

ENCLOSURE 1

NOTICE OF VIOLATION

Entergy Operations, Inc.
River Bend Station

Docket No.: 50-458
License No.: NPF-47

During an NRC inspection conducted on July 20 through August 30, 1997, six violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," NUREG-1600, the violations are listed below:

- A. Technical Specification 5.4.1.a states, in part, that written procedures shall be implemented covering the applicable procedures recommended in Appendix A of Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operation)," Revision 2, February 1978.

Appendix A of Regulatory Guide 1.33, Section 1, recommends administrative procedures for equipment control (e.g., locking and tagging).

Administrative Procedure ADM-0027, "Protective Tagging," Revision 16, Section 7.10 requires, in part, that the operator designated to remove Danger-Hold tags shall be responsible to perform tag removal and component positioning in the sequence shown on the Clearance Removal Sheet. In addition, Section 7.10 requires the independent verifier to verify restoration of the clearance and initial each step.

Contrary to the above, on July 17, 1997, the individuals responsible to perform these functions failed to close and independently verify closed, Breaker EGS-ACB11 (Division I emergency diesel generator neutral breaker) as specified during the removal of Protective Tagout 97-0725.

This is a Severity Level IV violation (Supplement I) (50-458/97013-01)

- B. Technical Specification 5.4.1.a states, in part, that written procedures shall be implemented covering the applicable procedures recommended in Appendix A of Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operation)," Revision 2, February 1978.

Appendix A of Regulatory Guide 1.33, Section 2.c, recommends a general operating procedure on recovery from reactor trips.

General Operating Procedure GOP-0003, "Scram Recovery," Revision 11, Enclosure 1, posttrip review checklist Section 2.f, states, "Document any observed premature (safety relief valve) lifts in the safety mode here and receive an engineering evaluation of the event prior to restart."

Contrary to the above, on August 22, 1997, during a reactor scram event, safety relief valves lifted in the safety mode; however, the licensee did not document the safety mode lifts in the posttrip review checklist or perform an engineering evaluation of the event prior to restart.

This is a Severity Level IV violation (Supplement I) (50-458/97013-03)

- C. Technical Specification 5.4.1.a states, in part, that written procedures shall be implemented covering the applicable procedures recommended in Appendix A of Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operation)," Revision 2, February 1978.

Appendix A of Regulatory Guide 1.33, Section 9.a recommends, in part, that maintenance that can affect the performance of safety-related equipment should be properly planned and performed in accordance with written procedures.

Administrative Procedure ADM-0081, "Cleanliness Control," Revision 4, Section 8.2.8.6 requires personnel to minimize the use of temporary flexible material such as plastic sheeting, drop cloths and rags when in containment below the 141-foot elevation and the reactor is at power.

Contrary to the above, on August 11, 1997, while on the 95-foot elevation in containment with the reactor at power, modification workers failed to minimize the use of temporary flexible material, in that used protective anticontamination clothing was allowed to accumulate in a large open bag before it was removed from the area.

This is a Severity Level IV violation (Supplement I) (50-458/97013-05)

- D. 10 CFR Part 50, Appendix B, Criterion III, "Design Control," states, in part, that measures shall be established to assure that the design basis is correctly translated into procedures and instructions. The Updated Final Safety Analysis Report (UFSAR) Section 3.4.1.1.3 requires closure of the Residual Heat Removal (RHR) Pump Room A water-tight door to prevent adverse effects from flooding.

Contrary to the above, plant staff failed to translate into any procedure the UFSAR Section 3.4.1.1.3 requirement for closure of the RHR Pump Room A water-tight door, with the result that from August 11 through August 19, 1997, the water-tight door was blocked open for performance of heat exchanger performance testing during the September 12 refueling outage and there was no documented evidence that operating the plant in this configuration was reviewed.

This is a Severity Level IV violation (Supplement I) (50-458/97-13-06).

- E. Technical Specification 5.4.1.a states, in part, that written procedures shall be implemented covering the applicable procedures recommended in Appendix A of Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operation)," Revision 2, February 1978, and for the fire protection program.

Appendix A of Regulatory Guide 1.33, Section 1 recommends administrative procedures controlling safety-related activities carried out during the operational phase of nuclear power plants.

Administrative Procedure ADM-0015, "Station Surveillance Test Program," Revision 18, Section 8.9 states, in part, "If a problem occurs during a surveillance procedure, IMMEDIATELY NOTIFY THE APPROPRIATE SECTION SUPERINTENDENT OR DESIGNEE AND THE OSS/CRS (Operations Shift Superintendent/Control Room Supervisor), THEN perform the following: (1) Determine the cause and extent of the problem. (2) The OSS/CRS with the guidance of the appropriate Section Superintendent or Designee, shall determine whether the test should be continued or terminated . . ."

Fire Protection Procedure FPP-0040, "Control of Transient Combustibles," Revision 7, Section 5.2.4, states, in part, that untreated packing materials shall not be left unattended during lunch breaks, shift changes, or other similar periods.

Contrary to the above: (1) on August 21, 1997, during the performance of Surveillance Test Procedure STP-251-3602, "Fire Pump Functional Test," Revision 9, a problem developed during the performance of Step 7.2.15, (in that the batteries did not switch during the sequential cranking test as expected;) however, the test was resumed without notifying the OSS/CRS and obtaining his concurrence to proceed, and (2) Procedure FPP-0040 was not implemented in that from July 30 through August 7, 1997, untreated packing materials (wooden boxes containing new fuel channels) were left unattended during breaks and overnight.

This is a Severity Level IV violation (Supplement I) (50-458/97013-07).

Pursuant to the provisions of 10 CFR 2.201, Entergy Operations, Inc. is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region IV, 611 Ryan Plaza Drive, Suite 400, Arlington, Texas 76011 and a copy to the NRC Resident Inspector at the facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations and (4) the date when full compliance will be achieved. Your response may reference or include previous docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an order or a

Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. However, if you find it necessary to include such information, you should clearly indicate the specific information that you desire not to be placed in the PDR and provide the legal basis to support your request for withholding the information from the public.

Dated at Arlington, Texas
this 26th day of September 1997