERATOR ON LINE	745	928.62	12570.62
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	2077296	2325576	34539226
10.	GROSS ELEC. ENERGY GENERATED (MWH)	688665	761199	11260593
11.	NET ELEC. ENERGY GENERATED (MWH)	662943	638846	10643303
12.	REACTOR SERVICE FACTOR	100%	13.5%	52.4%
13.	REACTOR AVAILABILITY FACTOR	100%	13.5%	58.9%
14.	UNIT SERVICE FACTOR	100%	12.7%	50.6%
15.	UNIT AVAILABILITY FACTOR	100%	12.7%	50.6%
16.	UNIT CAPACITY FACTOR (USING MDC)	85.9%	8.5%	41.3%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	82.5%	8.1%	39.7%
18.	UNIT FORCED OUTAGE RATE	0.0%	0.0%	16.4%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
NONE.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
N/A.

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: November 10, 1986
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761
MONTH: October 1986

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	583	17.	956
2.	577	18.	753
3	570	19.	861
4.	452	20.	986
5.	790	21.	1036
6.	672	22.	1008
7.	842	23.	1016
8.	956	24.	911
9.	946	25.	964
10.	939	26.	1028
11.	961	27.	985
12.	965	28.	985
13.	965	29.	989
14.	963	30.	1024
15.	954	31.	1019
16.	964		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE November 10, 1986
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

REPORT MONTH OCTOBER, 1986

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
4.	10/18/86	S	0.0	H	5	MSIV and TCV Surveillance Testing

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

There were no Safety Relief Valves Operated for Unit One during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
0-204-86	0DΦ01P Fuel Oil Transfer Pp.	Replace Contactor at MCC 135Y-2, F3.
0-205-86	0 DG Maintenance Switch	Change Filters
0-210-86	0 DG Air Dryer "B" Drain Trap	Repair Drain Trap
1-1512-86	1B D/G	Lubrication and Filter Change
1-1513-86	1E22-D310 HPCS Sightglass	Clean Sightglass
1-1556-86	1E12-C003	Repair Leaks
1-1557-86	1E12-C003	Replace Contactor

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the radioactive waste treatment system during this reporting period.

5. Indications of Failed Fuel Elements

There were no indications of Failed Fuel Elements from January 1, 1986 through October 31, 1986.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

OCTOBER, 1986

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

DOCUMENT ID 0036r/0005r

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E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations
2. ECCS System Outages
3. Off-Site Dose Calculation Manual Changes
4. Major Changes to Radioactive Waste Treatment System
5. Indications of Failed Fuel Elements

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

October 1-31

October 1, 0001 Hours.	The Unit entered October with the reactor critical and on-line at 84.1% (925 MWe).
October 3, 0030 Hours.	Began decreasing power 50 MWe to 79.5% (875 MWe) due to control rod manipulations.
October 3, 0040 Hours.	Began increasing power at 15 MWe/hr to 84.1% (925 MWe).
October 9, 2330 Hours.	Began decreasing power at 150 MWe/hr. to 48.4% (532 MWe) due to control rod manipulations.
October 10, 0330 Hours.	Began increasing power at 7.5 MWe/hr. to 90% (990 MWe).
October 17, 0005 Hours.	Began decreasing power 50 MWe to 81.8% (900 MWe) due to control rod manipulations.
October 17, 0125 Hours.	Began increasing power at 12 MWe/hr. to 90.3% (995 MWe).
October 24, 0000 Hours.	Began decreasing power 50 MWe to 81.8% (900 MWe) due to control rod manipulation.
October 24, 0215 Hours.	Began increasing power at 10 MWe/hr. to 90% (990 MWe).
October 29, 2330 Hours.	Began decreasing power at 150 MWe/hr. to 51.8% (570 MWe) due to load dispatcher.
October 30, 0405 Hours.	Began increasing power at 11 MWe/hr. to 90% (990 MWe).
October 31, 2400 Hours.	Reactor and generator on-line and holding at 91% (1000 MWe).

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

Amendment 25 - Incorporate the replacement of 8 - 26 inch and 2 - 8 inch Vent and Purge Isolation Valves by Clow Corp. that meet all vent and purge requirements.

Amendment 26 - Incorporate the instruments for Suppression Pool Water Level and Water Temperature Monitoring Instrumentation at the Remote Shutdown Panel.

Amendment 27 - Incorporate the Modification of Automatic Depressurization System Logic required by License Condition 2.C.(18)(d)(i).

Amendment 28 - Incorporate changes to reporting requirements for Iodine Spiking from a short term report to an item included in Annual Report, for reporting requirements on Primary Coolant Iodine Spikes.

