NUREG-0304 Vol. 22, No. 1

Regulatory and Technical Reports (Abstract Index Journal)

Compilation for First Quarter 1997 January – March

U.S. Nuclear Regulatory Commission

Office of Information Resources Management





9709030367 970630 PDR NUREG 0304 R PDR

AVAILABILITY NOTICE

Availability of Reference Materials Cited in NRC Publications

Most up to make the following sources:

- The NRC Fublic Document Room, 2120 L Street, NW., Lower Level, Washington, DC 20555-2001
- The Superintendent of Documents, U.S. Government Printing Office, P. O. Box 37082, Washington, DC 20402-9328
- 3. The National Technical Information Fervice, Springfield, VA 22161-0002

Although the listing that follows represents the majority of documents cited in NRC publications, it is not intended to be exhaustive.

Referenced documents available for inspection and copying for a fee from the NRC Public Document Room include NRC correspondence and internal NRC memoranda; NRC bulletins, circulars, information notices, inspection and investigation notices; licensee event reports; vendor reports and correspondence; Commission papers; and applicant and licensee documents and correspondence.

The following documents in the NUREG series are available for purchase from the Government Printing Office: formal NRC staff and contractor reports. NRC-sponsored conference proceedings, international agreement reports, grantee reports, and NRC booklets and brochures. Also available are regulatory guides, NRC regulations in the Code of Federal Regulations, and Nuclear Regulatory Commission Issuances.

Documents available from the National Technical Information Service include NUREG-series reports and technical reports prepared by other Federal agencies and reports prepared by the Atomic Energy Commission, forerunner agency to the Nuclear Regulatory Commission.

Docume vallable from public and special technical libraries include all open literature items, su as books, journal articles, and transactions. *Federal Register* notices, Federal and State is jislation, and congressional reports can usually be obtained from these libraries.

Documents such as theses, dissertations, foreign reports and translations, and non-NRC conference proceedings are available for purchase from the organization sponsoring the publication cited.

Single copies of NRC draft reports are available free, to the extent of supply, upon written request to the Office of Administration, Distribution and Mail Services Section, U.S. Nuclear Regulatory Commission, Washington DC 20555-0001.

Copies of industry codes and standards used in a substantive manner in the NRC regulatory process are maintained at the NRC Library. Two White Flint North, 11545 Rockville Pike, Rockville, MD 20852–2738, for use by the public. Codes and standards are usually copyrighted and may be purchased from the originating organization or, if they are American National Standards, from the American National Standards Institute, 1430 Broadway, New York, NY 10018–3308.

NUREG-0304 Vol. 22, No. 1

Regulatory and Technical Reports (Abstract Index Journal)

Compilation for First Quarter 1997 January – March

Date Published: June 1997

M. A. Sheehan, Project Manager

Publications Branch Office of Information and Resources Management U.S. Nuclear Regulatory Commission Washington, DC 20555-0001



Preface	۷
	ide) Tab
Main Citations and Abstracts Staff Reports Conference Proceedings Contractor Reports	1
Grant Reports International Agreement Reports	
Secondary Report Number Index	2
Personal Author Index	3
Subject Index	4
NRC Originating Organization Index (Staff Reports)	5
NRC Originating Organization Index (International Agreements)	6
NRC Contract Sponsor Index (Contractor Reports)	7
Contractor Index	8
International Organization Index	9
Licensed Facility Index	10

CONTENTS

PREFACE

This compilation consists of bibliographic data and abstracts for the formal regulatory and technical reports issued by the U.S. Nuclear Regulatory Commission (NRC) Staff and its contractors. It is NRC's intention to publish this compilation quarterly and to cumulate it annually. Your comments will be appreciated. Please send them to:

Publications Branch Office of Information Resources Management T-6 E7 U.S. Nuclear Receptory Commission Washington, D.C. 1955-0001

The main citations and abstracts in this compilation are listed in NUREG number order: NUREG-XXXX, NU-REG/CP-XXXX, NUREG/CR-XXXX, and NUREG/IA-XXXX. These precede the following indexes:

Secondary Report Number Index Personal Author Index Subject Index NRC Originating Organization Index (Staff Reports) NRC Originating Organization Index (International Agreements) NRC Contract Sponsor Index (Contractor Reports) Contractor Index International Organization Index Licensed Facility Index

A detailed explanation of the entries precedes each index.

The bibliographic elements of the main citations are the following:

Staff Report

NUREG-0808: MARK II CONTAINMENT PROGRAM EVALUATION AND ACCEPTANCE CRITERIA. ANDER-SON, C. J. Division of Safety Technology. August 1981. 90 pp. 8109140048. 09570:200.

Where the entries are (1) report number, (2) report title, (3) report author, (4) organizational unit of author, (5) date report was published. (6) number of pages in the report, (7) the NRC Document Control System accession number. (8) the microfiche address (for internal NRC use).

Conference Report

NUREG/CP-0017: EXECUTIVE SEMINAR ON THE FUTURE ROLE OF RISK ASSESSMENT AND RELIABIL-ITY ENGINEERING IN NUCLEAR REGULATION. JANERP, J.S. Argonne National Laboratory. May 1981. 141 pp. 8105280299. ANL-81-3. 08632:070.

Where the entries are (1) report number, (2) report title, (3) report author, (4) organization that compiled the proceedings, (5) date report was published, (6) number of pages in the report, (7) the NRC Document Control System accession number, (8) the report number of the originating organization, (9) the microfiche address (for NRC internal use).

Contractor Report

NUREG/CR-1556: STUDY OF ALTERNATE DECAY HEAT REMOVAL CONCEPTS FOR LIGHT WATER REACTORS-CURRENT SYSTEMS AND PROPOSED OPTIONS. BERRY, D.L.; BENNETT, P.R. Sandia Laboratories. May 1981. 100 pp. 8107010449. SAND80-0929. 08912:242.

Where the entries are (1) report number, (2) report title, (3) report authors. (4) organizational unit of authors or publisher, (5) date report was published, (6) number of pages in the report, (7) the NRC Document Control System accession number, (8) the report number of the originating organization (if given), (9) the micro-fiche address (for NRC internal use).

Grant Report

NUREG/GR-0013: APPLICATIONS OF A NEW MAGNETIC MONITORING TECHNIQUE TO IN SITU EVAL-UATION OF FATIQUE DAMAGE IN FERROUS COMPONENTS. JILES, D.C.; BINER, S.B.; GOVINDARAJU, M.; et al. Iowa State Univ., Ames, IA. June 1994, 41 pp. 9407250286, 80328:195.

Where the entries are(1) report number, (2) report title, (3) report authors, (4) organizational unit of authors or publisher, (5) date report was published. (6) number of pages in the report. (7) the NRC Document Control System accession number, (8) the report number of the originating organization (if given). (9) the micro-fiche address (for NRC internal use).

International Agreement Report

NUREG/IA-0001: ASSESSMENT OF TRAC-PD2 USING SUPER CANNON AND HDR EXPERIMENTAL DATA. NEUMANN, U. Kraftweek Union. August 1986. 223 pp. 8608270424. 37659:138.

Where the entries are(1) report number, (2) report title, (3) report author, (4) organizational unit of author, (5) date report was published, (6) number of pages in the report. (7) the NRC Document Control System accession number, (8) the report number of the originating organization (if given), and (9) the microfiche address (for NRC internal use).

The following abbreviations are used to identify the document status of a report:

- ADD addendum
- APP appendix
- DRFT draft
- ERR errata
 - N number
 - R revision
 - S supplement
 - V voluine

Availability of NRC Publications

Copies of NRC staff and contractor reports may be purchased either from the Government Printing Office (GPO) or from the National Technical Information Service, Springfield, Virginia 22161. To purchase documents from the GPO, send a check or money order, payable to the Superintendent of Documents, to the following address:

Superintendent of Documents U.S. Government Printing Office Post Office Box 37082 Washington, DC 20013-7082

You may charge any purchase to your GPO Deposit Account, MasterCard charge card, or VISA charge card by calling the GPO on (202) 512-2249 or (202) 512-2171. Non-U.S. customers must make payment in advance either by International Postal Money Order, payable to the Superintendent of Documents, or by draft on a United States or Canadian bank, payable to the Superintendent of Documents.

NRC Report Codes

The NUREG designation, NUREG-XXXX, indicates that the document is a formal NRC staff-generated report. Contractor-prepared formal NRC reports carry the report code NUREG/CR-XXXX. This type of identification replaces contractor-established codes such as ORNL/NUREG/TM-XXX and TREE-NUREG-XXXX, as well as various other numbers that could not be correlated with NRC sponsorship or the work being reported.

In addition to the NUREG and NUREG/CR codes, NUREG/CP is used for NRC-sponsored conference proceedings NUREG/GR is used for NRC grant reports, and NUREG/IA is used for international agreement reports.

All these report codes are controlled and assigned by the staff of the Publications Branch of the NRC Office of Information Resources Management.



Main Citations and Abstracts

The report listings in this compilation are arranged by report number, where NUREG-XXXX is an NRC staff-originated report, NUREG/CP-XXXX is an NRC-sponsored conference report, NUREG/CR-XXXX is an NRC contractor-prepared report, and NUREG/IA-XXXX is an international agreement report. The bibliographic information (see Preface for details) is followed by a brief abstract of this report.

NUREG-0040 V20 N03: LICENSEE CONTRACTOR AND VENDOR INSPECTION STATUS REPORT. Quarterly Report, July-September 1996 (White Book) * Office of Nuclear Reactor Regulation (Post 941001). January 1997. 147pp. 9702060133. 91659:001.

This periodical covers the results of inspections performed by the NRC's Special Inspection Branch, Vendor Inspection Section, that have been distributed to the inspected organizations during the period from July through September 1996.

NUREG-0040 V20 N04: LICENSEE CONTRACTOR AND VENDOR INSPECTION STATUS REPORT. Quarterly Report,October-December 1996.(White Book) * Office of Nuclear Reactor Regulation (Post 941001). March 1997. 154pp. 9703200250. 92191:001.

This periodical covers the results of inspections performed by the NRC's Special Inspection Branch, Vendor Inspection Section, that have been distributed to the inspected organizations during the period from October - December 1996.

NUREG-0304 V21 N03: REGULATORY AND TECHNICAL RE-PORTS (ABSTRACT INDEX JOURNAL). Compilation For Third Quarter 1996, July-September. * Office of Information Resources Management (Post 890205). February 1997. 41pp. 9703100239. 92020:309.

This journal includes all formal reports in the NUREG series prepared by the NRC staff and contractors; proceedings of conferences and workshops; as well as international agreement reports. The entries in this compilation are indexed for access by title and abstract, secondary report number, personal author, subject, NRC organization for staff and international agreements, contractor, international organization, and licensed facility.

NUREG-0540 V18 N11: TITLE LIST OF DOCUMENTS MADE PUBLICLY AVAILABLE.November 1-30, 1996. * Office of Information Resources Management (Post 890205). January 1997. 315pp. 9702070195. 91666:001.

This document is a monthly publication containing descriptions of information received and generated by the U.S. Nuclear Regulatory Commission (NRC). This information includes (1) docketed material associated with civilian nuclear power plants and other uses of radioactive materials, and (2) nondocketed material received and generated by NRC pertinent to its role as a regulatory agency. The following indexes are included: Personal Author, Corporate Source, Report Number, and Cross Reference of Enclosures to Principal Documents.

NUREG-0540 V18 N12: TITLE LIST OF DOCUMENTS MADE PUBLICLY AVAILABLE.December 1-31, 1996. * Office of Information Resources Management (Post 890205). March 1997. 298pp. 9704040207. 92332:001.

See NUREG-0540,V18,N11 abstract.

NUREG-0540 V19 N01: TITLE LIST OF DOCUMENTS MADE PUBLICLY AVAILABLE. January 1-31, 1997. * Office of Information Resources Management (Post 890205). March 1997. 325pp. 9704170086. 92517:001.

See NUREO-0540,V18,N11 abstract.

NUREG-0713 V17: OCCUPATIONAL RADIATION EXPOSURE AT COMMERICAL NUCLEAR POWER REACTORS AND OTHER FACILITIES,1995.Twenty-Eighth Annual Report. THOMAS,M.L. Division of Regulatory Applications (Post 941217). HAGEMEYER,D. Science Applications International Corp. (formerty Science Applications, Inc.). January 1997. 300pp. 9702190015. 91803:001.

