NRC FORM 366 (4-95)			U.S. NUCLEAR REGULATORY COMMISSION								APPROVED BY OMB NO. 3150-0104 EXPIRES 04/30/98 ISTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST 50.0 HRS. REPORTED LESSONS							
			ENSE	ENSEE EVENT REPORT (LER)								LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT REANCH IT 6 F/31. U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 2055-10001, AND TO THE PAPERWORK REDUCTION PROJECT 1315-20104						
			(See rev digits	verse /chai	e for req racters f	uire or e	d number o ach block)	f			UPPlus	UP MANAGEMEN	AND BUDGET.	WASHIN	1010N, DC 20*53			
FACILITY NAME (1) Millstone Nuclear Power Station Unit 3										050004	PAGE (3) 1 of 3							
TITLE (4)	ailure	to Co	mpletel Valves	y Te that	st the t Recei	The ve N	rmal Over Aultiple Ad	load Byp	ass Pro Signals	otectio	l n Logi	c of Safet	y Related	Moto	or Operate	d		
EVEN	TDATE	(5)	T	LE		ERI	6)	REPO	RTDAT	E (7)	OTHER FACILITIES INVOLVED (P)							
MONTH	DAY	YEAR	YEAR	SE	NUMBER	AL	REVISION	MONTH	DAY	YEAR	FACILI	TY NAME		DOCKET NUMBER				
08	15	97	97		047		00	09	15	97	FACILI	TY NAME	DOCKET NUMBER					
OPERA	TING	5	THIS R	EPOP	RT IS SUI	BMIT	TTED PURSU	ANT TO TH	HE REQU	JIREMEN	TSOF	10 CFR 8: 10	Check one o	r more) (11)			
MODE	(9)		20	.220	1(b)			20.22030	(a)(2)(v)		D	50.73(a)	2)(i)	TT	50.73(a)(2)(viii)		
POW	ER	000	20	.220	3(a)(1)			20.2203(a)(3)(i)			K	50.73(a)(2)(ii)		50.73(a)(2)(x)		x)		
LEVEL	(10)		20	20.2203(a)(2)(i)			20.2203(a)(3)(ii)				50.73(a)		2)(iii) 7		73.71	73.71		
			20	.220	3(a)(2)(ii)		20.2203	(a)(4)		-	50.73(a)(2)(iv)	+-+	OTHER			
			20	20.2203(a)(2)(iii)				50 36(c)((1)			50.73(a)(2)(v)	2)(v) Specifi		fy in Abstract below		
Sec.		20.2203(a)(2)(iv)				50.36(c)(2)			50.73(a)(2)(vii)			or in NRC Form 386A						
							LICENSEE	CONTACT	FOR TH	HIS LER (12)			_				
NAME	D	avid A	. Smith	n, M	P3 Nuc	lea	r Licensing) Manage	er			ELEPHONE NUM	8ER (Include A) (860)43	7-58	») 840			
		COMP	LETE O	NE		DR	EACH COM	MPONEN	TEAIL	URE D	ESCR	BED IN TH	IS REPOR	RT (1	31			
CAUSE	SYST	EM CO	MPONENT	MA	NUFACTU	RER	REPORTABL TO NPRDS	E	CAUS	E SY	STEM	COMPONENT	MANUFAC	TURER	REPORT. TO NPF	ABLE		
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YES (If yes,	comple	ete EXPI	CTED S	JBMI	SSIOND	ATE).	NO			DATE	SSION (15)						

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

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On August 13, 1997, with the unit in Mode 5, it was identified that the testing of the safety-related motor operated valves (MOVs) which receive multiple actuation signals, may not have been adequately tested to ensure that the thermal overload protection, was bypassed for each actuation signal as required by Technical Specification (TS) 3.8.4.2.1, "Electrical Power Systems, Motor-Operated Valve Thermal Overload Protection." On August 15, 1997, it was determined that the testing performed was not adequate, and that this condition was reportable. This condition was discovered and described in accordance with the Millstone Corrective Action program via Condition Report M3-97-2613.

Because proper operation of the thermal overload bypasses was not demonstrated, and this situation existed when the MOVs were required to be OPERABLE, this resulted in performance of inadequate (or incomplete) surveillances, which are equivalent to missed TS surveillances. TS 4.0.3 states, "Failure to perform a Surveillance Requirement within the allowed surveillance interval, defined by Specification 4.0.2, shall constitute noncompliance with the OPERABILITY requirements for a Limiting Condition for Operation." This is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B), as a condition or operation prohibited by the unit's TS.

Although, direct verification that the thermal overload protection would be properly bypassed upon receipt of each individual signal was not performed, there were no safety consequences involved in that the portions of the circuits, which function as the thermal overload bypass circuitry, are tested during loss of power and slave relay surveillance testing.

This is a historical event and the cause is indeterminate. By inspection, the root cause is considered to be inadequate initial development of the applicable surveillance test procedures. Prior to entry into Mode 4, surveillance testing procedures will be revised and testing performed such that the the mal overload bypass function is tested for Reactor Plant Component Cooling Water valves 3CCP*MOV222 through 229 and Service Water System valves 3SWP*MOV115A and B.

