

APPENDIX

U. S. NUCLEAR REGULATORY COMMISSION  
REGION IV

NRC Inspection Report: 50-445/86-21  
50-446/86-18

Permits: CPPR-126  
CPPR-127

Dockets: 50-445  
50-446

Category: A2

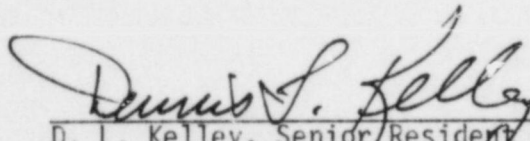
Applicant: Texas Utilities Electric Company (TUEC)  
Skyway Tower  
400 North Olive Street  
Lock Box 81  
Dallas, Texas 75201

Facility Name: Comanche Peak Steam Electric Station (CPSES), Units 1 & 2

Inspection At: Glen Rose, Texas

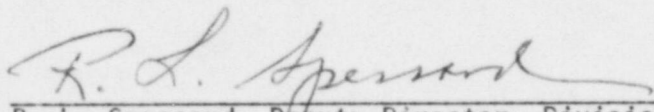
Inspection Conducted: July 1-31, 1986

Inspector:

  
D. L. Kelley, Senior Resident Reactor Inspector  
(SRRI), Region IV CPSES Group

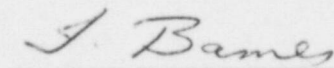
9/25/86  
Date

Reviewed  
By:

  
R. L. Spessard, Deputy Director, Division of  
Inspection Programs, Office of Inspection  
and Enforcement

9/22/86  
Date

Approved:

  
I. Barnes, Chief, Region IV CPSES Group

9/29/86  
Date

Inspection Summary

Inspection Conducted: July 1-31, 1986 (Report 50-445/86-21; 50-446/86-18)

Areas Inspected: Routine, unannounced inspection of (1) applicant actions on previous inspection findings; (2) calibration source recovery; (3) Unit 2 preoperational test procedure review; (4) plant tours; and (5) plant status.

Results: Within the five areas inspected, no violations or deviations were identified.

## DETAILS

### 1. Persons Contacted

#### Applicant Personnel

- \*A. B. Scott, Vice President, Nuclear Operations
- \*M. R. Blevins, Maintenance Superintendent
- C. Killough, Quality Surveillance Supervisor, Operations
- J. T. Merritt, Director, Construction/Director, Startup
- R. R. Wistrand, Operations Superintendent
- \*J. C. Smith, Operations Quality Assurance
- C. Cragg, STA/Operations Performance Evaluation Group
- \*M. J. Riggs, Operations Support Engineer
- F. L. Powers, Assistant Project Manager - Unit 1
- B. T. Lancaster, Administrative Superintendent
- \*G. M. McGrath, Special Project and Technical Support Lead
- \*T. L. Gosdin, Support Services
- \*E. Alercon, TUGCo Results Engineer
- \*W. I. Melton, TUGCo, Administrative Assistant to the Vice President, Nuclear Operations
- \*C. W. Audas, TUGCo Licensing

\*Denotes applicant representatives present during exit interview of August 4, 1986.

The NRC inspector also interviewed other applicant employees during this inspection period.

### 2. Applicant Actions on Previous Inspection Findings

- a. (Closed) Unresolved Item (445/8431-06): Maintenance Action Request processing deficiencies. During a followup inspection of this unresolved item in NRC Inspection 50-445/85-08, the SRRI verified that a part of the unresolved item had been corrected. The part that had not been corrected was the uninitialled crossouts in Startup Work Authorization (SWA) No. 21269. During this inspection period, the SRRI reviewed the SWA and noted that the crossouts had been initialled. No further applicant action is required on this item. This item is closed.
- b. (Closed) Open Item (445/8508-02): During inspection 50-445/84-31, a practice on mock maintenance activity was performed. During the parts requisition for maintenance drill, the site QA inspector noted a discrepancy between component (motor) nameplate horsepower rating and the drawing horsepower rating. The specific discrepancy was resolved; however, the NRC inspector was uncertain what actions were taken to insure that no other similar discrepancies existed. It was noted during further followup that a comparison was being made between all motor operated valve nameplates and the drawings, in



accordance with CPSES Problem Report PR 84-283 dated August 27, 1984. The comparison yielded 39 valve nameplates and the drawings. During inspection 50-445/85-08, the RRI noted that the corrective action had not been completed. Therefore, the item was not closed.