This report summarizes the occupational exposure data that are maintained in the U.S. Nuclear Regulatory Commission's Radiation Exposure Information and Reporting System (REIRS). The bulk of the information contained in the report was compiled from the 1995 annual reports submitted by the classes of NRC licensees subject to the reporting requirements of 10 CFR 20.2206. Annual reports for 1995 were received from a total of 294 NRC licensees, of which 109 were operators of nuclear power reactors in commercial operation. Compilations of the reports submitted by the 294 licensees indicated that 142,518 individuals were monitored, 76,822 of whom received a measurable dose. The collective dose incurred by these individuals was 24,536 person-cSv (person-rem) which represents a 1% decrease from the 1994 value. The number of workers receiving a measurable dose also decreased, resulting in the average measurable dose of 0.32 cSv (rem) for 1995. The average measurable dose is defined to be the total collective dose divided by the number of workers receiving a measurable dose. The figures have been adjusted to account for transient reactor workers. In 1995, the annual collective dose per reactor for light water reactor licensees was 199 person-cSv (person-rem). This is the same value that was reported for 1994. The annual collective dose per reactor for boiling water reactors was 256 person-cSv (person-rem) and, for pressurized water reactors it was 170 person-cSv (person-rem). Analyses of transient worker data indicated that 17,153 individuals completed work assignments at two or more licensees during the monitoring year. The dose distributions are adjusted each year to account for the duplicate reporting of transient workers by multiple licensees. In 1995, the average measurable dose calculated from reported data was 0.26 cSv (rem). The corrected dose distribution resulted in an average measurable dose of 0.32 cSv (rem).

NUREG-0750 V44 ID1: INDEXES TO NUCLEAR REGULATORY COMMISSION ISSUANCES.July-September 1996. * Office of Information Resources Management (Post 890205). January 1997. 21pp. 9701160166. 91456:300.

Digests and indexes for issuances of the Commission, the Atomic Safety and Licensing Board Panel, the Administrative Law Judges, the Directors' Decisions, and the Decisions on Petitions for Rulemaking are presented.

NUREG-0750 V44 N05: NUCLEAR REGULATORY COMMISSION ISSUANCES FOR NOVEMBER 1996. Pages 229-314. * Office of Information Resources Management (Post 890205). January 1997. 95pp. 9703030170. 91940:135.

Legal issuances of the Commission, the Atomic Safety and Licensing Board Panel, the Administrative Law Judges, and NRC Program Offices are presented. NUREG-0750 V44 N06: NUCLEAR REGULATORY COMMISSION ISSUANCES FOR DECEMBER 1996. Pages 315-432. * Office of Information Resources Management (Post 890205). February 1997. 123pp. 9703170243. 92128:222.

See NUREG-0750, V44, N05 abstract.

NUREG-0750 V45 N01: NUCLEAR REGULATORY COMMISSION ISSUANCES FOR JANUARY 1997. Pages 1-47. * Office of Information Resources Management (Post 890205). March 1997. 53pp. 9704040210. 92330:268.

See NUREG-0750, V44, N05 abstract.

NUREG-0837 V16 N03: NRC TLD DIRECT RADIATION MONI-TORING NETWORK.Progress Report. July-September 1996. STRUCKMEYER,R. Region 1 (Post 820201). January 1997. 228pp, 9702060125. 91655:001.

This report provides the status and results of the NRC Thermoluminescent Dosimeter (TLD) Direct Radiation Monitoring Network. It presents the radiation levels measured in the vicinity of NRC licensed facilities throughout the country for the third guarter of 1996.

NUREG-0637 V16 N04: NRC TLD DIRECT RADIATION MONI-TORING NETWORK.Progress Report. October-December 1996. STRUCKMEYER,R. Region 1 (Post 820201). March 1997. 322pp. 9704170167. 92513:001.

This report provides the status and results of the NRC Thermoluminescent Dosimeter (TLD) Direct Radiation Monitoring Network. It presents the radiation levels measured in the vicinity of NRC licensed facilities throughout the country for the fourth guarter of 1996.

NUREG-0936 V15 N02: NRC REGULATORY AGENDA.Semiannual Report.July-December 1996. * Rules & Directives Review Branch (Post 920323). March 1997. 58pp. 9704080379. 92389:001.

The NRC Regulatory Agenda is a compilation of all rules on which the NRC has recently completed action, or has proposed action, or is considering action, and all petitions for rulemaking which have been received by the Commission and are pending disposition by the Commission. The Regulatory Agenda is updated and issued semiannually.

NUREG-1021 INT R08: OPERATOR LICENSING EXAMINATION STANDARDS FOR POWER REACTORS. * Office of Nuclear Reactor Regulation (Post 941001). January 1997. 460pp. 9703050343. 91953:001.

NUREG-1021, "Operator Licensing Examination Standards for Power Reactors," establishes the policies, proc. Jures, and practices for examining licensees and applicants for reactor operator and senior reactor operator licenses at power reactor facilities pursuant to Title 10, Part 55, of the Code of Federal Regulations (10 CFR Part 55). The examination standards are intended to assist NRC examiners and facility licensees to better understand the processes associated with initial and requalification examinations. The standards also ensure the equitable and consistent administration of examinations for all applicants. The standards are for guidance purposes and are not a substitute for the operator licensing regulations (i.e., 10 CFR Part 55), and they are subject to revision or other changes in internal operator licensing policy. This interim revision permits facility licensees to prepare their initial operator licensing examinations on a voluntary basis pending an amendment to 10 CFR Part 55 that will require facility preparation. The NRC intends to solicit comments on this revision during the rulemaking process and to issue a final Revision 8 in conjunction with the final rule.

NUREG-1100 V13: BUDGET ESTIMATES.Fiscal Year 1998. * Division of Budget & Analysis (Post 890205). February 1997. 148pp. 9702190053. 91802:045.

This report contains the fiscal year budget justification to Congress. The budget provides estimates for salaries and expenses and for the Office of the inspector General for fiscal year 1998.

- NUREG-1275 V12: OPERATING EXPERIENCE FEEDBACK REPORT Assessment Of Spent Fuel Cooling. IBARRA, J.G.; JONES,W.R.; LANIK,G.F.; et al. Division of Safety Programs (Post 870413). February 1997. 48pp. 9703170237. 92131:199. This report is an assessment of the likelihood and consequences of loss of spent fuel pool cooling in the nuclear power industry. A generic pressurized water reactor spent fuel pool configuration is developed, and a generic boiling water reactor spent fuel pool configuration is developed. Over twelve years of operational data is reviewed and assessed. Six site visits were conducted to gather specific information on spent fuel pool physical configuration, licensee practices, and licensee procedures. The regulations on spent fuel pools were reviewed. Independent engineering assessments on the spent fuel pool system were performed on the electrical system, instrumentation, heat loads, and radiation. An assessment on the risk of loss of spent fuel cooling was performed. The overall conclusions are that the typical plant may need improvements in spent fuel pool instrumentation, operator procedures and training, and configuration control.
- NUREG-1492: REGULATORY ANALYSIS ON CRITERIA FOR THE RELEASE OF PATIENTS ADMINISTERED RADIOACTIVE MATERIAL Final Report SCHNEIDER, S., MCGUIRE, S.A. Division of Regulatory Applications (Post 941217). February 1997. 79pp. 9703200259. 92191:156.

This regulatory analysis was developed to respond to three petitions for rulemaking to amend 10 CFR Parts 20 and 35 regarding release of patients administered radioactive material. The petitions requested revision of these regulations to remove the ambiguity that existed between the 1-mSv (0.1-rem) total effective dose equivalent (TEDE) public dose limit in Part 20, adopted in 1991, and the activity-based release limit in 10 CFR 35.75 that, in some instances, would permit release of individuals in excess of the current public dose limit. Three alternatives for resolution of the petitions were evaluated. Under Alternative 1, NRC would amend its patient release criteria in 10 CFR 35.75 to match the annual public dose limit in Part 20 of 1 mSv (0.1 rem) TEDE. Alternative 2 would maintain the status quo of using the activity-based release criteria currently found in 10 CFR 35.75. Under Alternative 3, the NRC would revise the release criteria in 10 CFR 35.75 to specify a dose limit of 5 mSv (0.5 rem) TEDE. The evaluation demonstrates that adoption of Alternative 1 would be considerably more expensive to the public compared to Alternative 2 (the status guo), primarily due to increased health care costs associated with more patents remaining in the hospital than under the current activity-based reguirements. The evaluation also demonstrates that adoption of the 5-mSv (0.5-rem) dose limit under Alternative 3 would result in a higher net value to the public compared to Alternative 2 (the status quo), primarily due to lower health care costs and the increased psychological benefits to patients and their families by permitting earlier release from the hospital. Based on this analysis, the decision was made that adoption of the 5-mSv (0.5-rem) TEDE limit 's consistent with the provisions in 10 CFR 20.1301(c), and the recommendations of the International Commission on Radiological Protection that an individual be allowed to receive annual doses up to 5 mSv (0.5 rem) TEDE under certain circumstances. Further, it no longer restricts patient release to a specific activity, and therefore, permits release of patients with activities that are greater than currently allowed. The primary benefit is in reduced hospital stays that provide emotional benefits to patients and their families, and result in lower health care costs.

NUREG-1508: FINAL ENVIRONMENTAL IMPACT STATEMENT TO CONSTRUCT AND OPERATE THE CROWNPOINT URANI-UM SOLUTION MINING PROJECT, CROWNPOINT, NEW MEXICO.Docket No. 40-8968. (Hydro Resources, Inc.) * Division of Waste Management (NMSS 940403). February 1997. 432pp. 9703200270. BLM NM010-93-02, 92192:001.

This Final Environmental Impact Statement (FEIS) addresses issuing a combined source and 11e(2) byproduct material license and minerals operating leases for Federal and Indian lands to Hydro Resources, Inc. (HRI). This action would authorize the company to conduct in situ leach uranium mining in McKinley County, New Mexico. Such mining would involve drilling wells to the ore bodies, then recirculating ground water fortified with dissolved oxygen and sodium bicarbonate to mobilize uranium minerals found in the rock. Uranium would then be removed from the aqueous mining solutions using ion exchange technology in processing plants located in three separate project areas. A central plant would provide drying and packaging equipment for yellow-cake production for the entire project. The FEIS was prepared by a joint interagency review group, including the U.S. Nuclear Regulatory Commission (NRC), the U.S. Bureau of Land Management (BLM) and the U.S. Bureau of Indian Affairs (BIA). This FEIS describes the staffs analyses concerning the evaluation of: (1) the purpose of and need for the proposed action; (2) alternatives to the proposed action; (3) the environmental resources that could be affected by the proposed action and alternatives; (4) the potential environmental consequences of the proposed action and alternatives; and (5) the economic costs and benefits associated with the proposed action. The evaluation is based on a comprehensive review of HRI's license application, environmental reports, related submittais, independent information sources, and written and oral comments received on the Draft Environmental Impact Statement. On the basis of its independent review, the staff concludes that the potential significant impacts of the proposed project can be mitigated, and that HRI should be issued a combined source and 11e(2) byproduct material license from NRC. and minerals operating leases from BLM and BIA.

NUREG-1532: FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS CORPORATION MOAB MILL.Source Material License No. 3UA-917.Docket No. 40-3453.(Atlas Corporation) FLIEGEL,M.; BRUMMETT,E.; IBRAHIM,A.; et al. Division of Waste Management (NMSS 940403). March 1997. 200pp. 9704100173. 92418:016.

This final Technical Evaluation Report (TER) summarizes the U.S. Nuclear Regulatory Commission staff's review of Atlas Corporation's proposed reclamation plan for its uranium mill tailings pile near Moab. Utah. The proposed reclamation would allow Atlas to (1) reclaim the tailings pile for permanent disposal and long-term custodial care by a government agency in its current location on the Moab site, (2) propare the site for closure, and (3) relinquish responsibility of the site after having its NRC Il cense terminated. The NRC staff concludes that, subject to license conditions identified in the TER, the proposed reclamation, which appear primarily in 10 CFR Part 40.

NUREG-1536: STANDARD REVIEW PLAN FOR DRY SPENT FUEL CTORAGE SYSTEMS. Final Report. * Office of Nuclear Material Safety & Safeguards. January 1997. 179pp. 9703130386. 92107:104.

