## AVERAGE DAILY UNIT POWER LEVEL

Docket No.	50-272
Unit Name	Salem # 1
Date (	October10,1986
Telephone	609-935-6000
Extension	4451

IE at

Completed	by	Pell	White	

Month	Septer	ber .	1986
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Day Average Daily Power Level (MWe-NET)

Day Average Daily Power Level (MWe-NET)

1	1104	17	0
2	897	18	0
3	867	19	0
4	833	20	0
5	875	21	0
6	889	22	0
7	884	23	0
8	879	24	306
9	879	25	889
10	900	26	882
11	899	27	867
12	899	28	866
13	882	29	877
14	891	30	875
15	903	31	
16	740		

P. 8.1-7 R1

8610210239 860930 PDR АДОСК 05000272 R PDR

## OPERATING DATA REPORT

Docket No.50-272 Date October 10,1986 Telephone 935-6000 Extension 4451

# Completed by Pell White

N/A

## Operating Status

1. 2. 3. 4. 5. 6. 7. 8.	Unit Name Sa Reporting Period Se Licensed Thermal Power (MWt) Nameplate Rating (Gross MWe) Design Electrical Rating (Net MWe Maximum Dependable Capacity (Gross Maximum Dependable Capacity (Net If Changes Occur in Capacity Rat: Report, Give Reason N/A	alem No. 1 eptember 1986 3411 1170 e) 1115 s MWe) 1149 MWe) 1106 ings (items 3	<u>Notes</u> 3 through 7) sin	nce Last
9.	Power Level to Which Restricted,	if any (Net	MWe) 85%	
10.	Reasons for Restrictions, if any power transformers	Due to powe	er distribution	on station
		This Month	Year to Date	Cumulative
11.	Hours in Reporting Period	720	6551	81120
12.	No. of Hrs. Reactor was Critical	546.4	4888.2	49073.6
13.	Reactor Reserve Shutdown Hrs.	0	0	0
14.	Hours Generator On-Line	539	4714.8	47219.6
15.	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	1529129	14713558	145197587
17.	Gross Elec. Energy Generated	498930	4866410	48162420
18.	Net Elec. Energy Generated (MWH)	471116	4632833	45738325
19.	Unit Service Factor	74.8	72.0	58.2
20.	Unit Availability Factor	74.8	72.0	58.2
21.	Unit Capacity Factor	59.2	63.9	51.0
22.	Unit Capacity Factor	55.2	0.5.5	
	(using DER Net)	58.7	63.4	50.6
23.	Unit Forced Outage Rate	20,1	10.0	27.4
24.	Shutdowns scheduled over next 6	months (type)	, date and dura	tion of each

25. If shutdown at end of Report Period, Estimated Date of Startup: N/A

26.	Units i	n Test	Status	(Prior to Commen	rcial	Operation):	
						Forecast	Achieved
				Initial Critica	ality	9/30/76	12/11/76
				Initial Electri	icity	11/1/76	12/25/76
				'Commercial Open	cation	n 12/20/76	6/30/77
8-1	-7.R2						

#### UNIT SHUTDOWN AND POWER REDUCTIONS REPORT MONTH SEPTEMBER 1986

Docket No. 50-272 Unit Name Salem No.1 Date October 10,1986 Telephone 609-935-6000 Extension 4451

Completed by Pell White

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
666	0902	F	4.9	D	5		EG	TRANSF	Nuc. Reg. Proceedings and Hearings
668	0902	F	6.7	D	5		EG	TRANSF	Nuc. Reg. Proceedings and Hearings
670	0902	F	15.0	D	5		EG	TRANSF	Nuc. Reg. Proceedings and Hearings
672	0903	F	71.2	D	5		EG	TRANSF	Nuc. Reg. Proceedings and Hearings
682	0906	F	70.2	D	5		EG	TRANSF	Nuc. Reg. Proceedings and Hearings
684	0909	F	182.2	D	5		EG	TRANSF	Nuc. Reg. Proceedings and Hearings
698	0916	F	181	A	1		HB	PIPEXX	Steam Extraction Piping
700	0924	F	113.8	В	5		EG	TRANSF	Station Service Transformer station power auxiliary
702	0929	F	44.0	В	5		EG	TRANSF	Station Service Transformer station power auxiliary