During this inspection, the SRRI again reviewed the applicant's corrective action and noted that the drawings have been revised to reflect the correct nameplate horsepower ratings. This item is closed.

### 3. Calibration Source Recovery

On May 27, 1986, while attempting to raise the 5,000 curies source to perform instrument calibration, the source elevator chain broke leaving the source at the bottom of its thirty foot well. Since the source elevator failed in the storage position, no radiological hazard existed.

A plan was devised to recover the source, repair the elevator, and re-install the source. A procedure was written for the retrieval and re-installation of the source. The procedure was reviewed and approved by the Station Operating Review Committee on July 10, 1986. Prior to the actual recovery, the SRRI reviewed the procedure (HPA-TP-86-1, "Retrieval and Loading of C 137 Well Source").

As a prerequisite, several dry runs were made in the maintenance building where a mock-up had been constructed.

On July 19, 1986, after a briefing, the source recovery was started. The SRRI witnessed the recovery. Briefly, the recovery method was to re-attach a cover to the source module and disengage it from the elevator platform using a thirty foot rod with a television camera to assist. Attachment and release were effected and the rod was pulled through the transfer pig until the source was fully housed. The access port in the pig was then plugged and the pig stored until the elevator could be repaired. The actual recovery process took approximately one hour. No problems were encountered during the recovery. At the conclusion of the recovery, the pocket dosimeters of all personnel in the recovery area were checked. The dosimeters showed no radiation exposure.

No violations or deviations were identified.

### 4. Preoperational Test Procedure Review

The NRC inspector reviewed and commented on one draft preoperational test procedure, 2 CP-PT-55-01, "Reactor Coolant System Hydrostatic Test." The procedure will be reviewed in its final form after approval by the Joint Test Group. When the preoperational test is performed, a brief review will be conducted of the latest revisions to note any changes that may affect the test results.

The procedure was reviewed with specific emphasis on the following:

- a. Management review.
- b. Format clearly defines testing to be performed.
- c. Test objectives are clearly stated.
- d. Prerequisites are identified.
- e. Special conditions (if any) are specified.
- f. Acceptance criteria are identified and requirements are specified for comparison of results with the acceptance criteria.
- g. Source of acceptance criterion is identified.
- h. Initial test conditions are specified.
- i. Reference to appropriate FSAR sections, drawings, specifications, and codes are included.
- j. Step-by-step instructions of sufficient detail are included to ensure that conduct of the test will result in valid conclusions.
- k. Provisions for documenting that required steps have been performed and space for recording data are included.
- l. Temporary circuit changes, installation of jumpers, and restoration of circuits after testing are properly documented.
- m. Independent verification of critical steps or parameters is addressed.

No violations or deviations were identified.

## 5. Plant Tours

During this reporting period, the SRRI conducted inspection tours of Unit 1. In addition to the general housekeeping activities and general cleanliness of the facility, specific attention was given to areas where safety-related equipment was installed and where activities were in progress involving safety-related equipment. These areas were inspected to ensure that:

- o Work in progress was being accomplished using approved procedures,
- o Special precautions for protection of equipment were implemented, and additional cleanliness requirements were being adhered to for maintenance and welding activities, and

- o Installed safety-related equipment and components were being protected and maintained to prevent damage and deterioration.

Also during these tours, the SRRI reviewed the control room and shift supervisor's log books. Key items in the log review were:

- o Plant status,
- o Changes in plant status,
- o Tests in progress, and
- o Documentation of problems which arise during operating shifts.

No violations or deviations were identified.

6. Plant Status as of July 31, 1986

- a. Unit No. 1 remains at 99% complete. Replacement of condenser tubing is in progress. A significant amount of pipe support rework is in progress and/or in the planning stage.
- b. Unit No. 2 is now 80% complete. Preoperational testing of safety-related systems has not commenced; however, test procedures are being generated.

7. Exit Interview

An exit interview was conducted August 4, 1986, with the applicant representatives identified in paragraph 1 of this appendix. During this interview, the operations SRRI summarized the scope and findings of the inspection. The applicant acknowledged the findings.