



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 81 AND 69 TO

FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2

DOCKET NOS. 50-338 AND 50-339

Introduction:

By letter dated April 12, 1985, as modified September 9, 1985, the Virginia Electric and Power Company (the licensee) proposed changes to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). Specifically, the proposed changes revise the NA-1&2 TS to revise the Limiting Condition for Operation (LCO) and Surveillance Requirements (SR) for the reactor trip bypass breakers, undervoltage trip logic and shunt trip logic. The proposed changes are in accordance with Items 10 and 13 of the NRC review of Item 4.3 of Generic Letter 83-28, "Required Actions Based on Generic Implications of Salem ATWS Events," and the NRC staff's issuance on July 24, 1984, of staff Safety Evaluation Report (SER) for Auto Shunt Trip Modifications. In addition, the proposed changes are in conformance with subsequent NRC staff guidance provided in Generic Letter 85-09, "Technical Specifications for Generic Letter 83-28, Item 4.3," for the LCO and SR for the reactor trip and bypass breakers, undervoltage and shunt trip logic and manual scram switches.

Discussion:

The NRC staff review, as noted above, approved the auto shunt trip modification and indicated a need for additional testing of the reactor trip breakers and its associated equipment. To meet these requirements, the licensee has submitted TS changes for the reactor trip bypass breakers, undervoltage trip logic and shunt trip logic. Revised LCO, and SR require reactor trip bypass breaker testing prior to the routine testing of the reactor trip breakers to further assure the operability of the bypass breaker during the testing of the main breaker. Secondly, LCO's and SR's are submitted which would require operability and surveillance of both the undervoltage and shunt trip logic features. Testing of the newly installed auto shunt trip feature would be accomplished in accordance with the Westinghouse Owners Group Technique. Thirdly, existing specifications for the manual trip feature adequately assure operability and surveillance of this feature including testing at a refueling interval. Testing of the manual feature would be accomplished in accordance with approved station procedure and by the method described in NRC's SE. Finally, administrative notes would be added to clarify the surveillance testing requirements for reactor trip breakers, reactor trip bypass breakers, and the manual scram switches.

Evaluation:

The licensee's proposed changes to the NA-1&2 TS are in conformance with the NRC Generic Letters 83-28, and 85-09, Item 4.3. The proposed changes provide further definition of reactor trip breaker operability and clarification of surveillance test requirements for the reactor trip and reactor trip breakers, and the manual scram switches. As such, the proposed changes add requirements for operability and testing of reactor trip breakers and associated components thereby increasing the margin of safety in relation to the present NA-1&2 TS requirements for the reactor protection system. On this basis, therefore, we find the changes to be acceptable.

Implementation Schedule

As stated above, to meet the guidance set forth in Generic Letter 85-09, Technical Specifications for Generic Letter 83-28, Item 4.3 has been submitted by the licensee for NA-1&2 and found to be acceptable. Therefore, Item 4.3 for NA-1&2 is complete and implementation is effective within 30 days of the date specified below.

Environmental Consideration

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: June 9, 1986

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