The Standard Review Plan (SRP) for Dry Cask Storage Systems provides guidance to the Nuclear Regulatory Commission staff in the Spent Fuel Project Office for performing safety reviews of dry cask storage systems. The SRP is intended to ensure the quality and uniformity of the staff reviews and present a basis for the review scope and requirements. Part 72. Subpart B generally specifies the information needed in a license application for the independent storage of spent nuclear fuel and high level radioactive waste. Regulatory Guide 3.61, "Standard Format and Content for a Topical Safety Analysis Report for a Spent Fuel Dry Storage Cask" contains an outline of the specific information required by the staff. The SRP is divided into 14 sections which reflect the standard application format. Regulatory requirements, staff position, industry codes and standards, acceptance criteria, and other information are discussed. Comments, errors or omissions, and suggestions for improvement should be sent to the Director, Spent Fuel Project Office, U.S. Nuclear Regulatory Commission, DC 20555-0001.

NUREG-1545: EVALUATION CRITERIA FOR COMMUNICA-TIONS-RELATED CORRECTIVE ACTION PLANS. * Office of Nuclear Reactor Regulation (Post 941001). * Division of Systems Technology (Post 941217). February 1997. 69pp. 9703100227. 92020:240.

This document provides guidance and criteria for U.S. Nuclear Regulatory Commission (NRC) personnel to use in evaluating corrective action plans for nuclear power plant communications. The document begins by describing the purpose, scope, and applicability of the evaluation criteria. Next, it presents background information concerning the communications process, not causes of communication errors, and development and implementation of corrective actions. The document then defines specific criteria for evaluating the effectiveness of the corrective action plan, interview protocols, and an observation protocol related to communication processes. This document is intended only as guidance. It is not intended to have the effect of a regulation, and it does not establish any binding requirements or interpretations of NRC regulations.

NUREG-1562 DRFT FC: STANDARD REVIEW PLAN FOR APPLI-CATIONS FOR LICENSES TO DISTRIBUTE BYPRODUCT MA-TERIAL TO PERSONS EXEMPT FROM THE REQLIREMENTS FOR AN NRC LICENSE.10CFR Parts 30.14,30.15, 30.16,30.18,30.19 & 30.20. CAMPER.L.; RICH.T.; GREENE,S. Division of Industrial & Medical Nuclear Safety (Post 870729). January 1997. 82pp. 9702190056. 91802:274.

Exemptions from the requirements for an NRC license to persons who receive, possess, use, transfer, own, or acquire byproduct material in exempt distribution products are provided in 10 CFR Part 30, "Rules of General Applicability to Domestic Licensing of Byproduct Material." Exempt distribution products include silicon chips, electron tubes, resins, check sources, gunsights, and smoke detectors and are generally distributed by persons who have a specific license from the Commission authorizing such distribution to persons exempt from the requirements for an NRC license. This document provides assistance to applicarits and licensees in preparing license applications and describes the methods acceptable to NRC license reviewers in implementing the regulations and the techniques used by the reviewers in evaluating the applications to determine if the proposed exempt distribution activity is acceptable for licensing purposes. The guidance contained herein does not represent new or proposed regulatory requirements, and licensees will not be inspected against any portion of it. In accordance with NPC usage, the word "should" is used when discussing or reference ing NRC regulations. Additionally, regulatory compliance with all applicable regulations is not assured by licensees who adopt any portion of, or apply the principles described in, this guidance.

NUREG-1571: INFORMATION HANDBOOK ON INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS. RADDATZ,M.G.; WATERS.M.D. Office of Nuclear Material Safety & Safeguards. December 1996. 140pp. 9703030211. 91940:001.

In this information handbook, the staff of the U.S. Nuclear Regulatory Commission describes (1) background information regarding the licensing history of independent spent fuel storage installations (ISFSIs), (2) a discussion of the licensing process, (3) a description of all currently approved or certified models of dry cask storage systems (DCSSs), and (4) a description of sites currently storing spent fuel in an ISFSI. Storage of spent fuel at ISFSIs must be in accordance with the provisions of 10 OFR PART 72. The staff has provided this handbook for information purposes only. The accuracy of any information herein is not guaranteed. For verification or for more details, the reader should refer to the respective docket files for each DCSS and ISFSI site. The information in this handbook is current as of September 1, 1996.

Main Citations and Abstracts

NUREG-1574 DRFT FC: STANDARD REVIEW PLAN ON ANTITRUST.Draft Report For Comment LAMBE,W.M.; DAVIS,M.J. Office of Nuclear Reactor Regulation (Post 941001). January 1997, 31pp. 9702190059, 91804:328.

The Nuclear Regulatory Commission is issuing this draft Standard Review Plan to describe the procedure used to implement the antitrust review and enforcement prescribed in Sections 105 and 186 of the Atomic Energy Act of 1954, as amended. This draft SRP reflects current regulations and policy, and will be updated to reflect changes in NRC regulations.

NUREG-1577 DRFT FC: STANDARD REVIEW PLAN ON POWER REACTOR LICENSEE FINANCIAL QUALIFICATIONS AND DE-COMMISSIONING FUNDING ASSURANCE.Draft Report For Comment. WOOD,R.S. Office of Nuclear Reactor Regulation (Post 941001). January 1997. 20pp. 9702190062. 91809:334.

The Nuclear Regulatory Commission is issuing this draft Standard Review (SRP) to describe the process it uses to review the financial gualifications and methods of providing decommissioning funding assurance required of power reactor licensees. This draft SRP reflects current regulations and policy, and will be updated to reflect changes in NRC regulations.

NUREG/CP-0154: PROCEEDINGS OF THE CNRA/CSNI WORK-SHOP ON STEAM GENERATOR TUBE INTEGRITY IN NUCLE-AR POWER PLANTS. DIERCKS,D.R. Argonne National Laboretory. February 1997. 625pp. 9703100272. ANL-96/14. 92026:001.

An International Workshop on Steam Generator Tube Integrity in Nuclear Power Plants, sponsored by the Committee on Nuclear Regulatory Activities (CNRA) and the Committee on the Safety of Nuclear Installations (CSNI) of the OECD-NEA, was held at Oak Brook (suburban Chicago), Illinois, on October 30-November 2, 1995. The USNRC Office of Nuclear Regulatory Research served as host. The objective of the workshop was to provide a working forum for the exchange of information by contributing experts on current issues related to PWR steam generator tube integrity. One hundred persons from 15 countries attended the workshop, including 36 from regulatory and nuclear policy agencies, 28 from research and development laboratories, 18 from nuclear vendors and consulting firms, and 18 from electrical utilities. The workshop opened with a plenary session: the first part of the session covered international steam generator regulatory practices and issues, featuring speakers from regulatory bodies in Belgium, France, Japan, Spain, and the United States. In Part 2 of the plenary session, comprehensive technical overviews on steam generator tubing degradation, inspection, and integrity were presented by authorities in these fields from the United States, France, and Belgium. Parallel working sessions on the second and third days of the workshop then developed findings and recommendations in the areas of (1) tubing degradation, (2) tubing inspection, (3) tubing integrity, (4) preventative and corrective measures, and (5) operational aspects and risk analysis. On the final day of the workshop, the working-session facilitators presented summaries of their sessions to the workshop attendees.

NUREG/CP-0157 V01: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING.Plenary Session, High Burnup Fuel, Containment And Structural Aging, MONTELEONE, S. Brookhaven National Laboratory, January 1997, 357pp. 9703120266, 92077:003.

This three-volume report contains papers presented at the Twenty-Fourth Water Reactor Safety Information Meeting held at the Bethesda Marriott Hotel, Bethesda, Maryland, October 21-23, 1996. The papers are printed in the order of their presentation in each session and describe progress and results of programs in nuclear safety research conducted in this country and abroad. Foreign participation in the meeting included papers presented by researchers from Finland, France, Japan, Norway, Russia and the United Kingriom. The titles of the papers and the names of the authors have been updated and may differ from those that appeared in the final program of the meeting.

NUREG/CP-0157 V02: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING.Reactor Pressure Vessel Embrittlement And Thermal Annealing.Reactor Vessel Lower Head Integrity And Evaluation And Projection of Steam Generator tube... MONTELEONE,S. Brookhaven National Laboratory. February 1997. 444pp. 9703120276, 92075:001.

See NUREG/CP-0157, V01 abstract.

NUREG/CP-0157 V03: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING.PRA And HRA, And Probabilistic Seismic Hazard Assessment And Seismic Siting Criteria. MONTELEONE, S. Brookhaven National Laboratory. February 1997, 180pp. 9703120265. 92076:061.

See NUREG/CP-0157,V01 abstract.

NUREG/CR-4218 V12 N2: HEAVY-SECTION STEEL TECHNOL-OGY PROGRAM.Semiannual Progress Report For April 1995 Through September 1995. PENNELL,W.E. Oak Ridge National Laboratory. January 1997. 98pp. 9702070204. ORNL/TM-9593. 91667:001.

The Heavy-Section Steel Yechnology (HSST) Program is conducted for the Nuclear Regulatory Commission (NRC) by Oak Ridge National Laboratory (ORNL). The program's focus is on the development and validation of technology for the assessment of fracture-prevention margins in commercial nuclear reactor vessels. The HSST program is organized in seven tasks: (1) program management, (2) constraint effects analytical development and validation, (3) evaluation of cladding effects, (4) ductile-to-cleavage fracture-mode conversion, (5) fracture analysis methods development and application, (6) material property data and test methods, and (7) integration of results. The program tasks have been structured to place emphasis on resolution of fracture mechanics issues with near-term licensing significance. Resources to execute the research tasks are drawn from ORNL with sub-contract support from universities, and other research laboratories. Close contact is maintained with the sister Heavy- Section Steel Irradiation (HSSI) Program at ORNL and with related research programs both in the United States and abroad. This eport provides an overview of principal developments in each of the seven program tasks from April 1995 through September 1995.

NUREG/CR-4400 V00: DATA BASE ON DOSE REDUCTION PROJECTS FOR NUCLEAR POWER PLANTS. KHAN,T.A.; XIE,J.W. Brookhaven National Laboratory. January 1997. 167pp. 9702060168. BNL-NUREG-51934. 91659:149.

This is the sixth volume in a series of reports that provide information on dose reduction research and health physics technology for nuclear power plants. The information is taken from two of several databases maintained by Brookhaven National Laboratory's ALARA Center for the U.S. Nuclear Regulatory Commission. The research section of the report covers dose reduction projects that are in the experimental or development phase. It includes topics such as need for cost-effective measures to control radiation fields, the highly effective full-system decontamination, progress in addressing the increase in radiation fields upon switching from normal water chemistry to hydrogen water chemiatry in BWRs, addition of depleted zinc to reduce radiation fields, and cobalt free wear-resistant alloys. The section on health physics technology discusses dose reduction efforts that are in place or in the process of being implemented at nuclear power plants. A total of 67 new or updated projects are described. The appendix provides a complete listing of all the material in this area, including that from previous reports. The material is available through a fax machine from our ACEFAX on-line s/system. The procedure for accessing ACEFAX is also described.

NUREG/CR-6161 R01: A PILOT APPLICATION OF RISK-IN-FORMED METHODS TO ESTABLISH INSERVICE INSPEC-TION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION VO,T.V.: PHAN,H.K.; GORE,B.F.; et al. Battelle Memorial Institute, Pacific Northwest Laboratory, February 1997, 74pp, 9703100220, PNNL-9020, 92027;244.

As part of the Nondestructive Evaluation Reliability Program sponsored by the U.S. Nuclear Regulatory Commission, the Pacific Northwest National Laboratory has developed risk-informed approaches for inservice inspection plans of nuclear power plants. This method uses probabilistic risk assessment (PRA) results to identify and prioritize the most risk-important components for inspection. The Surry Nuclear Power Station Unit 1 was selected for pilot application of this methodology. This report, which incorporates more recent plant-specific information and improved risk-informed methodology and tools, is Revision 1 of the earlier report (NUREG/CR-6181). The methodology discussed in the original report is no longer current and a preferred methodology is presented in this Revision. This report, NUREG/OR-6181, Rev. 1, therefore supersedes the earlier NUREG/CR-6181 published in August 1994. The specific systems addressed in this report are the auxiliary feedwater, the low-pressure injection, and the reactor coolant systems. The results provide a risk-informed ranking of components within these systems.

NUREG/CR-6295: REASSESSMENT OF SELECTED FACTORS AFFECTING SITING OF NUCLEAR POWER PLANTS. DAVIS,R.E.; HANSON,A.L.; MUBAYI,V.; et al. Brookhaven National Laboratory. February 1997. 117pp. 9703170247. BNL-NUREG-52442, 92130:197.

