

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Perry Nuclear Power Plant, Unit 1										DOCKET NUMBER (2) 05000440				PAGE (3) 1 OF 2		
TITLE (4) Inadvertant Bumping of IRM Cables Causes Reactor Protection System Actuation																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)				
01	27	87	87	004	0002	20	8	7				05000440				
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)														
4		20.402(b)				20.405(e)				<input checked="" type="checkbox"/> 80.73(a)(2)(iv)		73.71(b)				
POWER LEVEL (10) 0010		20.405(a)(1)(i)				80.36(a)(1)				80.73(a)(2)(v)		73.71(a)				
		20.405(a)(1)(ii)				80.36(a)(2)				80.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 308A)				
		20.405(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(vii)(A)						
		20.405(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(vii)(B)						
		20.405(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME Gregory A. Dunn, Compliance Engineer, ext 6484										TELEPHONE NUMBER 2146251913 171317						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On January 27, 1987, at 1033 an unexpected Reactor Protection System (RPS) actuation occurred due to upscale trips on Intermediate Range Neutron Monitors (IRM). No rod movement occurred since all rods were already fully inserted.

The cause of the event was personnel error. A plant helper performing decontamination work in the under vessel area inadvertently bumped the IRM cables causing spiking and upscale trips on IRM's B, C, D, E and G.

To prevent recurrence a sign will be posted at the entrance to the under vessel area warning that movement of instrument cabling may result in a reactor scram. Additionally, the individuals involved have been counseled regarding the need to exercise caution when working around instrumentation in the under vessel area.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Perry Nuclear Power Plant, Unit 1	0 5 0 0 0 4 4 0	8 7	— 0 0 4	— 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On January 27, 1987 at 1033, an unexpected Reactor Protection System (RPS)[JC] actuation occurred due to upscale trips on the Intermediate Range Neutron Monitors (IRM)[IG]. No rod movement occurred since all rods were already fully inserted. At the time of the event the plant was in Operational Condition 4 (Cold Shutdown), reactor vessel [RPV] pressure was approximately atmospheric and reactor coolant temperature approximately 155 degrees.

At the time of the event Health Physics technicians and plant helpers were in the process of decontaminating the under vessel area following removal of IRM H for maintenance. Drywell Hi Leakage Rate of Change annunciator actuated, due to runoff from the decontamination spray, coincident with the RPS actuation. Upscale alarms on IRM's B, C, D, E and G were noted by the Control Room operator. At 1040 the RPS actuation and the IRM trips were reset and decontamination work in the under vessel area was suspended.

The cause of this event was personnel error. A plant helper decontaminating the under vessel area inadvertently bumped the IRM cables resulting in spiking of the IRM channels and upscale trips. This was confirmed by placing the individual IRM's in BYPASS and wiggling the associated cable in the under vessel while observing the channel indication. Large signal spikes were noted as the cables were moved.

The IRMs are designed to provide neutron flux information during the reactor startup and heatup operations from the upper portion of the source range to the lower portion of the power range. The IRM system detects conditions that could lead to local fuel damage and provides trip signals which are used to prevent such damage. The system consists of eight identical neutron detection channels (A-H). If this event had occurred in Operational Condition 2 (Startup) an unnecessary reactor scram would have occurred. Access to the drywell is restricted during power operation due to high radiation concerns. Consequently, this event had no safety significance. No previous similar events were identified.

To prevent recurrence a sign will be posted at the entrance to the under vessel area warning that movement of instrument cabling may result in a reactor scram. Additionally, the individuals involved have been counseled regarding the need to exercise caution around instrumentation in the under vessel area.

Energy Industry Identification System Codes are identified in the text as [XX].



THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

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MURRAY R. EDELMAN
SR. VICE PRESIDENT
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February 19, 1987
PY-CEI/NRR-0594 L

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Perry Nuclear Power Plant
Docket No. 50-440
LER 87-004-00

Dear Sir:

Enclosed is Licensee Event Report 87-004-00 for the Perry
Nuclear Power Plant.

Very truly yours,

Murray R. Edelman
Senior Vice President
Nuclear Group

MRE:njc

Enclosure: LER 87-004-00

cc: Jay Silberg, Esq.
Paul Leech (2)
K. Connaughton

U.S. Nuclear Regulatory Commission
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Glen Ellyn, Illinois 60137

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