1

F: Forced S: Scheduled

A-Equipment Failure-explain B-Maintenance or Test C-Refueling D-Regulatory Restriction E-Operator Training & Licensing Exam F-Administrative G-Operational Error-explain H-Other-explain

2 Reason

3 Method	4 Exhibit G	5 Exhibit 1
1-Manual	Instructions	Salem as
2-Manual Scram.	for Prepara-	Source
3-Automatic Scram.	tion of Data	
4-Continuation of	Entry Sheets	
Previous Outage	for Licensee	
5-Load Reduction	Event Report	
9-Other	(LER) File	
	(NUREG 0161)	

MAJOR PLANT MO REPORT MONTH	DIFICATIONS SEPTEMBER 1986	DOCKET NO.: UNIT NAME: DATE: COMPLETED BY: TELEPHONE:	50-272 Salem 1 October 10, 1986 L. Miller 609/339-4497
*DCR NO.	PRINCIPAL SYSTEM	SUBJECT	r
1EC-1676A	Reactor Coolant Sys.	Replace Re range resi detectors qualified	eactor Coolant narrow istance temperature with environmentally elements.
1SC-1268	Hydrogen Recombiners	s Install la Unit No. 1 access to Monitor Sy to perform	adder and platform in Containment to gain 11 and 12 Hydrogen ystem sensor assembly maintenance.
1SC-1579	Main Steam Safety Valve	Remove the forked lev spindle nu safety val are on Cro which is a to be inst spindle ro	e lever #17, the ver #22 and the at #12 on all (20) lves. Above items osby print #G-52950 attached. Item #25 called to protect od.

\* DCR - Design Change Request

MAJOR PLANT MODIFICATIONS REPORT MONTH SEPTEMBER 1986

\*DCR

DOCKET NO.: 50-272 UNIT NAME: Salem 1 DATE: October 10, 1986 COMPLETED BY: L. Miller TELEPHONE: 609/339-4497

#### SAFETY EVALUATION 10 CFR 50.59

1EC-1676A This design change enhances plant safety by reducing the closing response times of the MS-7 and MS-8 valves. The 5.0 second response time provides a conservative limit relative to the accident analysis of the UFSAR. The modification does not alter any process or discharge and does not affect the existing plant impact. No unreviewed safety or environmental questions are involved.

1SC-1268 The telescoping ladders and platforms conform to ANSI standards. Seismic restraints are provided to negate the effects that a seismic event would have on the platforms while they are in the stored position. The modification does not change any plant process or discharge and does not affect the existing plant impact. No unreviewed safety or environmental questions are involved.

1SC-1579 The intended operation of the valves and the function of the valves is in no way altered. The component removal does not result in a violation of Section III since the pressure setpoints will not be affected by the removal. The lifting lever requirement for safety valves has been deleted from ASME Section III since 1977. Due to the cleaner environment of the nuclear power plant over that of fossil plants, oxidation in the nozzle/disk area is precluded. Thus the manual lifting levers are unnecessary. This modification neither affects the seismic integrity of the valves or increases their likelihood of failure. There is no impact on any Technical Specification basis or safety analysis. The modification does not change any plant process or discharge and does not affect the existing plant impact. No unreviewed safety or environmental questions are involved.

\* Design Change Request

## PSE&G SALEM GENERATING STATION SAFETY RELATED WORK ORDER LOG

WO NO	UNIT	EQUIPMENT IDENTIFICA	TION
8605080850	1	GE 4KV BREAKER	
		FAILURE DESCRIPTION:	VISUALLY INSPECT AND TIME 4KV MAGNABLAST CIRCUIT BREAKER.
		CORRECTIVE ACTION:	5 YEAR OVERHAUL BY G.E. PO PI-123227. SERIAL 0224A6261-018. INSTALLED 22BSD 22 SPT INFEED 2B 4KV B.
8608210479			
	1	4KV-1200A BREAKER	
		FAILURE DESCRIPTION:	TROUBLESHOOT G.E. MAG BLAST 4KV-1200A BREAKER SERIAL #0224A6268-002 IAW DR SMD-IE-86-257.
		CORRECTIVE ACTION:	REPLACED PROP SPRINGS. PERFORMED PROCEDURES M3D AND M3J.
8608290014			
	1	BATTERY	
		FAILURE DESCRIPTION:	WHILE PERFORMING M10 SST 009-1 ON #1 DIESEL FIRE PUMP THE BATTERY BLEW UP. PLEASE REPLACE THE BATTERY BANK AND INVESTIGATE CAUSE.
		CORRECTIVE ACTION:	REPLACED BATTERY BANK 4 BATTERIES. BAD CELL HAD INTERNAL FAILURE.

