

NOTICE OF VIOLATION

Florida Power Corporation
Crystal River Nuclear Plant
Unit 3

Docket No. 50-302
License No. DPR-72
EA 97-330

During NRC inspections conducted on June 2 through June 20, 1997, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedures for NRC Enforcement Actions," NUREG-1600, the violations are listed below:

- A. 10 CFR 50.59(a)(1) states, in part, that licensees may make changes to the facility or procedures as described in the safety analysis report, without prior Commission approval, unless the proposed change involves an unreviewed safety question (USQ). 10 CFR 50.59(a)(2) states, in part, that a proposed change shall be deemed to involve a USQ if the probability of occurrence of a malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased. 10 CFR 50.59 (b)(1), in part, states that the licensee shall maintain records of changes in the facility and of changes in procedures made pursuant to this section. These records must include a written safety evaluation which provides the bases for the determination that the change does not involve a USQ. 10 CFR 50.59 states that a licensee who desires to make a change in the facility or procedures described in the safety analysis report which involves a USQ shall submit an application for amendment of his license pursuant to 10 CFR 50.90.

Contrary to the above, on December 21, 1987, the licensee made a change to the facility, without prior Commission approval, that involved an unreviewed safety question (USQ). The change, which was implemented under plant modification MAR No. 80-09-13-01, Emergency Diesel Generators (EDG) Relaying Modification, modified the EDG control circuits as described in Figure 8-9 of the Final Safety Analysis Report (FSAR). The change increased the probability of failure of the EDGs by adding electrical components to the EDG protective lockout circuit that were not bypassed during emergency operation and were not installed with two out of three coincidence logic. A single failure in the added components could cause the EDG to trip during emergency operation. The modification safety evaluation, which was approved by the licensee's staff on February 17, 1987, failed to identify the USQ. (01013)

- B. 10 CFR 50.71(e) states that each person licensed to operate a nuclear power reactor shall periodically update the FSAR to assure that the information included in the FSAR contains the latest material developed. Revisions must be filed annually or 6 months after each refueling outage provided the interval between successive updates does not exceed 24 months.

Contrary to the above, as of June 25, 1997, the licensee had failed to update Figure 8-9 of the FSAR to include MAR 80-09-13-01, Emergency Diesel Generators (EDG) Relaying Modification, completed on December 21, 1987. (01023)

These violations represent a Severity Level III problem (Supplement I).

The NRC has concluded that information regarding the reason for the violations, the corrective actions taken and planned to correct the violations and prevent recurrence, and the date when full compliance will be achieved is already addressed on the docket in your letter dated August 11, 1997. However, you are required to submit a written statement or explanation pursuant to 10 CFR 2.201 if the description therein does not accurately reflect your corrective actions or your position. In that case, or if you choose to respond, clearly mark your response as a "Reply to a Notice of Violation," and send it to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region II, and a copy to the NRC Resident Inspector at Crystal River, within 30 days of the date of the letter transmitting this Notice of Violation (Notice).

Under the authority of Section 182 of the Atomic Energy Act, 42 U.S.C. 2232, any response provided shall be submitted under oath or affirmation.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. If personal privacy or proprietary information is necessary to provide an acceptable response, then please provide a bracketed copy of your response that identifies the information that should be protected and a redacted copy of your response that deletes such information. If you request withholding of such material, you must specifically identify the portions of your response that you seek to have withheld and provide in detail the bases for your claim of withholding (e.g., explain why the disclosure of information will create an unwarranted invasion of personal privacy or provide the information required by 10 CFR 2.790(b) to support a request for withholding confidential commercial or financial information). If safeguards information is necessary to provide an acceptable response, please provide the level of protection described in 10 CFR 73.21.

Dated at Atlanta, Georgia
this 5th day of September 1997

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NRC Resident Inspector
 U.S. Nuclear Regulatory Commission
 6745 N. Tallahassee Road
 Crystal River, FL 34428

*JXL concurred per
 phone message
 Per analysis
 9/15/97 JGW*

*SEE PREVIOUS PAGE FOR CONCURRENCE

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NR: Resident Inspector
 U.S. Nuclear Regulatory Commission
 6745 N. Tallahassee Road
 Crystal River, FL 34428

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