Brookhaven National Laboratory has performed a series of probabilistic consequence assessment calculations for nuclear reactor sizing. This study takes into account recent insights into severe accident source terms and examines consequences in a risk based format consistent with the quantitative health objectives (QHOs) of the NRC's Safety Goal Policy. Simplified severe accident source terms developed in this study are based on the risk insights of NUREG-1150 and compared to those used in earlier studies, particularly the Sandia Siting Study. The results of the present study indicate that both the quantity of radioactivity released in a severe accident as well as the likelihood of a release are lower than those predicted in earlier studies. The accident risks using the simplified source terms are examined at a series of generic plant sites that vary in population distribution, meteorological characteristics, and exclusion boundary distances. Sensitivity calculations are performed to evaluate the effects of emergency protective action assumptions on the risk of prompt fatality and latent cancers fatality, and population relocation. The study finds that based on the new source terms, the prompt and latent fatality risks at all generic sites meet the QHOs of the NRC's Safety Goal Policy by margins ranging from one to more than three orders of magnitude.

NUREG/CR-6370: BLOCKAGE 2.5 USER'S MANUAL. RAC.D.V.; BRIDEAU,J.; et al. Science & Engineering Associates, Inc. BERNAHL,W. Software Edge, Inc., December 1996, 129pp. 9702060212, SEA963104010A:3, 91657:126.

The BLOCKAGE 2.5 Lode described in this User's Manual was developed by the United States Nuclear Regulatory Commission (NRC) as a tool to evaluate licensee compliance with NRC Bulletin 96-03. "Potential Plugging of Emergency Core Cooling Suction Strainers by Debris in Boiling Water Reactors". As such, BLOCKAGE 2.5 provides a generalized framework into which a user can input plant-specific and insulation-specific data for performing analyses in accordance with Regulatory Guide 1.82, Rev. 2. This user's manual describes the capabilities of BLOCKAGE 2.5 along with a description of the graphics user's interface provided for data entry. Each input/output dialog is described in detail along with special considerations related to developing and executing BLOCKAGE. Also, several sample problems are provided such that user can easily modify them to suit a particular plant of interest. The models used in BLOCKAGE 2.5 and their valication are presented in the accompanying NUREG/CR-6371. The BLOCKAGE models were designed to be parametric in tature, allowing the user flexibility to examine the impact of saveral modeling assumptions and to conduct sensitivity analyses. As a result, BLOCKAGE 2.5 results are known to be very sensitive to the user provided input. It is therefore strongly recommended that users become thoroughly familiar with BLOCKAGE models and their limitations as desoribed in NUREG/CR-6224.

NUREG/CR-6371: BLOCKAGE 2.5 REFERENCE MANUAL. SHAFFER,C.J.; BRIDEAU,J.; et al. Science & Engineering Associates, Inc. BERNAHL,W. Software Edge, Inc., December 1996. 163pp, 9702060227. SEA 96-3104-A:4. 91654:118.

The BLOCKAGE 2.5 code was developed by the United States Nuclear Regulatory Commission (NRC) as a tool to evaluate licensee compliance regarding the design of suction strainers for emergency core cooling system (ECCS) pumps in boiling water reactors (BWR) as required by NRC Bulletin 96-03, "Potential Plugging of Emergency Core Cooling Suction Strainers by Debris in Bolling Water Reactors." Science and Engineering Associates, Inc. (SEA) and Software Edge, Inc. (SE) developed this PC-based code. The instructions to effectively use this code to evaluate the potential of debris to sufficiently block a pump suction strainer such that a pump could lose NPSH margin was documented in a User's Manual [NRC, NUREG/CR-6370]. The Reference Manual contains additional information that supports the use of BLOCKAGE 2.5. It contains descriptions of the analytical models contained in the code, programmer guides illustrating the structure of the code, and summaries of coding verification and model validation exercises that were performed to ensure that the analytical models were correctly coded and applicable to the evaluation of BWR pump suction strainers. The BLOCKAGE code was developed by SEA and programmed in FORTRAN as a code that can be executed from the DOS level on a PC. A graphical users interface (GUI) was than developed by SEA to make BLOCKAGE easier to use and to provide graphical output capability. The GUI was programmed in the C language. The user has the option of executing BLOCKAGE 2.5 with the GUI or from the DOS level and the Users Manual provides instruction for both methods of execution.

NUREG/CR-6379: AN IMPROVED CORRELATION PROCEDURE FOR SUBSIZE AND FULL-SIZE CHARPY IMPACT SPECIMEN DATA. SOKOLOV, M.A.; ALEXANDER, D.J. Oak Ridge National Laboratory. March 1997. 302pp. 9704170020. ORNL-6888. 92516:001.

To examine the potential for using subsize Charpy specimens to evaluate the material properties of vessel materials for life extension, a study was conducted on the behavior of subsize impact specimens of five different geometries. Effects of notch depth, angle, and radius, as well as overall specimen dimensions were determined. Correlations of the transition temperature determined by the different subsize specimens as compared to full-size specimens were evaluated. A new procedure for transforming data from subsize specimens was developed.

NUREG/CR-6389: IPIRG-2 TASK 1 - PIPE SYSTEM EXPERI-MENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS.Final Report.September 1991 - November 1997: SCOTT.P.: OLSON.R.: MARSCHALL,C.; et al. Battelie Memorial Institute, Columbus Laboratories. February 1997: 363pp. 9703170239. BMI-2187. 92129:001.

This report presents the results from Task 1 of the Second International Piping Integrity Research Group (IPIRG-2) program. The rationale for and objective of Task 1 was to build on the results of the first IPIRG program by evaluating: (1) the fracture behavior of circumferentially cracked pipe subjected to more complex load histories, such as simulated seismic load histories; (2) cracks at geometric discontinuities, such as elbow girth welds; (3) smaller circumferential surface cracks, more typical of those considered in in-service flaw evaluations, subjected to dynamic, cyclic load histories; and (4) circumferential through-wall-cracked pipe subjected to dynamic, cyclic load histories. As a result of these Task 1 efforts, it was shown that: (1) the load-carrying capacity of a cracked pipe subjected to a simulated seismic load history is no worse than that of a cracked pipe subjected to the single-frequency excitation evaluated in IPTRG-1; (2) cracks at elbow girth weids can be adequately analyzed using methods previously developed for cracks in straight pipe; and (3) analysis methods previously developed and verfied for large circumferential surface cracks and circumferential through-wall cracks work equalty well for smaller cracks, even when subjected to more complex load histories.

NUREG/CR-6393: INTEGRATED SYSTEM VALIDATION: METH-ODOLOGY AND REVIEW CRITERIA. O'HARA, J.; STUBLER, W.; HIGGINS, J.; et al. Brookhaven National Laboratory. January 1997. 116pp. 9702060198. BNL-NUREG-52483. 91655:231.

The U.S. Nuclear Regulatory Commission reviews the human factors engineering (HFE) aspects of advanced nuclear power plant designs. In order to support the advanced reactor design certification reviews, the HFE Program Review Model was developed. The model describes the HFE program elements that are necessary and sufficient to develop an acceptable detailed design and provides the review criteria for their evaluation. One of the review elements is verification and validation. The purpose of this document is to discuss the detailed methodological considerations necessary for a review of an HFE integrated system validation. A conceptual approach, or paradigm, to integrated system validation is presented which identifies important validation principles and their relationships. The validation paradigm was used to identify the methodological aspects of the validation process that are needed to meet the general paradigm requirements. The methodology must support a logical and detensible interence to be made from validation tests to predicted integrated system performance under actual operating conditions. The validation paradigm is based upon four general forms of validity system representation, performance representation, test design and statistical conclusion validity. Validating an integrated system is based on establishing that these four types of validity are satisfied. Such assessments are made by reviewing the methodology used to conduct validation tests. Methodological factors relevant to each of the aspects of validity are discussed.

NUREG/CR-6397: RADIATION SAFETY CONCERNS FOR PREGNANT OR BREAST-FEEDING PATIENTS. The Positions Of The NCRP and The ICRP, MEINHOLD, C.B. Brookhaven Netional Laboratory, January 1997. 23pp. 9702070213. BNL-NUREG-52484. 91666:316.

For many years, protecting the fetus has been a concern of the National Council on Radiation Protection and Measurements (NCRP) and the International Commission on Radiological Protection (ICRP). Early recommendations focused on the possibility of a wide variety of detrimental developmental effects while later recommendations focused on the potential for severe mental retardation and/or reduction in the intelligence quotient (I.Q.) The latest recommendations also note that the risk of cancer for the fetus is probably two to three times greater per Sy than in the adult. For all these reasons, the NCRP and the ICRP have provided guidance to physicians on taking all reasonable steps to ascertain whether any woman requiring a radiological or nuclear medicine procedure is pregnant or nursing a child. The NCRP and the ICRP also advise the clinician to postpone such procedures until after delivery or cessation of nursing, if possible.

NUREG/CR-6400: HUMAN FACTORS ENGINEERING (HFE) IN-SIGHTS FOR ADVANCED REACTORS BASED UPON OPER-ATING EXPERIENCE. HIGGINS, J.; NASTA, K. Brookhaven National Laboratory January 1997. 61pp. 9704100181. BNL-NUREG-52485. 92416:244.

The NRC Human Factors Engineering Program Review Model (HFE PRM, NUREG-0711) was developed to support a design process review for advanced reactor design certification under 10CFR52. The HFE PRM defines ten fundamental elements of a human factors engineering program. An Operating Experience Review (OER) is one of these elements. The main purpose of an OER is to identify potential safety issues from operating plant experience and ensure that they are addressed in a new design. Broad-based experience reviews have typically been performed in the past by reactor designers. For the HFÉ PRM, the intent is to have a more focussed OER that concentrates on HFE issues or experience that would be relevant to the humansystem interface (HSI) design process for new advanced reactors. This document provides a detailed list of HFE-relevant operating experience pertinent to the HSI design process for advanced nuclear power plants. This document is intended to be used by NRC reviewers as part of the HFE PRM review process in determining the completeness of an OER performed by an applicant for advanced reactor design certification.

NUREG/CR-6414: PIPING BENCHMARK PROBLEMS FOR THE WESTINGHOUSE AP600 STANDARDIZED PLANT. BEZLER,P.: DEGRASSI,G.: BRAVERMAN,J.: et al. Brookhaven National Laboratory. January 1997. 300pp. 9702250218. BNL-NUREG-52487, 91871:001.

To satisfy the need for verification of the computer programs and modeling techniques that will be used to perform the final piping analyses for the Westinghouse AP600 Standardized Plant, three benchmark problems were developed. The problems are representative piping systems subjected to representative dynamic loads with solutions developed using the methods being proposed for analysis for the AP600 standard design. It will be required that the combined licensees demonstrate that their solutions to these problems are in agreement with the benchmark problem set.

NUREG/CR-6426 V01: DUCTILE FRACTURE TOUGHNESS OF MODIFIED A 302 GRADE B PLATE MATERIALS, DATA ANALY-SIS. MCCABE, D.E.; MANNESCHMIDT, E.; SWAIN, R.L. Oak Ridge National Laboratory. January 1997. 86pp. 9702190023. ORNL-6692. 91802:192.

The objective of this work was to develop ductile fracture toughness data in the form of J-R curves for modified A 302 grade B plate materials typical of those used in fabricating reactor pressure vessels. A previous experimental study at Materials Engineering Associates, Lanham, Maryland, on one particular heat of A 302 grade B plate showed decreasing J-R curvos with increased specimen thickness. This characteristic has not been observed in numerous tests made on the more recent production materials of A 533 grade B and A 508 class 2 pressure vessel steels. It was unknown if the departure from norm for the MEA material was a generic characteristic for all heats of A 302 grade B steels or just unique to that one particular plate.

NUREG/CR-6426 V02: DUCTILE FRACTURE TOUGHNES 3 OF MODIFIED A 302 GRADE B PLATE MATERIALS.Data Frecords. MCCABE, D.E.; MANNESCHMIDT, E.; SWAIN, R.L. Oak Ridge National Laboratory. February 1997. 600pp. 9703200279. ORNL-6892. 92194:001.

