NRC For 19-83)	·m 366				LIC	ENSE	E EVE	NT RE	PORT	(LER)	U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO 3150-0104 EXPIRES 8/31/85					
FACILIT	Y NAME	1)	-								DOCKET NUMBER	(2)	FAI	25 12		
Hone Creek Commetica Chatian								0 5 0 0	-	1						
TITLE I			-								0 10 10 10	10/2/2/4	1,10,	1013		
Tn	adve	rter	+ T	solation	of Rea	ctor	· Wat	or C	leani	in Suct	o.m.					
EV	ENT DATE	1 (6)	T	LER NUMBER			PORT DAT		ream	THE RESERVE TO SHAREST PARTY OF THE PARTY OF	FACILITIES INVO	VED (A)				
MONTH	DAY	VEAR	YEAR	SEQUENTI.	AL REVEION		DAY	YEAR		FACILITY NA		DOCKET NUMBE	81/ 5)			
	-		+	NUMBER	NUMBER		-					0 15 10 10 10 1 1 1				
100	1							l +				0 12 0 1	101			
110	110	8 6	8	6 0 7	7-1010	111	110	8 6								
1	1110		_		TED PURBUANT	TO THE O	11 0		cen 8 //		of the following) (1	0 15 10 10	Tol			
OP M	ODE (9)	12		10.402(b)	T	20.406	THE OWNER WHEN PERSON NAMED IN	ENTS OF TO	V V		or the followings (1)					
POWER		+	20.406(a)(1)(i) 20.406(a)(1)(ii)			90.36(e)(1)			50.73(a)(2)(iv)		73.71(b)					
LEVEL 01 15										50.73(a)(2)(v)		73.71(e)				
(10)	10	1.10	-		-	80.36(c)			-	50.73(a)(2)(vii)			necity in At			
			-	10.408(a)(1)(NI)	-	80.73(e)				50.73(e)(2)(viii)		366A)				
			H	10.406(a)(1)(iv)	-	80.73(a)				60.73(a)(2/(viii)	(B)					
			11	10.408(a)(1)(v)		60.73ia				50.73(a)(2)(x)						
NAME						ICEMBEE	CONTACT	FOR THIS	LER (12)							
NAME												TELEPHONE NUI	HBER			
											AREA CODE					
R. (G. B	irle	У								61019	31 31 91-	1512	1318		
				COMPLET	E ONE LINE FOR	EACH CO	OMPONEN'	T FAILURE	DESCRIBE	D IN THIS REPO						
CAUSE	SYSTEM	сомя	ONENT	MANUFAC	REPORTABLE			CAUSE	SYSTEM	COMPONENT	MANUFAC	REPORTABLE	ales:			
				TURER	TO NPROS			CAUSE	STSTEM	COMPONENT	TURER	TO NPROS	300			
		-		1111					1 1	0.1.1	1 1 1 1					
											1					
	1	-1	1 1	1111					1	1-1-1	1 1 1 1					
				SUPPLEI	MENTAL REPORT	EXPECTE	D (14)				+	MONT	DAY	YEAR		
						T					EXPECT!	ED	1	1		
YE	5 //f yes, co	ompiete £	XPECTE	D SUBMISSION DA	TE)	×	NO NO				DATE (1		1 1	1		
ABSTRAC	T (Limit t	o 1400 w	MCM / A	approximately fifte	en sinale spece No											

This event consisted of an automatic isolation of the Reactor Water Cleanup (RWCU) System from an invalid high differential flow signal. Investigation by System Engineering revealed that the differential flow indication was in error apparently due to entrapped air in the system and sensing lines. Air entered the system during inadvertent partial draining of the inventory while the system was out of service. The Blowdown Line Isolation Valve was found to be partially open even though it indicated full closed which resulted in a drain path to the Main Condenser. Corrective action was to repair the valve and initiate evaluations with regard to filling and venting of the system following prolonged shutdown.