2. Facility or procedure changes requiring NRC approval.
There were no facility or procedure change requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.
There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.
The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L61246	RCIC Turb. Trip and Throttle Valve 2TTV.	Dirty Limit Switch Contactors.	During testing the 2TTV would not trip from the Control Room.	Cleaned Limit Switch Contactors.
L61247	IRM B 2C51-K601B	Loose connection at Voltage Regulator Card.	Sporadic operation at Erratic time intervals.	Adjusted loose connection.
L61977	RCIC Water Leg Pp. 2E51-C003	Worn Pump and Intervals.	Pump is extremely loud with excessive vibration.	Installed new pump and seal.
L62291	Ion Chamber Power Supply	Defective Power Supply.	Incomplete Surveillance LIS-NR-211	Replaced Power supply.
L62465	"B" VC Ammonia Detector OXY-VC165B	Bad optic cell board.	Detector will not reset.	Replaced optic cell board.
L62622	Reactor Pressure Recorder 2C34-R609	Incorrect Gain Setting on Recorder.	Erratic Indication.	Adjusted gain on Recorder.
L62683	DIV-III, 125 VDC Ground Detector	Shorted Ground Detector.	DIV-III has a 125 Volt Ground.	Replaced Ground Detector.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, October 1 through October 31, 1986. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence

There were no Licensee Event Reports for this unit during this reporting period.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE November 10, 1986
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: October, 1986 GROSS HOURS IN REPORTING PERIOD: 745
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 982
4. REASONS FOR RESTRICTION (IF ANY): Administrative

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	745	5150.05	10539.45
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	29.83	29.83
7.	HOURS GENERATOR ON LINE	745	5070.62	10306.92
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	2107104	14329560	29838112
10.	GROSS ELEC. ENERGY GENERATED (MWH)	696253	4734546	9844615
11.	NET ELEC. ENERGY GENERATED (MWH)	669984	4548344	9371559
12.	REACTOR SERVICE FACTOR	100%	70.6%	42.4%
13.	REACTOR AVAILABILITY FACTOR	100%	70.9%	42.5%
14.	UNIT SERVICE FACTOR	100%	69.5%	41.4%
15.	UNIT AVAILABILITY FACTOR	100%	69.5%	41.4%
16.	UNIT CAPACITY FACTOR (USING MDC)	86.8%	60.2%	50.7%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	83.4%	57.8%	48.7%
18.	UNIT FORCED OUTAGE RATE	0.0%	30.5%	29.0%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): A refueling outage is scheduled to begin January 2, 1987.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP NA

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: November 10, 1986
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761
MONTH: October, 1986

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	883	17.	946
2.	874	18.	942
3	873	19.	936
4.	889	20.	929
5.	882	21.	923
6.	878	22.	914
7.	874	23.	909
8.	870	24.	931
9.	870	25.	931
10.	691	26.	963
11.	852	27.	918
12.	940	28.	912
13.	951	29.	907
14.	947	30.	779
15.	942	31.	924
16.	936		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE November 10, 1986
COMPLETED BY James Peters
TELEPHONE (815)357-6761

REPORT MONTH OCTOBER, 1986

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
12.	10/10/86	S	0.0	H	5	Control Rod Manipulations

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT

There were no safety relief valves operated for Unit #2 during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-575-86	RCIC Steam Drain Pot	Troubleshoot Level Switch.
2-576-86	RCIC System	Set limits on trip and throttle valve.
2-577-86	RCIC Water Leg Pp.	Replace Pump.
2-578-86	2E51-F010	Replace 74 Relay.
2-580-86	RCIC Water Leg Pp.	Rebuild Motor.
2-581-86	Condenser Vacuum Pp. 2OG03P	Repair Vacuum Pp.
2-611-86	2A DG	Change Soak Back Filters.
2-614-86	2B DG	Change Soak Back Filters.

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the radioactive waste treatment system during this reporting period.

5. Indications of Failed Fuel Elements.

There is currently one suspected Fuel failure (one rod) on Unit #2 from the time period of January 1, 1986 to October 31, 1986. The reason for this suspected failure is that during January 20, 1986, the summation of 6 Nuclide activities at the Steam Jet Air Ejector increased approximately 450 uCi/sec. to 3000 uCi/sec. No unusual power maneuvers were occurring at the time, and we suspect an undetectable manufacturing flaw or just a "weak spot" in the Cladding that gave away.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

November 10, 1986

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period October 1, 1986 through October 31, 1986.

Very truly yours,

G. J. Diederich
Station Manager
LaSalle County Station

GJD/JPP/jdp

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
A. Bournia, NRR Project Manager
D. P. Galle, CECo
D. L. Farrar, CECo
INPO Records Center
L. J. Anastasia, PIP Coordinator SNED
M. A. Ortin, GE Resident
H. E. Bliss, Nuclear Fuel Services Manager
C. F. Dillon, Senior Financial Coordinator, LaSalle
Central File

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