NRC FORM 366A (4-95)			U.S.	NUCLE	ARRI	EGULATORY	COMMISSIO			
FACILITY NAME (1)	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION					(1)				
		YEAR	SEQUENTIAL REVISION				FAGE (3)			
Millstone Nuclear Power Station Unit 3	05000423		NUMBER		NUMBER	2 of 3				
		97		047		00				

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

1. Description of Event

On August 13, 1997, with the unit in Mode 5, it was identified that the testing of the safety-related motor operated valves (MOVs) listed below, which receive multiple actuation signals, may not have been adequately tested to ensure that the thermal overload protection, was bypassed for each actuation signal as required by Technical Specification (TS) 3.8.4.2.1, "Electrical Power Systems, Motor-Operated Valve Thermal Overload Protection." On August 15, 1997, it was determined that the testing performed was not adequate, and that this condition was reportable. This condition was discovered and described in accordance with the Millstone Corrective Action program via Condition Report M3-97-2613.

•	3CCP*MOV222	Containment Recirculation Cooling Coil C Supply \	/alv
	3CCP*MOV223	Containment Recirculation Cooling Coil C Supply \	Jalu

- 3CCP*MOV224
 Containment Recirculation Cooling Coil C Return Valve
- 3CCP*MOV225 Containment Recirculation Cooling Coil C Return Valve
- 3CCP*MOV226 Containment Recirculation Cooling Coil B Supply Valve
- 3CCP*MOV227 Containment Recirculation Cooling Coil B Supply Valve
- 3CCP*MOV228 Containment Recirculation Cooling Coil B Return Valve
- 3CCP*MOV229 Containment Recirculation Cooling Coil B Return Valve
- 3SWP*MOV115A Circulating Water Pumps Lubricating Water Valve
- 3SWP*MOV115B Circulating Water Pumps Lubricating Water Valve

TS Surveillance Requirement (SR) 4.8.4.2.1 states: "The thermal overload protection for the above referenced valves shall be verified to be bypassed by the appropriate accident signal(s) by performance of a TRIP ACTUATION DEVICE OPERATIONAL TEST of the bypass circuitry during COLD SHUTDOWN or REFUELING at least once per 18 months." The APPLICABILITY section of TS 3.8.4.2.1 specifies that the bypass device is required to be OPERABLE whenever the MOV is required to be OPERABLE. Because proper operation of the thermal overload bypasses was not demonstrated, and this situation existed when the MOVs were required to be OPERABLE, this resulted in performance of inadequate (or incomplete) surveillances, which are equivalent to missed TS surveillances. TS 4.0.3 states, "Failure to perform a Surveillance Requirement within the allowed surveillance interval, defined by Specification 4.0.2, shall constitute noncompliance with the OPERABILITY requirements for a Limiting Condition for Operation." This is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B), as a condition or operation prohibited by the unit's Technical Specifications.

II. Cause of Event

This is a historical event and the cause is indeterminate. By inspection, the root cause is considered to be inadequate initial development of the applicable surveillance test procedures.

III. Analysis of Event

Failure to adequately test safety-related actuation logic circuitry is safety significant. Inoperable essential electrical components required for automatic actuation of accident mitigation systems can contribute to overall risk and can place additional burden on plant operators in requiring manual actuation of required functions. Additionally, compliance with the TS surveillance requirements is essential to maintaining the validity of assumptions in the licensing basis accident analyses. The OPERABILITY of the motor-operated valves thermal overload protection and bypass devices, ensures that the thermal overload protection will not prevent the safety-related MOVs from performing their safety functions.

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. LICENSE	EXT CONTINUATION	LER)						
ACILITY NAME (1)	DOCKET NUMBER (2) LER NUMBER (6)							
Millstone Nuclear Power Station Unit 3	05000423	YEAR	SEC	UNBER	3 of 3			
		97	**	047 -	00			
TEXT (If more space is required, use additional copies of NRC Form	n 366A) (17)							
Fillure to ensure complete testing of the thermal overlo missed TS surveillances.	oad bypass logic results ir	n testing	g inad	dequaci	es that const	itute		
Although, direct verification that the thermal overload prindividual signal was not performed, there were no safe function as the thermal overload bypass circuitry, are te	rotection would be proper ety consequences involve ested during loss of powe	ly bypa ed. The r and sl	e port lave i	d upon re tions of t relay su	eceipt of eac he circuits w veillance tes	h hich ⊧ting.		
IV. Corrective Action								
While in Mode 5, there was no impact on equipment of	operability and no immed	iate cor	rrecti	ive actio	ns were req	uired.		
The following corrective action has been completed.								
 A review of the surveillance test procedures asso identify those motor operated valves receiving mu bypass function was tested for each actuation sig 	ciated with Technical Sp ultiple actuation signals, a mal.	ecificat and to v	ion 3 verify	8.4.2.1 that the	was perforr thermal ov	ned to erload		
The following corrective action will be taken prior to en	ntry into Mode 4:							
 Surveillance testing procedures will be revised an operation of the thermal overload bypass function 3CCP*MOV222 through 229 and Service Water S 	nd testing performed for e n for Reactor Plant Comp System valves 3SWP*MC	each ac onent ()V115A	tuati Cooli A and	on signa ng Wate d B.	al, to verify p er valves	roper		
V. Additional Information								
None								
Similar Events								
An LER discussing inadequate testing and is identified Program are being conducted to detect design and lice	d below. Various elemer censing basis problems.	nts of th	ne Co	onfigura	tion Manage	ment		
LER 97-017-01 Inadequate Testing of Logic Circuit	ts							
Manufacturer Data								
EIIS System Code Reactor Plant (Closed) Component Cooling Water Sy Essential & Nonessential Service Water Systems	stemCC Bl a	nd KG						

NRC FORM 366A (4-95)