The objective of this work was to develop ductile fracture toughness data in the form of J-R curves for modified A 302 grade B plate materials typical of those used in fabricating reactor pressure vessels. A previous experimental study at Materials Engineering Associates (MEA) on one particular heat of A 302 grade B plate showed decreasing J-R curves with increased specimen thickness. This characteristic has not been observed in numerous tests made on the more recent production materials of A 533 grade B and A 508 class 2 pressure vessel steels. It was unknown if the departure from norm for the MEA matrial was a generic characteristic for all heats of A 302 grace B steels or just unique to that one particular plate. Seven heats of modified A 302 grade B steel and one heat of vintage A 533

4

grade B steel were provided to this project by the General Ele : tric Company of San Jose, California. All plates were tested for chemical content, tensile properties, Charpy transition temperature curves, drop-weight nil-ductility transition (NDT) temperature, and J-R curves. Tensile tests were made in the three principal orientations and at four temperatures, ranging from room temperature to 550 degrees F (288 degrees C). Charpy V-notch transition temperature curves were obtained in longitudinal, transverse, and short transverse orientations. J-R curves were made using four specimen sizes (1/2T, IT, 2T, and 4T). The fracture mechanics-based evaluation method covered three test orientations and three test temperatures [180, 400, and 550 de-grees F (82, 204, and 288 degrees C)]. However, the coverage of these variables was contingent upon the amount of material provided. Drop-weight NDT temperature was determined for the T-L orientation only. None of the seven heats of modified A 302 grade B showed size effects of any consequence on the J-R curve behavior. Crack orientation effects were present, but none were severe enough to be reported as atypical. A test temperature increase from 180 to 550 degress F (82 to 288 degrees C) produced the usual loss in J-R curve fracture toughness. Generic J-R curves and mathematical curve fits to the same were generated to represent each heat of material. Volume 1 deals with evaluation of data and discussion of technical findings. This volume (Volume 2) is a compilation of all data developed.

NUREG/CR-6432: CONTAINMENT PERFORMANCE OF PROTO-TYPICAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCIDENT CONDITIONS. KLAMERUS,E.W.; BOHN,M.P. Sandia National Laboratories. WESLEY,D.A.; et al. EQF Engineering Consultants (formerly EQE Engineering, Inc.). December 1996. 125pp. 9702060245. SAND96-2445. 91657:001.

In SECY-90-016, the NRC proposed a safety goal of a condi-tional containment failure prc. ability (CCFP) of 0.1 and the alternative acceptance criteria allowed for steel containments, which specifies that the stresses should not exceed ASME Level C allowables for severe accident pressures and temperatures. In this work, the need for an equivalent criterion for concrete containments was studied. Six surrogate containments were designed and analyzed in order to compare the margins between design pressure, pressure resulting in exceedance of Level C (or yield) stress limits, and ultimate pressure. For comparability, each containment has an identical internal volume and design pressure. Results from the analysis showed margins to yield are comparable and display a similar margin for both steel and concrete containments. In addition, the margin to tailure, although slightly higher in the steel containments, were also comparable. Finally, a CCFP for code design was determined based on general membrane behavior and imposing an upper bound severe accident curve developed in the DCH studies. The resulting CCFP's were less then 0.02 (or 2%) for all the surrogate containments studied, showing that these containment designs all achieved the NRC safety goal.

NUREG/CR-6446: FRACTURE TOUGHNESS EVALUATIONS OF TP304 STAINLESS STEEL PIPES. "UDLAND,D.L.; BRUST,F.W.; WILKOWSKI,G.M. Battelle Memorial Institute, Columbus Laboratories. February 1997. 116pp. 9703100252. BMI-2194. 92062:225.

In the IPIRG-1 program, the J-R curve calculated for a 16inch nominal diameter, Schedule 100 TP304 stainless steel (DP2-A8) surface-cracked pipe experiment (Experiment 1.3-3) was considerably lower than the guasi-static, monotonic J-R curve calculated from a C(T) specimen (A8-12a). The results from several related investigations conducted to determine the cause of the observed toughness difference are: (1) Chemical analyses on sections of Pipe DP2-A8 from several surfacecracked pipe and material property specimen fracture surfaces indicate that there are two distinct heats of material within Pipe DP2-A8 that differ in chemical composition. (2) SEN(T) specimen experimental results indicate that the toughness of a surface-cracked specimen is highly dependent on the depth of the initial crock. In addition, the J-R curves from the SEN(T) specimens closely match the J-R curve from the surface-cracked pipe experiment. (3) C(T) experimental results suggest that there is a large difference in the quasi-static, monotonic toughness between the two heats of DP2-A8, as well as a toughness degradation in the lower toughness heat of material (DP2-A8II) when loaded with a dynamic, cyclic (R = -0.3) loading history.

NUREG/CR-6452: THE SECOND INTERNATIONAL PIPING IN-TEGRITY RESEARCH GROUP (IPIRG-2) PROGRAM. Final Report. HOPPER, A.; WILKOWSKI,G.M.; SCOTT, P.; et al. Battelle Memorial Institute, Columbus Laboratories. March 1997. 292pp. 9704080384. BMI-2195. 92387:001.

The IPIRG-2 program was an international group program managed by the U.S. NRC and funded by organizations from 15 nations. The emphasis of the IPIRG-2 program was the development of data to verify fracture analyses for cracked pipes and fittings subjected to dynamic/cyclic load histories typical of seismic events. The scope included: (1) the study of more complex dynamic/cyclic load histories, i.e., multi-frequency, variable amplitude, simulated seismic excitations, than those considered in the IPIRG-1 program, (2) crack sizes more typical of those considered in Leak-Before-Break (LBB) and in-service flaw evaluations, (3) through-wall-crecked pipe experiments which can be used to validate LBB-type fracture analyses, (4) cracks in and around pipe fittings, such as elbows, and (5) laboratory specimen and separate effect pipe experiments to provide better insight into the effects of dynamic and cyclic load histories. Also undertaken were an uncertainty analysis to identify the issues most important for LBB or in-service flaw evaluations, updating computer codes and databases, the development and conduct of a series of round-robin analyses, and analyst's group meetings to provide a forum for nuclear piping experts from around the world to exchange information on the subject of pipe fracture technology.

NUREG/CR-6456: REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING, AND FEEDRING CRACKING AND WALL THINNING. SHAH,V.N.; WARE,A.G.; PORTER,A.M. Idaho National Engineering Laboratory. March 1997. 190pp. 9704170076. INEL-96/0089. 92531:091.

Review of industry efforts to manage ther.nal fatigue, flow-accelerated corrosion, and steam generallyr water hammer damage to Pressurized Water Reactor (PWH) feedwater nozzles, piping, and feedrings is presented in this report. The review includes an evaluation of design modifications, operating procedure changes, augmented inspection and monitoring programs, and mitigation, repair and replacement activities. Four specific actions were taken to perform the evaluation (a) review of field experience to identify trends of operating events; (b) review of the related technical literature; (c) visits to three PWR plants and a PWR vendor; and (d) solicitation of information from foreign utilities. Our assessment of field experience indicates the USNAC licensees have apparently taken sufficient action to minimize the feedwater nozzle cracking caused by thermal fatigue, wall thinning of J-tubes and feedwater piping. and steam generator water hammer in both top-feed and preheat steam generators. A major finding of this review is that the analysis, inspection, monitoring, mitigation, and replacement techniques have been developed for managing thermal fatigue and flow-accelerated corrosion damage to feedwater nozzles. piping, and feedrings. Adequate training and appropriate applications of these techniques would ensure effective management of this damage. Several PWR plant operators have been proactive in managing this damage.

NUREG/CR-6469: EXPERIMENTS TO INVESTIGATE DIRECT

CONTAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT. BLANCHAT,T.K.; PILCH,M.M.; ALLEN,M.D. Sandia National Laboratories. February 1997. 195pp. 9703170250. SAND96-2289. 92130:001.

The Surtsey Test Facility at Sandia National Laboratories (SNL) is used to perform scaled experiments for the Nuclear Regulatory Commission (NRC) that simulate High Pressure Melt Ejection (HPME) accidents in a nuclear power plant (NPP). These experiments are designed to investigate the effects of direct containment heating (DCH) phenomena on the containment load. In previous experiments, high-temperature, chemically reactive (thermitic) melt was ejected by high-pressure steam into a scale model of either the Zion or Surry NPP. The results from the Zion and Surry experiments were extrapolated to other Westinghouse plants. This report describes tests performed with Combustion Engineering plant geometries (in particular, Calvert Cliffs-like) and the impact of codispersed water as part of the overall DCH issue resolution. Integral effects tests were performed with a 1/10th scale model of the Calvert Cliffs NPP inside the Surtsey test vessel. The experiments investigated the effects of codispersal of water, steam, and molten core simulant materials on DCH loads under prototypic accident conditions and plant configurations. The results indicated that large amounts of coejected water reduced the DCH load by a small amount. Large amounts of debris were dispersed from the cavity to the upper dome (via the annular gap).

NUREG/CR-6474: PRELIMINARY PHENOMENA IDENTIFICA-TION AND RANKING TABLES (PIRT) FOR SBWR STARTUP STABILITY. ROHATGI,U.S.; CHENG,H.S.; KHAN,H.J.; et al. Brookhaven National Laboratory. March 1997. 81pp. 9703200285. BNL-NUREG-52504. 92196:001.

Phenomena identification and Ranking Tables (PIRT) have been developed for a start-up transient for SBWR. The information used for PIRT came from RAMONA-4B and TRACG analyses of the transient and from related small scale tests. The transient was divided into four distinct phases, namely, Sub-Cooled Core Heat-up, Subcooled Chimney, Saturated Chimney, and Power Ascension. The assessment criterion selected was Minimum Critical Power Ratio. The SBWR system was divided into ten components. A total of 35 distinct phenomena among the components were identified. The Phase I has 28 ranked phenomena with 17 low, 6 medium, and 5 high ranking. The Phase II has 39 ranked phenomena with 18 low, 13 medium and 8 high renking. The Phase III has 47 ranked phenomena with 22 low, 10 medium and 15 high ranking. The Phase IV has 46 ranked phenomena with 16 low, 12 medium and 18 high ranking.

NUREG/CR-6486: ASSESSMENT OF MODULAR CONSTRUC-TION FOR SAFETY-RELATED STRUCTURES AT ADVANCED NUCLEAR POWER PLANTS. BRAVERMAN,J.; MORANTE,R.; HOFMAYER,C. Brookhaven National Laboratory. March 1997. 201pp. 9704170099. BNL-NUREG-52520. 92518:014.

Modular construction techniques have been successfully used in a number of industries, both domestically and internationally. Recently, the use of structural modules has been proposed for advanced nuclear power plants. The objective in utilizing modular construction is to reduce the construction schedule, reduce construction costs, and improve the quality of construction. This report documents the results of a program which evaluated the proposed use of modular construction for safety-related structures in advanced nuclear power plant designs. The program included review of current modular construction technology, development of licensing review criteria for modular construction, and initial validation of currently available analytical techniques applied to concrete-filled steel structural modules. The program was conducted in three phases. The objective of the first phase was to identify the technical issues and the need for further study in order to support NRC licensing review activities. The two key findings were the need for supplementary review criteria to augment the Standard Review Plan and the need for verified design/analysis methodology for unique types of modules, such as the concrete-filled steel module. In the second phase of this program, Modular Construction Review Criteria were developed to provide guidance for licensing reviews. In the third phase, an analysis effort was conducted to determine if currently available finite element analysis techniques can be used to predict the response of concrete-filled steel modules.

NUREG/CR-6493: DOSES TO THE HAND DURING THE ADMIN-ISTRATION OF RADIOLABELED ANTIBODIES CONTAINING Y-90,TC-99M,I-131, AND LU-177, BARBER,D.E. Minnesota, Univ. of, Minneapolis, MN. CARSTEN,A.L.; KAURIN,D.G.L.; et al. Brookhaven National Laboratory, February 1997, 60pp. 9703100224, BNL-NUREG-52510, 92035:235.

Exposure of the hands of medical personnel administering radiolabeled antibodies (RABS) was evaluated on the basis of (a) observing and photo-documenting administration techniques, and (b) experimental data on doses to thermoluminescent dosimeters (TLDs) on fingers of phantom hands holding syringes, and on syringes, with radionuclides in the syringes in each case. Dose rate coefficients to the skin, if in contact with the syringe wall, were 89, 1.9, 3.8, and 0.41 uSv s(-1) averaged over 1 CM(2) at 7 mg CM(-2) per 37 MBg (1 mCi) for Y-90, Tc -99m, I-131, and Lu-177, respectively. When using Y-90 the importance of avoiding direct contact with syringes containing RABs and of using a beta-particle shield on the syringe was indicated. In using a syringe for injection, doses can best be approximated for the geometry studied by (a) wearing a finger dosimeter on the middle finger, toward the outside of the hand, on the hand operating the plunger, and (b) wearing finger dosimeters on the inner (palm) side of the finger on the hand that supports the syringe for energetic beta-particle emitters, such as Y-90 and Re-188.