WO NO	UNIT	EQUIPMENT IDENTIFICA	TION
0099170213	1	11 CFCU SW INLET	
		FAILURE DESCRIPTION:	IT APPEARS THAT THE VANE HAS BROKEN LOOSE FROM THE VANE ARM. WHEN SCORE ON VANE ARM INDICATES CLOSED THERE IS 400 GPM; WHEN INDICATING OPEN THERE IS ZERO FLOW.
		CORRECTIVE ACTION:	CHECKED VALVE. REPLACED LINER, SHAFT PINS, PUT VALVE BACK IN PIPING.
0099182025			
	1	ICU55 CHG CONTROL	
		FAILURE DESCRIPTION:	PLEASE REMOVE THE MINIMUM STOP ON CV55 TO ALLOW BETTER CONTROL AT REDUCED RCS PRESS.
		CORRECTIVE ACTION:	ADJUSTED 1CU55 STEM FOR MIN FLOW OF 30 GPM AS PER OPERATIONS. ADJUSTED 1CV55 STEM FOR MIN FLOW OF 7 GPM.
0099182319			
	1	OHA	
		FAILURE DESCRIPTION:	ALARM WAS NOT RECEIVED FOR "14 RCP BRG COOLING WATER FLOW LOW" AFTER CHARGING WAS ISOLATED. PLEASE INVESTIGATE AND REPAIR.
		CORRECTIVE ACTION:	FOUND ALARM SWITCH STICKING, FIXED SAME. PERFORMED CAL ON 1FIC-622 VERIFIED OHA C/29.

WO NO	UNIT	T EQUIPMENT IDENTIFICATION		
8512180781	1	RX TRIP BREAKER		
		FAILURE DESCRIPTION:	PERFORM CONTROLS VERIFICATION ON RX TRIP BREAKER 1ASTR-B IN ACCORDANCE WITH TEST PACKAGE FOR 1EC-1365. REFER TO PROCEDURE 11C-18.1.011.	
		CORRECTIVE ACTION:	CONNECTED LEADS FOR SPDS 1ASTR B TO TB518-1,2. ALSO CONNECTED GROUND WIRE TB-518-3 TO CHASSIS GROUND. VERIFIED OPERATION AS PER IO 400026.	
604021124				
	1	SFP SKIMMER RAD. MONITOR		
		FAILURE DESCRIPTION:	INVESTIGATE IRREGULAR READINGS FROM U/1 SPENT FUEL SKIMMER FILTER RAD. MONITOR. HP SURVEYS INDICATE READINGS CONSIDERABLY LESS THAN THAT ON MONITOR. SEE ATTACHED WORK ORDER COPY.	
		CORRECTIVE ACTION:	FOUND METER READING LOW. REPLACED WITH NEW METER AND MADE LOOP CHECK ON NEW METER AND DRAWER.	
606160075				
	1	4KV CKT BREAKER VALVE		
		FAILURE DESCRIPTION:	TROUBLESHOOT AND REPAIR G.E. 4KV CKT BREAKER NO. 0224A6260-006. THIS BREAKER FAILED TO CLOSE UPON RECEIVING A START SIGNAL FOR 12 AUX FEED PUMP. FOUND ANTI-PUMP RELAY HUNG UP. BREAKER HAD 5 YEAR OVERHAUL ON 3/86.	
		CORRECTIVE ACTION:	BREAKER WAS RETURNED TO G.E. FOR REPAIR IAW DR SMD-IE-86-187. BREAKER RETESTED UNDER WORK ORDER 86-08-21-053-3.	