35 1/1

8611140174 861110 PDR ADOCK 05000354 NRC Form 366A

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO 3150-0104

FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)						PAGE (3)		
		· v	EAR		SEQUENTIAL NUMBER		REVISION NUMBER				
Hope Creek Generating Stati	on 0 5 0 0 0 3 5	4 8	16	-	01717	-	0 0	0/2	OF	0 3	

TEXT (If more space is required, use additional NRC Form 386A's) (17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor (BWR/4)
Reactor Water Cleanup System (EIIS Designator: CE)

IDENTIFICATION OF OCCURRENCE

Inadvertent Isolation of Reactor Water Cleanup System

Event Date: 10/10/86 Event Time: 0130

This LER was initiated by Incident Report No. 86-226.

CONDITIONS PRIOR TO OCCURRENCE

Plant in OPERATIONAL CONDITION 2 with a Reactor power of 0.5%. Startup in progress to allow continuence of power ascension program. Reactor Water Cleanup (RWCU) System in service in the blowdown mode to control Reactor water level.

DESCRIPTION OF OCCURRENCE

On October 10, 1986 at 0130 hours, while in the process of varying the blowdown flow rate of the RWCU System, both differential flow indicators increased to the fullscale value and initiated the system isolation time delay relays. The blowdown flowrate was reduced in an attempt to lower the indicated differential flow and prevent closure of the isolation valves. Before the flow could be reduced to below the trip setpoint, the time delay relays timed out and a system isolation occurred. Both isolation valves closed as designed which resulted in a trip of the running RWCU Recirculation Pump. Following an initial investigation of this incident, the RWCU System was returned to service without recurrence of the isolation.

ANALYSIS OF OCCURRENCE

The RWCU System utilizes differential flow instrumentation to compare flow rates within the various system pathways which provides a means of determining if gross leakage out of the system has occurred. Upon high differential flow being sensed, valves are closed to isolate the system from the Reactor. Investigation of this incident revealed that the high differential flow indication was in error and apparently the result of air entrapped in the system. The air was caused by partial draining of the system during an out of service period.

A review, as to how air became entrapped in the system revealed that the Blowdown Line Isolation Valve was partially open even though it indicated full closed in the Control Room. Prior to this event, the system had been isolated for a lengthy period of time to

NRC Form 366A

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

FACILITY NAME (1)	DOCKET NUMBER (2)		A NUMBER (6)	PAGE (3)					
		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER			
Hope Creek Generating Station	0 5 0 0 0 3 5 4	8 6	_	01717	-	010	013	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

ANALYSIS OF OCCURRENCE CONT'D

accommodate the Site Loss of Power (LOP) Test. During this isolated period, part of the inventory inadvertently drained to the Main Condenser through the partially open Blowdown Line Isolation Valve. Subsequent to the LOP Test, the system was returned to service without the piping being completely filled which caused the air in the system to enter the differential flow transmitters. Varying of the flow rate in the blowdown mode magnified the differential flow indication error caused by the entrapped air and resulted in the inadvertent isolation. The root cause of this incident was equipment malfunction in that the Blowdown Line Isolation Valve would not fully close due to improper limit switch adjustment. This LER is being submitted pursuant to 10CFR 50.73(a)(2)(iv).

CORRECTIVE ACTION

The Blowdown Line Isolation Valve limit switches were adjusted to allow full closure of the valve to preclude inadvertent draining. Additional corrective actions will be for the Operations Department to evaluate the need to fill and vent the RWCU System prior to placing in service following prolonged isolation.

Sincerely,

R. S. Salvesen

General Manager -

Hope Creek Operations

RGB:tlb

SORC Mtg. 86-292



Public Service Electric and Gas Company P.O. Box L. Hancocks Bridge, New Jersey 08038 Hope Creek Operations

November 10, 1986

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Dear Sir:

HOPE CREEK GENERATING STATION DOCKET NO. 50-354 UNIT NO. 1 LICENSEE EVENT REPORT 86-077-00

This Licensee Event Report is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(iv).

Sincerely yours,

R. S. Salvesen General Manager -

Hope Creek Operations

RGB:tlb

Attachment SORC Mtg. 86-292

C Distribution

JE22/1