NUREG/CR-6513 N01: NRC HIGH-LEVEL RADIOACTIVE WASTE MANAGEMENT PROGRAM ANNUAL PROGRESS REPORT: FISCAL YEAR 1996. SAGAR B. Center for Nuclear Waste Regulatory Analyses. January 1997. 317pp. 9704080389. 92385:001.

This annual status report for fiscal year 1996 documents technical work performed on ten key technical issues (KTIs) that are most important to performance of the proposed geologic repository at Yucca Mountain. This report was prepared jointly by the staff of the Nuclear Regulatory Commission (NRC) Division of Waste Management and the Center for Nuclear Waste Regulatory Analyses. The programmatic aspects of restructuring the NRC repository program in terms of KTIs is discussed and a brief summary of work accomplished is provided in Chapter 1. The other ten chapters provide a comprehensive summary of the work in each KTI. Discussions on probability of future volcanic activity and its consequences, impacts of structural deformation and seismicity, the nature of the near-field environment and its effects on container life and source term, flow and transport including effects of thermal loading, aspects of repository design, estimates of system performance, and activities related to the U.S. Environmental Protection Agency standard are provided.

NUREG/CR-6528: ENVIRONMENTAL ASSESSMENT PRO-POSED LICENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MASSACHUSETTS. MILLER, R.L.; EASTERLY, C.E.; LOMBARDI, D.A.; et al. Oak Ridge National Laboratory. February 1997. 88pp. 9703100266. 92020:152.

This Environmental Assessment was prepared to evaluate environmental issues associated with the renewal of NRC Licensee Nos. SMB-179 and SUB-1452 for facilities operated by Nuclear Metals, Inc. (NMI) in Concord Massachusetts. License renewal is needed to permit the continuation of NMI operations involving depleted and Garal uranium.

NUREG/CR-6529: VALIDATION OF TECTONIC MODELS FOR AN INTRAPLATE SEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA. TALWANI, P.; KELLOGG, J.N.; TRENKAMP, R. South Carolina, Univ. of, Columbia, SC. February 1997, 54pp, 9703100260, 92015, 299.

Although the average strain rate in intraplate settings is 2-3 orders of magnitude lower than at plate boundaries, there are pockets of high strain rates within intraplate regions. The results of a Global Positioning System survey near the location of current seismicity (and the interred location of the destructive 1886 Charleston, South Carouna earthquake) suggest that there is anomalous strain build-up occurring there. By reoccupying 1930 triangulation and 1980 GPS sites with six Trimble SST dual frequency receivers, a strain rate of 0.4 x 10(-7) yr(-1) was observed. At the 95% confidence level, this value is not significant; however, at a lower level of confidence (~ 85%) it is about two orders of magnitude greater than the background of 10(-9) to 10(-10) yr(-1). The direction of contraction inferred from the GPS survey 66 degrees \pm 11 degrees is in excellent agreement with the direction of the maximum horizontal stress (N 60 degrees E) in the area, suggesting that the observed strain rate is also real.



Secondary Report Number Index

This index lists, in alphabetical order, the performing organization-issued report codes for the NRC contractor and international agreement reports in this compilation. Each code is cross-referenced to the NUREG number for the report and to the 10-digit NRC Document Control System accession number.

SECONDARY REPORT NUMBER AEOD/E97-01 ANL-96/14 BIA E15-92-001 BUM NM010-93-02 BMI-2194 BMI-2194 BMI-2195 BNL-NUREG-51934 BNL-NUREG-51934 BNL-NUREG-52482 BNL-NUREG-52483 BNL-NUREG-52487	REPORT NUMBER NUREG/CR-6456 NUREG/CR-0154 NUREG-1508 NUREG/CR-6386 NUREG/CR-6446 NUREG/CR-6446 NUREG/CR-6462 NUREG/CR-6452 NUREG/CR-6393 NUREG/CR-6393 NUREG/CR-6397 NUREG/CR-6400 NUREG/CR-6414	SECONDARY REPORT NUMBER BNL-NUREG-52504 BNL-NUREG-52500 INEL-06/0089 NEA/CNRA/R(00)1 ORNL-6882 ORNL-6882 ORNL-6882 ORNL-6882 ORNL-6882 ORNL-6882 SAND96-2289 SAND96-2289 SAND96-2445 SEA 96-3104-A:4 SEA963104010A:3	REPORT NUMBER NUREG/CR-6474 NUREG/CR-6495 NUREG/CR-6495 NUREG/CR-6495 NUREG/CR-6495 NUREG/CR-6495 NUREG/CR-6426 V02 NUREG/CR-6426 V02 NUREG/CR-64191 V12 N2 NUREG/CR-6191 N01 NUREG/CR-6493 NUREG/CR-64971 NUREG/CR-6370	
---	---	---	---	--



Personal Author Index

This index lists the personal authors of NRC staff, contractor, and international agreement reports in alphabetical order. Each name is followed by the NUREG number and the title of the report(s) prepared by the author. If further information is needed, refer to the main citation by the NUREG number.

ALEXANDER D.J.

NUREG/CR-8379 AN IMPROVED CORRELATION PROCEDURE FOR SUBSIZE AND FULL-SIZE CHARPY IMPACT SPECIMEN DATA.

ALLEN.M.D.

NUREG/CR-6469 EXPERIMENTS TO INVESTIGATE DIRECT CON-TAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE GALVERT CLIFFS NUCLEAR POWER PLANT.

BARBER D.E.

NUREG/CR-6493 DOSES TO THE HAND DURING THE ADMINISTRA-TION OF RADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M, I-131, AND LU-177

RAUM J.V

NUREF/CR-6493 DOSES TO THE HAND DURING THE ADMINISTRA-TUN OF RADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M, I-131, AND LU-177

BERNAHL,W

NUREG/CR-6370: BLOCKAGE 2.5 USER'S MANUAL NUREG/CR-6371: BLOCKAGE 2.5 REFERENCE MANUAL

BEZLER,P

NUREG/CR-6414 PIPING BENCHMARK PROBLEMS FOR THE WES-TINGHOUSE AP600 STANDARDIZED PLANT

BLANCHAT,T.K.

NUREG/CR-6469 EXPERIMENTS TO INVESTIGATE DIRECT CON-TAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT

BOHN,M.P.

NUREG/CR-6433: CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCI-DENT CONDITIONS

BRAVERMAN.J.

NUREG/CR-6414 PIPING BENCHMARK PROBLEMS FOR THE WES

TINGHOUSE AP600 STANDARDIZED PLANT. NUREG/CR-8466 ASSESSMENT OF MODULAR CONSTRUCTION FOR SAFETY-RELATED STRUCTURES AT ADVANCED NUCLEAR POWER PLANTS

BRIDEAU.J.

NUREG/CR-6370: BLOCKAGE 2.5 USER'S MANUAL NUREG/CR-6371: BLOCKAGE 2.5 REFERENCE MANUAL

BROWNW

NUREG/CR-6393 INTEGRATED SYSTEM VALIDATION: METHODOLO-GY AND REVIEW CRITERIA.

BRUMMETT,E.

NUREG-1532 FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-PORATION MOAB MILL Source Material License No. SUA-917 Docket No. 40-3453 (Atlas Corporation)

BRUST.F.W

NUREG/CR-6446: FRACTURE TOUGHNESS EVALUATIONS OF TP304 STAINLESS STEEL PIPES.

CAMPER.L

-

NUREG-1562 DRFT FC: STANDARD REVIEW FLAN FOR APPLICA-TIONS FOR LICENSES TO DISTRIBUTE BYPRODUCT MATERIAL TO PERSONS EXEMPT FROM THE REQUIREMENTS FOR AN NRC LICENSE 10CFR Parts 30.14,30.15, 30.16,30.18,30.19 & 30.20.

CARSTEN,A.L.

NUREG/CR-6493: DOSES TO THE HAND DURING THE ADMINISTRA-TION OF RADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M, 1-131, AND LU-177

CHENG.H.S.

NUREG/CR-8474: PRELIMINARY PHENOMENA IDENTIFICATION AND RANKING TABLES (PIRT) FOR SBWR STARTUP STABILITY

DAVIS M.J.

NUREG-1574 DRFT FC: STANDARD REVIEW PLAN ON ANTITRUST.Draft Report For Comment.

DAVIS R.F.

NUREG/CR-6295: REASSESSMENT OF SELECTED FACTORS AF-FECTING SITING OF NUCLEAR POWER PLANTS.

DEGRASSI.G.

NUREG/CR-6414: PIPING BENCHMARK PROBLEMS FOR THE WES-TINGHOUSE AP600 STANDARDIZED PLANT

DIERCKS.D.R.

NUREG/CP-0154 PROCEEDINGS OF THE CNRA/CSNI WORKSHOP ON STEAM GENERATOR TUBE INTEGRITY IN NUCLEAR POWER PLANTS.

DOCTOR,S.R.

NUREG/CR-6181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION.

EASTERLY.C.E.

NUREG/CR-6528: ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MASSA-CHUSETTS

FLIEGEL M.

NUREG-1532 FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-PORATION MOAB MILL Source Material License No. SUA-917.Docket No. 40-3453.(Atias Corporation)

FRANCINI.R.

NUREG/CR-8389: IPIRG-2 TASK 1 - PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS, Final Report September 1991 - November 1995

GHADIALI.N.

NUREG/CR-8452: THE SECOND INTERNATIONAL PIPING INTEGRITY RESEARCH GROUP (IPIRG-2) PROGRAM.Final Report.

QORE E.F.

NUREG/CR-6181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION.

GREENE.S.

NUREG-1562 DRFT FC: STANDARD REVIEW PLAN FOR APPLICA-TIONS FOR LICENSES TO DISTRIBUTE BYPRODUCT MATERIAL TO PERSONS EXEMPT FROM THE REQUIREMENTS FOR AN NRC LICENSE 10CFR Parts 30.14,30.15, 30.16,30.18,30.19 & 30.20.

HAGEMEYER,D.

NUREG-0713 V17: OCCUPATIONAL RADIATION EXPOSURE AT COM-MERICAL NUCLEAR POWER REACTORS AND OTHER OTHER FACILITIES, 1995. Twenty-Eighth Annual Report.

.

Personal Author Index 14

HANSON,A.L.

NUREG/CR-6295: REASSESSMENT OF SELECTED FACTORS AF-FECTING SITING OF NUCLEAR POWER PLANTS.

HIGGINS J.

- NUREG/CR-6393 INTEGRATED SYSTEM VALIDATION. METHODOLO-GY AND REVIEW CRITERIA. NUREG/CR-6400: HUMAN FACTORS ENGINEERING (HFE) INSIGHTS
- FOR ADVANCED REACTORS BASED UPON OPERATING EXPERI-ENCE

HOFMAYER.C.

NUREG/CR-6486 ASSESSMENT OF MODULAR CONSTRUCTION FOR SAFETY-RELATED STRUCTURES AT ADVANCED NUCLEAR POWER PLANTS.

HOPPER,A

- NUREG/CR-6369 IPIRG-2 TASK 1 PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS Final Report September 1991 - November 1995. NUREG/CR-6452: THE SECOND INTERNATIONAL PIPING INTEGRITY
- RESEARCH GROUP (IPIRG-2) PROGRAM Final Report.

D.L.ARRABI

NUREG-1275 V12 OPERATING EXPERIENCE FEEDBACK REPORT Assessment Of Spent Fuel Cooling.

A MIHARB

NUREG-1532 FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-POFIATION MOAB MILL Source Material License No. SUA-917.Docket No. 40-3453 (Atlas Corporation)

JOHNSON T

NUREG-1532: FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-PORATION MOAB MILL Source Material License No. SUA-917.Docket No. 40-3453.(Atias Corporation)

JONES W.R.

NUREG-1275 V12 OPERATING EXPERIENCE FEEDBACK REPORT Assessment Of Spent Fuel Cooling.