SALEM UNIT 1

NO NO	UNIT	EQUIPMENT IDENTIFICAT	ION
8607031988	1	RMS SPENT FUEL FILTER	
		FAILURE DESCRIPTION:	THE POWER SUPPLY FOR 1R28 APPEARS TO HAVE BURNED UP AND THE FUSES ARE BLOWN. PLEASE INVESTIGATE AND REPAIR.
		CORRECTIVE ACTION:	REPLACED TRANSFORMER AND TRANSISTOR Q2, REPAIRED BROKEN WIRE AT FIELD CONNECTOR. CALIBRATED CHANNEL AND DETECTOR.
8607240145	1	RX TRIP BREAKER 02YN201-1	
		FAILURE DESCRIPTION:	REACTOR TRIP BREAKER 02YN201-1 / PERFORM SEMI-ANNUAL INSPECTION. NOTE DURING REFUELING PERIODS PERFORM IN CONJUNCTION WITH IO 252135 AND IO 201908.
		CORRECTIVE ACTION:	PERFORMED SEMI-ANNUAL INSPECTION IAW M3Q-2 ON THIS REACTOR TRIP BREAKER. SER #02YN201-1.
8607241613	1	S/G 14 SNUBBER (12A)	
		FAILURE DESCRIPTION:	HYDRAULIC SNUBBER SER #12A, LOCATED IN 14 S/G SE POSITION HAS A LEAK ON THE 90 DEGREE FITTING GOING INTO THE VALVE BLOCK ALSO RESERVOIR IS LOW. TIGHTEN LEAKING FITTING AND FILL RESERVOIR.
		CORRECTIVE ACTION:	TIGHTENED FITTING AND REPLACED OIL.

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8012022	1	DUCT LOCAL LEVEL	
	1	DWCT LOCAL LEVEL	
		INDICATION	
		FAILURE DESCRIPTION:	LOCAL INDICATION READS 42.5' CONTROL BEZEL CHANNEL D & B ARE READING 41.9' IWH OHA D-35 IN SOLID. PLEASE INVESTIGATE AND REPAIR.
		CORRECTIVE ACTION:	CALIBRATED LOCAL INDICATOR AND ALARM SWITCHES. RETURNED TO SERVICE READING 41' COMPARED TO BEZEL CHANNELS B & D READING 41'.

FAILURE DESCRIPTION: THE RCS PRESSURE MASTER CONTROLLER WAS INDICATING 85% SPRAY DEMAND (68% ON THE MASTER DEMAND) BEFORE THE ACTUAL SPRAY VALVE DEMAND MOVED FROM THE 0% POSITION. ACTUAL RCS PRESSURE WAS AT 2262#. PLEASE INVESTIGATE AND REPAIR.

CORRECTIVE ACTION: REMOVED CONTROLLER SN E0123 FROM 1PC-455F INSTALLED CONTROLLER FROM 1PC-556. REMOVED MAN/AUTO STATION ON SN 102 FRM 1HC 455A INSTALLED SN 20141 IN 1HC 455B AND SN E0128 IN 1PC-455G. PERFORMED CAL ON 1PC-455F.

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1 RCP THERM. BARR. DISCHARGE

> FAILURE DESCRIPTION: THE HIGH FLOW ALARM (175 GPM) IS COMING IN AND OUT WITH NO OTHER INDICATION OF A HIGH FLOW CONDITION. THIS ALARM SHUTS THE 1CC131 WHICH ISOLATES CC FROM THE RCP'S. INVESTIGATE AND REPAIR. CORRECTIVE ACTION: CHECKED CAL ON 1F12625 FOUND IN SPEC. ADJUSTED

> > CLOSER TO REQUIRED.

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201	12.12.2	0747	* *

WO NO	UNIT	EQUIPMENT IDENTIFICATION		
8609050238	1	1N36 NIS 7 DAY FUNCTION		
		FAILURE DESCRIPTION:	NUCLEAR INSTRUMENTATION SYSTEM/PERFORM CHANNEL FUNCTIONAL TEST OF INTERMEDIATE RANGE CHANNEL N-36 MUST BE PERFORMED WITHIN SEVEN DAYS OF STARTUP.	
		CORRECTIVE ACTION:	FUNCTION SATISFACTORY.	
8609050661				
	1	BEZEL ALARM VCT HI/LO PRE		
		FAILURE DESCRIPTION:	THE COMPARATOR IS TRIPPED AND THE FUSE IS BLOWN FOR THIS BEZEL ALARM. PLEASE INVESTIGATE AND REPAIR.	
		CORRECTIVE ACTION:	FOUND CONNECTOR ON 1PC-139A/B LOOSE. REPLACED CONNECTOR AND CHECKED TRIP SETPOINT 11C-2.10.024.	
8609110532				
	1	16SW15		
		FAILURE DESCRIPTION:	PLEASE REMOVE ACTUATOR. MAINTENANCE HAS TO OPEN, INSPECT AND REPAIR VALVE AS NECESSARY.	
		CORRECTIVE ACTION:	INSTALLED HANDSENDER ON VALVE. CHECKED STROKE, ADJUSTED LIFT OFF SPRING TENSION, INSTALLED NEW GAUGE ON REGULATOR, REMOVED HANDSENDER. CALIBRATED AND RETURNED TO SERVICE.	

