



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

REGION IV  
611 RYAN PLAZA DRIVE, SUITE 400  
ARLINGTON, TEXAS 76011-8064

September 22, 1997

Harold B. Ray, Executive Vice President  
Southern California Edison Co.  
San Onofre Nuclear Generating Station  
P.O. Box 128  
San Clemente, California 92674-0128

SUBJECT: NRC INSPECTION REPORT 50-361/97-22; 50-362/97-22

Dear Mr. Ray:

The purpose of this letter is to announce our Maintenance Rule Baseline Team Inspection at your San Onofre Nuclear Generating Station, Units 2 and 3, reactor facilities. We have scheduled this inspection for December 1-5, 1997. The designated inspection team leader, J. E. Whittemore, of this office, will make arrangements with your staff to telephonically discuss the inspection plans in more detail.

The inspection objective will be to evaluate the effectiveness of your initial activities establishing baseline conditions to support compliance with the requirements of 10 CFR 50.65. The inspection will be conducted using Inspection Procedure 62706, "Maintenance Rule." This inspection will involve a team of 6-8 NRC personnel.

The inspection team will require information about your program and its implementation for their in-office preparation for the inspection. A description of the material that may be needed is in the enclosure to this letter. Our designated team leader will be contacting your staff and arranging for specific material to be available to the team by November 7, 1997.

If you have any questions concerning this inspection, please contact me at 817/860-8180 or Dr. Dale A. Powers, Chief, Maintenance Branch, at 817/860-8195.

Sincerely,

fr Arthur T. Howell, III, Director  
Division of Reactor Safety

Enclosure: as stated

Docket Nos.: 50-361; 50-362  
License Nos.: NPF-10; NPF-15

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PDR ADOCK 05000361  
Q PDR



cc w/enclosure:

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E-Mail report to T. Frye (TJF)  
 E-Mail report to T. Hiltz (TGH)  
 E-Mail report to NRR Event Tracking System (IPAS)  
 E-Mail report to Document Control Desk (DOCDESK)  
 E-Mail report to Richard Correia (RPC)  
 E-Mail report to Frank Talbot (FXT)

bcc to DCD (IE01)  
 bcc distrib. by RIV:

Regional Administrator  
 DRP Director  
 Branch Chief (DRP/F, WCFO)  
 Senior Project Inspector (DRP/F, WCFO)  
 Branch Chief (DRP/TSS)  
 WCFO File

Resident Inspector  
 DRS-PSB  
 MIS System  
 RIV File  
 M. Hammond (PAO, WCFO)

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## ENCLOSURE

### PREINSPECTION REQUESTED ITEMS LIST

1. Copy of Maintenance Rule program
2. Copies of procedures that directly relate to and support the Maintenance Rule program
3. List of acronyms and alpha-numeric designations of equipment within the scope of the Maintenance Rule (computer printout or computer disk is acceptable)
4. List of structures, systems, and components (SSCs), or functions, within the scope of the Maintenance Rule - this list should identify those SSCs, or functions that have been designated as risk/safety significant and the current performance criteria/goals
5. List of SSCs, or functions, that were excluded from the scope of the Maintenance Rule, including justifications for exclusions
6. List of SSCs, or functions, placed within (a)(1) and (a)(2) categories of the Maintenance Rule
7. Background explanation for SSCs currently assigned to the (a)(1) category
8. Results and/or information on historical performance review used to initially identify category (a)(1) SSCs for July 10, 1996, effective date of Maintenance Rule
9. Risk reduction worth, risk achievement worth, and core damage frequency survey results for SSCs
10. Listing of top 20 nonrisk-significant SSCs, which were not classified as risk-significant and rationalization for nonrisk-significant classification
11. Any updated material to the initial individual plant examination submittal
12. Updated risk ranking of SSCs including the numerical importance measure(s) for each SSC and the criteria utilized to establish the risk significance of these SSCs for the Maintenance Rule program
13. List of SSCs within the probabilistic risk assessment that were downgraded/changed to nonrisk significant or excluded from the Maintenance Rule program and the rationale for the changes
14. Listing of reliability and availability basic events in the probabilistic risk assessment used for risk ranking compared to the performance criteria established for those SSCs under the Maintenance Rule program
15. Criteria or bases used by the expert panel to determine risk significance of SSCs when the plant was not in Mode 1

16. The shutdown safety assessment tool/chart/matrix showing which systems are used for maintaining decay heat removal, reactor coolant system inventory control, electric power availability, reactivity control, and containment integrity while shutdown
17. The on-line maintenance procedures, tools, chart, and/or matrices including the bases for program risk levels
18. Current Maintenance Rule program trending data
19. List of items, events, and conditions that were evaluated for maintenance preventable functional failures for the previous two fuel cycles, and specific identification of MPFFs and repetitive MPFFs
20. Current program performance criteria and goals; also, provide a list of SSCs that are currently classified as "inherently reliable" or "run to failure"
21. Copies of any internal and external Maintenance Rule assessments or related audits
22. Current organization chart depicting key personnel involved in the Maintenance Rule implementation and a list of their office telephone numbers
23. Any additional information on organizational (engineering, operations, quality assurance, planning/scheduling, work control and maintenance) Maintenance Rule functional responsibilities
24. Qualifications and backgrounds of expert panel members

**IN ADDITION, THE INFORMATION BELOW SHOULD BE AVAILABLE TO THE INSPECTION TEAM UPON THEIR ARRIVAL ONSITE**

1. Minutes of expert panel meetings
2. Copies of corrective action documents and/or proposed corrective actions associated with identification of problems during implementation of the Maintenance Rule program, including probabilistic risk assessment aspects
3. Listing of spray shields, doors/hatches and detection alarms assumed operational in the probabilistic risk assessment for internal flooding
4. Copies of senior reactor operator and reactor operator logs for the last 6 months and copies of any equipment status documents (limiting condition for operation logs, equipment tagging logs, etc.), which provide information of what equipment (balance-of-plant and safety-related) was out-of-service for the last 6 months