JL STUS.P

NUREG-1532 FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-PORATION MOAB MILL Source Material License No. SUA-917.Docket to: 40-3453 (Atlas Corporation)

KAURIN, D.G.L.

NUREG/CR-6493 DOSES TO THE HAND DURING THE ADMINISTRA-TION OF RADIOLABELED ANTIBODIES CONTAINING Y-90,TC-99M,I-131, AND LU-177

KELLOGG, J.N.

NUREG/CR-8529. VALIDATION OF TECTONIC MODELS FOR AN IN-TRAPLATE SEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA

KHAN H.J.

NUREG/CR-6474: PRELIMINARY PHENOMENA IDENTIFICATION AND RANKING TABLES (PIRT) FOR SBWR STARTUP STABILITY

KHAN, T.A.

NUREG/CR-4409 V06: DATA BASE ON DOSE REDUCTION PROJECTS FOR NUCLEAR POWER PLANTS.

KILINSKI,T

NUREG/CR-6452: THE SECOND INTERNATIONAL PIPING INTEGRITY RESEARCH GROUP (IPIRG-2) PROGRAM.Final Report.

KLAMERUS, E.W

NUREG/CR-8433 CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCI-DENT CONDITIONS

KRISHNASWAMY,C.

NUREG/CR-6433: CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCI-DENT CONDITIONS.

LAMBE, W.M.

NUREG-1574 DRFT FC: STANDARD REVIEW PLAN ON ANTITRUST Draft Report For Commont.

LANIK.G.F.

NUREG-1275 V12: OPERATING EXPERIENCE FEEDBACK REPORT Assessment Of Spent Fuel Cooling.

LAYTON.M.

NUREG-1532 FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-PORATION MOAB MILL Source Material License No. SUA-917.Docket No. 40-3453.(Atlas Corporation)

LOMBARDIDA

NUREG/OR-6528 ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS.INC. CONCORD, MASSA-CHUSETTS.

MANNESCHMIDT.E.

NUREG/OR-6426 VO1: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS.DATA ANALYSIS. NUREG/CR-8426 V02: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS Data Records.

MARSCHALL C.

NUREG/CR-6989: IPIRG-2 TASK 1 - PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS Final Report September 1991 - November 1995.

MCCABE, D.E.

NUREG/CR-6426 V01: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS, DATA ANALYSIS. NUREG/CR-6426 V02: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS.Data Records.

MCGUIRE,S.A.

NUREG-1492: REGULATORY ANALYSIS ON ORITERIA FOR THE RE-ADMINISTERED LEASE PATIENTS OF RADIOACTIVE MATERIAL Final Report

MEINHOLD C.B.

NUREG/CR-6397: RADIATION SAFETY CONCERNS FOR PREGNANT OR BREAST-FEEDING PATIENTS The Positions Of The NORP And The ICRP

MILLER.R.L.

NUREG/CR-6528 ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MASSA-CHUSETTS.

MOHAN.R.

NUREG/CR-6452 THE SECOND INTERNATIONAL PIPING INTEGRITY RESEARCH GROUP (IPIRG-2) PROGRAM Final Report.

MONTELEONE,S.

NUREG/CP-0157 V01 PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING. Plenary Ses-

- sion, High Burnup Fuel, Containment And Structural Aging. NUREG/CP-0157 V02 PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING Reactor Pres-sure Vessel Embrittlement And Thermal Annealing,Reactor Vessel Lower Head Integrity And Evaluation And Projection of Steam Generator tube.
- NUREG/CP-0157 V03: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING.PRA And HRA, And Probabilistic Seismic Hazard Assessment And Seismic Siting Criteria.

MORANTE,R.

NUREG/CR-6488: ASSESSMENT OF MODULAR CONSTRUCTION FOR SAFETY-RELATED STRUCTURES AT ADVANCED NUCLEAR POWER PLANTS.

MUBAYI,V.

NUREG/OR-6295 REASSESSMENT OF SELECTED FACTORS AF-FECTING SITING OF NUCLEAR POWER PLANTS.

1

NASTAK.

NUREG/CR-6400: HUMAN FACTORS ENGINEERING (HFE) INSIGHTS FOR ADVANCED REACTORS BASED UPON OPERATING EXPERI-ENCE

NOURBAKHSH,H.P.

NUREG/CR-6295 REASSESSMENT OF SELECTED FACTORS AF-FECTING SITING OF NUCLEAR POWER PLANTS

O'HARA,J. NUREG/CR-6393: INTEGRATED SYSTEM VALIDATION: METHC XOLO-**GY AND REVIEW CRITERIA**

OLSON.R

NUREG/CR-0389 IPIRG-2 TASK 1 - PIPE SYSTEM EXPERIMENTS WITH CIRCUMPERENTIAL CRACKS IN STRAIGHT-FVPE

LOCATIONS Final Report September 1991 - November 1995. NUREG/OR-6452 THE SECOND INTERNATIONAL PIPING INTEGRITY RESEARCH GROUP (IPIRG-2) PROGRAM Final Report.

ORNSTEIN,H.L. NUREG-1275

OPERATING EXPERIENCE V12 PEEDBACK REPORT Assessment Of Spent Fuel Cooling.

PAULD.

NUREG/CR-6452 THE SECOND INTERNATIONAL PIPING INTEGRITY RESEARCH GROUP (IPIRG-2) PROGRAM Final Report.

PENNELL, W.E

NUREG/CR-4219 V12 N2. HEAVY-SECTION STEEL TECHNOLOGY PROGRAM.Semiannual Progress Report For April 1995 Through Septerriture 1005

PHAN,H.K.

NUREG/CR-6161 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION.

PILCH, M.M.

TAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT.

PORTER,A.M.

NUREG/CR-6456 REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING. AND FEEDRING ORACKING AND WALL THINNING.

PULLANI,S.V

NUREG-1275 V12 OPERATING EXPERIENCE FEEDBACK REPORT Assessment Of Spant Fuel Cooling

RADDATZ,M.G.

NUREG-1571: INFORMATION HANDBOOK ON INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS.

RAO, D.V.

NUREG/CR-6370: BLOCKAGE 2.5 USER'S MANUAL NUREG/CR-6371: BLOCKAGE 2.5 REFERENCE MANUAL

RICH,1

NUREG-1562 DRFT FC. STANDARD REVIEW PLAN FOR APPLICA-TIONS FOR LICENSES TO DISTRIBUTE BYPRODUCT MATERIAL TO PERSONS EXEMPT FROM THE REQUIREMENTS FOR AN NRC LICENSE 10CFR Parts 30.14.30.15, 30.16.30.18.30.19 & 30.20.

ROHATGI,U.S.

NUREG/CR-6474 PRELIMINARY PHENOMENA IDENTIFICATION AND RANKING TABLES (PIRT) FOR SEWR STARTUP STABILITY

ROM,D.

NUREG-1532: FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-PORATION MOAB MILL Source Material License No. SUA-917 Docket No. 40-3453 (Atlas Corporation)

RUDLAND.D.L.

- NUREG/CR-8389: IPIRG-2 TASK 1 PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS Final Report September 1991 - November 1995. NUREG/CR-6446: FRACTURE TOUGHNESS EVALUATIONS OF TP304
- STAINLESS STEEL PIPES. NUREG/CR-6452: THE SECOND INTERNATIONAL PIPING IN: 2GRITY
- RESEARCH GROUP (IPIRG-2) PROGRAM Final Report.

SAGAR.B.

NUREG/CR-6513 NO1: NRC HIGH-LEVEL RADIOACTIVE WASTE MAN-AGEMENT PROGRAM ANNUAL PROGRESS REPORT FISCAL YEAR 1008

SCHNEIDER,S.

NUREG-1492 REGULATORY ANALYSIS ON CRITERIA FOR THE RE-LEASE OF PATIENTS ADMINISTERED RADIOACTIVE MATERIAL Final Report

SCOTT.P.

- NUREG/OR-6388 IPIRG-2 TASK 1 PIPE SYSTEM EXPERIMENTS **CRACKS** IN CIRCUMPERENTIAL STRAIGHT-PIPE WITH LOCATIONS Final Report September 1991 - November 1995. NUREG/OR-6452: THE SECOND INTERNATIONAL PIPING INTEGRITY
- RESEARCH GROUP (IPIRG-2) PROGRAM Final Report.

SHAFFER.C.J.

NUREG/CR-6370: BLOCKAGE 2.5 USER'S MANUAL. NUREG/CR-6371: BLOCKAGE 2.5 REFERENCE MANUAL.

SHAH, V.N

NUREG/CR-6456 REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING, AND FEEDRING CRACKING AND WALL THINNING.

SIMOMEN, F.A.

NUREG/CR.6181 (201: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION

SOKOLOV, M.A.

NUREG/CR-6379 AN IMPROVED CORRELATION PROCEDURE FOR SUBSIZE AND FULL-SIZE CHARPY IMPACT SPECIMEN DATA.

SOUTO,F

NUREG/CR-6370: BLOCKAGE 2.5 USER'S MANUAL.

STRUCKMEYER,R.

NUREG-0837 V16 N03 NRC TLD DIRECT RADIATION MONITORING NETWORK Progress Report. July-September 1996. NUREG-0837 V16 N04: NRC TLD DIRECT RADIATION MONITORING NETWORK Progress Report. October-December 1996.

STUBLER.W.

NUREG/CR-6393. INTEGRATED SYSTEM VALIDATION: METHODOLO-GY AND REVIEW CRITERIA.

SWAIN,R.L.

NUREG/CR-6426 V01: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS, DATA ANALYSIS. NUREG/CR-6426 V02: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS.Data Records.

TAL WANI,P

NUREG/CR-6529: VALIDATION OF TECTONIC MODELS FOR AN IN-TRAPLATE SEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA

THOMAS.M.L

NUREG-0713 V17: OCCUPATIONAL RADIATION EXPOSURE AT COM-MERICAL NUCLEAR POWER REACTORS FACILITIES, 1995. Twenty-Eighth Annual Report. AND OTHER

TREITLER,I.E.

NUREG/CR-8528 ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MASSA-CHUSETTS

TRENKAMP,R

NUREG/CR-8529 VALIDATION OF TECTONIC MODELS FOR AN IN-TRAPLATE SEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA

VO.T.V

NUREG/CR-6181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION.

WANG, Y.K.

NUREG/CR-6414: PIPING BENCHMARK PROBLEMS FOR THE WES-TINGHOUSE AP600 STANDARDIZED PLANT.

WARE A.G.

NUREG/CR-6456: REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING, AND FEEDRING CRACKING AND WALL THINNING.

WATERS, M.D.

NUREG-1571: INFORMATION HANDBOOK ON INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS

Personal Author Index 16

WEBLEY, D.A.

NUREQ/CR-6433: CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCI-DENT CONDITIONS.

WILKOWSKI,G.M.

- NUREG/CR-6369: IPIRG-2 TASK 1 PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS Final Report September 1991 November 1995.
- NUREG/CR-6446 FRACTURE TOUGHNESS EVALUATIONS OF TP304 STAINLESS STEEL PIPES.
- NUREG/CR-6452 THE SECOND INTERNATIONAL PIPING INTEGRITY RESEARCH GROUP (IPIRG-2) PROGRAM.Final Report.

WINDOW,R.T.

.

NUREG/OR-6528 ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MASSA-CHUSETTS

WOLTERMAN,R.

WITH CIRCUMPERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS Final Report September 1991 - November 1995.

WOOD,R.S.

TOR LICENSEE FINANCIAL QUALIFICATIONS AND DECOMMIS-SIONING FUNDING ASSURANCE.Draft Report For Comment.

WULFF,W. NUREG/CR-6474: PRELIMINARY PHENOMENA IDENTIFICATION AND RANKING TABLES (PIRT) FOR SBWR STARTUP STABILITY.

XIE J.W

NUREG/OR-4409 V06: DATA BASE ON DOSE REDUCTION PROJECTS FOR NUCLEAR POWER PLANTS.

ZIMMERMAN, G.P.

NUREG/CR-8628: ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MASSA-CHUSETTS.

8

Subject Index

This index was developed from keywords and word strings in titles and abstracts. During this development period, there will be some redundancy, which will be removed later when a reasonable thesaurus has been developed through experience. Suggestions for improvements are welcome.

A 302 Grade B Steel Plate

NUREG/CR-8428 V01: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS DATA ANALYSIS. NUREG/OR-6426 V02: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS.Data Records.