WO NO	UNIT	EQUIPMENT IDENTIFICATION EXCORE NIS SYSTEM		
8609151271	1			
		FAILURE DESCRIPTION:	CALIBRATE THE TARGET DELTA-I TO A VALUE OF -4.0% (100% RTP VALUE). SEE SPECIAL INSTRUCTIONS FOR IMPLEMENTATION TIMING.	
		CORRECTIVE ACTION:	RECALIBRATED AXIAL FLUX DIFFERENCE TARGET BAND TO 4% AS PER 1IC-16.1.008. ADJUSTED 10M200M XA-8440 RB GREEN PEN AND XA-8740RB RED PEN.	
8609210031 1 12 WASTE GAS COMPRESSOR		12 WASTE GAS COMPRESS	OR	
		FAILURE DESCRIPTION:	INVESTIGATE AND REPAIR COMPRESSOR. WILL NOT TURN.	
		CORRECTIVE ACTION:	REPLACED SEAL COVER ROTOR. INSPECTED INTERNAL SURFACES AND FOUND TO BE ERODED BADLY.	

## SALEM GENERATING STATION MONTHLY OPERATING SUMMARY - UNIT NO. 1 SEPTEMBER 1986

### SALEM NO. 1

On September 1, 1986, power was reduced to approximately eighty percent (80%) due to problems associated with the loading of the station power transformers. This problem is addressed in the summary for Unit No. 2. On September 16, 1986 load, was increased to 98% power based on the availability of electrical power from Unit 2 to operate No. 11 Condensate Pump. At 2115 hours, the same day, the Unit was taken off line due to a steam leak at the weld joint between a bleed steam pipe and the high pressure turbine exhaust line. The leak was caused by steam erosion. The pipe replacement was completed on September 22, 1986. On September 24, 1986, following the correction of problems with No. 13 Auxiliary Feed Pump steam supply check valves, the Unit was returned to service. The unit operated at approximately eight three percent (83%) power for the remainder of the period.

COMP	REFUELING INFORMATION DOCKET NO.: 50-272 UNIT NAME: Salem 1 DATE: October 10, 1986 TELEPHONE: 609/935-6000 EXTENSION: 4497
Mont	h September 1986
1.	Refueling information has changed from last month: YES NOX
2.	Scheduled date for next refueling: September 12, 1987
3.	Scheduled date for restart following refueling: November 26, 1987
4.	<ul> <li>A) Will Technical Specification changes or other license amendments be required? YES NO NOT DETERMINED TO DATE X     </li> <li>B) Has the reload fuel design been reviewed by the Station Operating Review Committee? YES NO X         If no, when is it scheduled? August 1987     </li> </ul>
5.	Scheduled date(s) for submitting proposed licensing action: August 1987 if required
6.	Important licensing considerations associated with refueling:
7.	Number of Fuel Assemblies: A) Incore B) In Spent Fuel Storage 380
8.	Present licensed spent fuel storage capacity: 1170
	Future spent fuel storage capacity: 1170
9.	Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001
8-1-	7.R4

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Public Service Electric and Gas Company P.O. Box E. Hancocks Bridge, New Jersey 08038

Salem Generating Station

#### October 10, 1986

Director, Office of Inspection and Enforcement U.S. Nuclear Regulatory Commission Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT SALEM NO. 1 DOCKET NO. 50-272

In compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of September 1986 are being sent to you.

> Average Daily Unit Power Level Operating Data Report Unit Shutdowns and Power Reductions Major Plant Modification Safety Related Work Orders Operating Summary Refueling Information

> > Sincerely yours,

Zullo (va)

J. M. Zupko, Jr. General Manager - Salem Operations

JR:sl

cc: Dr. Thomas E. Murley Regional Administrator USNRC Region I 631 Park Avenue King of Prussia, PA 19406

> Director, Office of Management U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Enclosures 8-1-7.R4