

## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

September 2, 1999

Mr. Oliver D. Kingsley, President Nuclear Generation Group Commonwealth Edison Company Executive Towers West III 1400 Opus Place, Suite 500 Downers Grove, IL 60515

SUBJECT: RESPONSE TO THE REQUESTS FOR ADDITIONAL INFORMATION TO GENERIC LETTER 92-01, REVISION 1, SUPPLEMENT 1 - BRAIDWOOD STATION, UNITS 1 AND 2, AND BYRON STATION, UNIT 2 (TAC NOS. MA0527, MA0528 AND MA0530)

Dear Mr. Kingsley:

On May 19, 1995, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter 92-01, Revision 1, Supplement 1 (GL), "Reactor Vessel Structural Integrity," to holders of nuclear operating licenses. In issuing the GL the staff requested addressees to:

- identify, collect and report any new data pertinent to the analysis of structural integrity of the reactor pressure vessels (RPVs) at their nuclear plants, and
- assess the impact of that data on their RPV integrity analyses relative to the requirements of Sections 50.60 and 50.61 to Part 50 of Title 10 of the <u>Code of Federal Regulations</u> (10 CFR 50.60 and 10 CFR 50.61), and to the requirements of Appendices G and H to Part 50 of Title 10 of the <u>Code of Federal Regulations</u> (Appendices G and H to 10 CFR Part 50).

On August 17, 1995, Commonwealth Edison Company (ComEd) submitted its initial response to the GL and provided the requested information relative the structural integrity assessments for Braidwood Station, Units 1 and 2, and Byron Station, Unit 2. The staff evaluated ComEd's response to the GL and provided its conclusion relative to the response on August 29, 1996. However, since the time of the staff's closure letter, the Combustion Engineering (CE) Owners' Group and the Babcock and Wilcox (B&W) Owners' Group each have submitted additional data regarding the alloying chemistries of beltline welds in CE and B&W fabricated vessels. The additional alloying data were submitted in Topical Reports CE NPSD-1039, Revision 2, CE NPSD-1119, Revision 1, for CE fabricated RPV welds, and BAW-2325, Revision 1, for B&W fabricated RPV welds. In addition, Chicago Bridge and Iron (CB&I) boiling water reactor data were submitted in Topical Report BWRVIP-46. As a result of the efforts by CE and B&W, the staff determined that additional information was necessary relative to the structural integrity assessments for Braidwood, Units 1 and 2, and Byron 2. On June 1, 1998, the staff issued a request for additional information (RAI) in regard to the alloying chemistries of beltline welds, ComEd's assessment of surveillance data for the facilities, pressure-temperature (P-T) limits, and pressurized thermal shock (PTS) assessments for the plants. In general, with respect to the contents of the RAI, the staff requested that ComEd reassess the alloying chemistries

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for the beltline welds and RPV surveillance welds relative to the chemistries provided in the applicable topical report, and provide the impact of any changes to the best-estimate chemistries for the beltline RPV welds on the structural integrity assessments for the facilities relative to the requirements of 10 CFR 50.60, 10 CFR 50.61, a: Appendices G and H to 10 CFR Part 50, as applicable to the licensing bases.

ComEd provided its response to the staff's RAI for Braidwood and Byron 2 on September 3. 1998. As a result of the staff's review of the responses to GL 92-01, Revision 1; GL 92-01, Revision 1, Supplement 1; and the Supplement 1 RAI, the staff has revised the information in the Reactor Vessel Integrity Database (RVID) and is releasing it as RVID Version 2.

The new database diskettes are posted on the world-wide-web at a location which is linked to the NRC home page (http://www.nrc.gov/NRR/RVID/index.html). We recommend that ComEd review this information. If the staff does not receive comments by October 1, 1999, we will assume that the data entered into the RVID are acceptable for the plants. No additional information is necessary with regard to the structural integrity assessments. Future submittals on P-T limits, PTS, or upper shelf energy should reference the most current information.

This closes the staff's efforts in regard to TAC Nos. MA0527, MA0528 and MA0530. The staff appreciates your efforts in regard to this matter.

Sincerely,

Original Signed By

George F. Dick, Jr., Project Manager, Section 2 Project Directorate III **Division of Licensing Project Management** Office of Nuclear Reactor Regulation

Docket Nos. STN 50-455, STN 50-456. and STN 50-457

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