

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 7 1				PAGE (3) OF 0 3		
TITLE (4) Inadequate Containment Leak Rate Testing Due To Lack Of Procedural Requirement For Venting																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
									Sequoyah, Unit 2				0 5 0 0 0 3 2 8			
0 9 1	0 8 6	8 6		0 4 3		0 0 1	0 1 0	8 6					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)														
5		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.405(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)		
0 1 0 1 0		20.405(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)				XX 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME										TELEPHONE NUMBER						
David P. Ormsby, Plant Operating Review Staff										AREA CODE						
										6 1 5 8 7 0 - 6 1 4 6						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDs						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

While both units were in mode 5 at zero percent power, a review was conducted of procedures used to meet technical specifications (TSs) surveillance requirements (SRs) for containment leak rate testing. On September 10, 1986, it was determined that Surveillance Instruction (SI)-156 for SR 4.6.1.2 did not provide for venting of three containment isolation valves during testing. Although additional testing has shown that leakage is within limits, the inadequate test procedure did not provide the necessary assurance that SRs were being met. This constitutes operation prohibited by TS and is reportable in accordance with 10 CFR 50.73, paragraph a.2.i.B, for both units 1 and 2.

The results of type C valve leak testing were found for all valves to be zero leakage; therefore, no adjustment to the latest measured containment integrated leak rate test is necessary. SI-156 will be changed to require venting before the next required performance.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0500032786	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		86	043	0	0	2	OF 03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

Two words

With both units 1 and 2 in mode 5 at zero percent power, a review was conducted of plant surveillance instructions (SIs) used to complete technical specifications (TSs) surveillance requirements (SRs) for periodic leak testing. On September 10, 1986, it was determined that procedure SI-156, "Containment Integrated Leak Rate Test," for SR 4.6.1.2 was inadequate for leak testing of containment isolation valves FSV-30-134 (EIIS System-DB), FSV-30-135 (EIIS Identifier-FSV), and VLV-59-529 (EIIS Identifier-V). FSV-30-134 and FSV-30-135 are containment annulus differential pressure isolation valves. To properly test leakage through these valves, it is necessary to vent the line inside of the annulus to atmospheric conditions. VLV-59-529, in the demineralized water system (EIIS System-KC), is also required to be vented to ensure that the backside of the valve is at atmospheric pressure. SI-156 did not contain provisions for venting of these valves. This condition is applicable to both units 1 and 2, and it constitutes operation prohibited by TS which is reportable in accordance with 10 CFR 50.73, paragraph a.2.i.B. No immediate operator action is required, because containment isolation is not required in the current mode 5 operation.

ANALYSIS OF EVENT

SI-156 ensures that for type "A" integrated leak rate testing, the total containment leakage volume will not exceed the values assumed in the accident analysis. Without proper venting, the actual leakage which could have occurred during accident conditions could not be accurately determined. However, subsequent testing of VLV-59-529 with the required venting shows zero leakage. In addition, local leak rate testing (Type C test) previously performed on valves FSV-30-134 and FSV-30-135 has shown negligible leakage. Therefore, there is reasonable assurance that the leakage through the valves has been within limits, and that the total leakage volumes assumed in the accident analysis have not been violated.

CAUSE OF EVENT

The inadequacies in the leak test procedure are attributed to a previously inadequate SI review for verification of applicable test requirements. The need for venting these isolation valves to ensure accurate leak test results was not considered for these valves during the initial or subsequent periodic reviews.

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Sequoyah, Unit 1	0 5 0 0 0 3 2 7	8 6	- 0 4 3	- 0 0	0 3	OF 0 3

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CORRECTIVE ACTION

VLV-59-529 has been retested with the required venting and shows zero leakage. Previous Type C testing of valves FSV-30-134 and FSV 30-135 with required venting shows zero leakage; therefore, no adjustment to previous SI-156 results is required. SI-156 will be changed before the next required test performance to reflect the proper vent requirements. The SI review program currently in progress which identified this event will ensure the technical adequacy of this SI.

ADDITIONAL INFORMATION

A review of LER history files shows one previous occurrence of reportable deficiencies in containment leak testing procedures - SQRO-50-327/86040.

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TENNESSEE VALLEY AUTHORITY
Sequoyah Nuclear Plant
Post Office Box 2000
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October 10, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

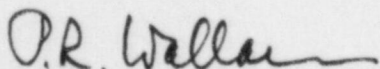
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNITS 1 AND 2 - DOCKET
NOS. 50-327 AND 50-328 - FACILITY OPERATING LICENSE DPR-77 - AND -79 -
REPORTABLE OCCURRENCE REPORT SQRO-50-327/86043

The enclosed licensee event report provides details concerning inadequate
containment leak rate testing due to a lack of procedural requirements
for venting. This event is reported in accordance with 10 CFR 50.73,
paragraph a.2.i.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

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NRC Inspector, Sequoyah Nuclear Plant

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11