ALARA

NEIBEG/CR-4409 V06: DATA BASE ON DOSE REDUCTION PROJECTS FOR NUCLEAR POWER PLANTS

Advanced Nuclear Power Plant

NUREG/CR-6486: ASSESSMENT OF MODULAR CONSTRUCTION FOR SAFETY-RELATED STRUCTURES AT ADVANCED NUCLEAR POWER PLANTS.

Antitrust

NUREG-1574 DRFT FC. STANDARD REVIEW PLAN ON ANTITRUST.Draft Report For Comment.

Atles Corporation

NUREG-1532 FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-PORATION MOAB MILL Source Material License No. SUA-917.Docket No. 40-3453 (Atlas Corporation)

BLOCKAGE 2

NUREG/CR-8570: BLOCKAGE 2.5 USER'S MANUAL NUREG/CR-8371: BLOCKAGE 2.5 REFERENCE MANUAL

Budget Estimate

NUREG-1100 V13: BUDGET ESTIMATES Fiscal Year 1998.

Byproduct Material

NUREG-1562 DRFT FC: STANDARD REVIEW PLAN FOR APPLICA TIONS FOR LICENSES TO DISTRIBUTE BYPRODUCT MATERIAL TO PERSONS EXEMPT FROM THE REQUIREMENTS FOR AN NRC LICENSE 10CFR Parts 30.14,30.15, 30.16,30.18,30.19 & 30.20.

CNRA/CSNI Workshop

NUREG/CP-0154 PROCEEDINGS OF THE ONRA/CSNI WORKSHOP ON STEAM GENERATOR TUBE INTEGRITY IN NUCLEAR POWER PLANTS.

Calvert Cliffs

NUREG/CR-6469 EXPERIMENTS TO INVESTIGATE DIRECT CON-TAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT

Certificate Of Compliance

NUREG-1571 INFORMATION HANDBOOK ON INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS.

Certification

NUREG/CR-6400: HUMAN FACTORS ENGINEERING (HFE) INSIGHTS FOR ADVANCED REACTORS BASED UPON OPERATING EXPERI-ENCE.

Charpy Impact

NUREG/OR-6379: AN IMPROVED CORRELATION PROCEDURE FOR SUBSIZE AND FULL-SIZE CHARPY IMPACT SPL TIMEN DATA

Circumferentiai Crack

NUREG/CR-6300 IPIRG-2 TASK 1 - PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS Final Report September 1991 - November 1995

Cladding Effect

NUREG/CR-4219 V12 Nº HEAVY-SECTION STEEL TECHNOLOGY PROGRAM.Semiannual Progress Report For April 1995 Through Septerribar 1995

Communication

NUREG-1545: EVALUATION CRITERIA FOR COMMUNICATIONS-RE-LATED CORRECTIVE ACTION PLANS.

Containment

NUREG/CP-0157 V01: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING. Plenary Session, High Burnup Fuel, Containment And Structural Aging

Containment Performance

NUREG/CR-6433: CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCI-DENT CONDITIONS

NUREG/CR-6393 INTEGRATED SYSTEM VALIDATION METHODOLO-

GY AND REVIEW CRITERIA.

Corrective Action Plan

NUREG-1545: EVALUATION CRITERIA FOR COMMUNICATIONS-RE-LATED CORRECTIVE ACTION PLANS.

Crownpoint

Control Room

NUREG-1508 FINAL ENVIRONMENTAL IMPACT STATEMENT TO CONSTRUCT AND OPERATE THE CROWNPOINT URANIUM SOLU-TION MINING PROJECT, CROWNPOINT, NEW MEXICO.Dooket No. 40-8968.(Hydro Resources, Inc.)

Crustal Strain

NUREG/CR-6529 VALIDATION OF TECTINIC MODELS FOR AN IN-TRAPLATE SEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA

Debris Generation

NUREG/CR-6370: BLOCKAGE 2.5 USER'S MANUAL NUREG/CR-6371: BLOCKAGE 2.6 REFERENCE MANUAL

Decommissioning

NUREG-1577 DRFT FC: STANDARD REVIEW PLAN ON POWER REAC-TOR LICENSEE FINANCIAL QUALIFICATIONS AND DECOMMIS-SIONING FUNDING ASSURANCE.Draft Report For Comment.

Design Criteria

NUREG/CR-6433 CONTAL-MENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCI-DENT CONDITIONS.

Direct Containment Heating NUREG/CR-6469: EXPERIMENTS TO INVESTIGATE DIRECT CON-TAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT

Dose Reduction

NUREG/CR-4409 VO6: DATA BASE ON DOSE REDUCTION PROJECTS FOR NUCLEAR POWER PLANTS.

Dosimetry

NUREG/CR-6483: DOSES TO THE HAND DURING THE ADMINISTRA-TION OF RADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M.I-131, AND LU-177.

Ductile Fracture

NUREG/CR-6428 V01: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE & PLATE MATERIALS, DATA ANALYSIS.

Dynamic Load

NUREG/CR-6414: PIPING BENCHMARK PROBLEMS FOR THE WES-TINGHOUSE AP600 STANDARDIZED PLANT

18 Subject Index

Embrittiement

NUREG/CP-0157 V02 PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR BAFETY INFORMATION MEETING Reactor Pressure Vessel Embrittlement And Thermal Annealing Reactor Vessel Lower Head Integrity And Evaluation And Projection of Steam Generator tube ...

Embryo

NUREG/OR-6997: RADIATION SAFETY CONCERNS FOR PREGNANT OR BREAST-FEEDING PATIENTS The Positions Of The NCRP And The ICRP

Environmental Assessment

NUREG/CR-8528 ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MASSA-CHUSETTS

Examination Standard

NUREG-1021 INT ROB OPERATOR LICENSING EXAMINATION STAND-ARDS FOR POWER REACTORS.

Exempt Distribution License

NUREG-1562 DRFT FC STANDARD REVIEW PLAN FOR APPLICA-TIONS FOR LICENSES TO DISTRIBUTE BYPRODUCT MATERIAL TO PERSONS EXEMPT FROM THE REQUIREMENTS FOR AN NRC LICENSE 10CFR Parts 30 14,30 15, 30 16,30 18,30 19 & 30.20.

Feedring Cracking

NUREG/CR-6456 REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING, AND FEEDRING CRACKING AND WALL THINNING.

Feedwater Nozzle

NUREG/CR-6456 REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING AND FEEDRING CRACKING AND WALL THINNING.

Final Environmental impact Statement

NUREG-1506 FINAL ENVIRONMENTAL IMPACT STATEMENT TO CONSTRUCT AND OPERATE THE CROWNPOINT URANIUM SOLU-TION MINING PROJECT, CROWNPOINT, NEW MEXICO.Docket No. 40-8968 (Hydro Resources, Inc.)

Financia Qualification

NUREG-1577 DRFT FC: STANDARD REVIEW PLAN ON POWER REAC-TOR LICENSEE FINANCIAL QUALIFICATIONS AND DECOMMIS SIONING FUNDING ASSURANCE Draft Report For Comment

- Fracture Mechanics NUREG/CP-0157 V02 PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING Reactor Pressure Vessel Embrittlement And Thermal Annealing,Reactor Vessel Lower Head Integrity And Evaluation And Projection of Steam Genera-
 - RESEARCH GROUP (IPIRG-2) PROGRAM.Final Report.

Fracture Toughness

- NUREG/CR-4219 V12 N2: HEAVY-SECTION STEEL TECHNOLOGY PROGRAM.Semiannual Progress Report For April 1995 Through September 1995
- NUREG/CR-6380: IPIRG-2 TASK 1 PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS Final Report September 1991 - November 1995. NUREG/CR-6426 V01: DUCTILE FRACTURE TOUGHNESS OF MODI-
- FIED A 302 GRADE B PLATE MATERIALS, DATA ANALYSIS. NUREG/CR-6426 V02: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS.Data Records. NUREG/CR-6446: FRACTURE TOUGHNESS EVALUATIONS OF TP304
- STAINLESS STEEL PIPES.

Fuel Reck

NUREG-1275 V12: OPERATING EXPERIENCE FEEDBACK REPORT Assessment Of Spent Fuel Cooling

Funding Assurance

NUREG-1577 DRFT FC: STANDARD REVIEW PLAN ON POWER REAC-TOR LICENSEE FINANCIAL QUALIFICATIONS AND DECOMMIS-SIONING FUNDING ASSURANCE Draft Report For Commant.

Geodetic Date

NUREG/CR-6329: VALIDATION OF TECTONIC MODELS FOR AN IN-TRAPLATE SEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA

Heavy-Section Steel Technology Program

NUREG/CR-4219 V12 N2 HEAVY-SECTION STEEL TECHNOLOGY PROGRAM.Semiannual Progress Report For April 1995 Through September 1995.

High Burnup Fuel

NUREG/OP-0157 V01: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING. Plenary Seesion, High Burnup Fuel, Containment And Structural Aging

High-Level Waste

NUREG/OR-6513 NO1: NRC HIGH-LEVEL RADIOACTIVE WASTE MAN-AGEMENT PROGRAM ANNUAL PROGRESS REPORT: FISCAL YEAR 1008

Human Factor

NUREG-1545: EVALUATION CRITERIA FOR COMMUNICATIONS-RE-LATED CORRECTIVE ACTION PLANS.

Human Factors Engineering

- NUREG/CR-6393 INTEGRATED SYSTEM VALIDATION: METHODOLO-GY AND REVIEW CRITERIA. NUREG/CR-8400: HUMAN FACTORS ENGINEERING (HFE) INSIGHTS
- FOR ADVANCED REACTORS BASED UPON OPERATING EXPERI-ENCE

IPIRG-2 Task 1

NUREG/CR-6389. IPIRG-2 TASK 1 - PIPE SYSTEM EXPERIMENTS CIRCUMFERENTIAL ORACKS IN STRAIGHT-PIPE WITH LOCATIONS Final Report September 1991 - November 1995.

ISFS

NUREG-1538: STANDARD REVIEW PLAN FOR DRY SPENT FUEL STORAGE SYSTEMS. Final Report.

Impact Testing

NUREG/CR-8379: AN IMPROVED CORRELATION PROCEDURE FOR SUBSIZE AND FULL-SIZE CHARPY IMPACT SPECIMEN DATA

In Situ Lepoh

NUREG-1508: FINAL ENVIRONMENTAL IMPACT STATEMENT TO CONSTRUCT AND OPERATE THE CROWNPOINT URANIUM SOLU-TION MINING PROJECT, CROWNPOINT, NEW MEXICO.Docket No. 40-8968.(Hydro Flesources, Inc.)

Independent Spent Fuel Storage Installation

NUREG-1571: INFORMATION HANDBOOK ON INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS

Inservice Inspection

NUREG/CR-8181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION.

Integrated System

NUREG/CR-6393 INTEGRATED SYSTEM VALIDATION METHODOLO-GY AND REVIEW CRITERIA.

Interim Storage

NUREG-1571 INFORMATION HANDBOOK ON INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS.

J-Integral

- NUREG/CR-6389 IPIRG-2 TASK 1 PIPE SYSTEM EXPERIMENTS W.TH CIRCUMFERENTIAL. CRACKS IN STRAIGHT-PIPE LOCATIONS Final Report September 1991 - November 1995. NUREG/CR-8452: THE SECOND INTERNATIONAL PIPING INTEGRITY
- RESEARCH GROUP (IPIRG-2) PROGRAM Final Report.

J-R Curve

- NUREG/CR-6426 V02: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS Data Records NUREG/CR-8446: FRACTURE TOUGHNESS EVALUATIONS OF TP304
- STAINLESS STEEL PIPES. NUREG/CR-6452: THE SECOND INTERNATIONAL PIPING INTEGRITY
- RESEARCH GROUP (IPIRG-2) PROGRAM Final Report.

Legal leauances

- NUREG-0750 V44 K01: INDEXES TO NUCLEAR REGULATORY COM-MISSION ISSUANCES.July-September 1996. NUREG-0750 V44 NO5: NUCLEAR REGULATORY COMMISSION IS-
- SUANCES FOR NOVEMBER 1996. Pages 229-314.

NUREG-0750 V44 NO6 NUCLEAR REGULATORY COMMISSION IS-SUANCES FOR DECEMBER 1998, Pages 315-432, NUREG-0750 V45 N01: NUCLEAR REGULATORY COMMISSION IS-

SUANCES FOR JANUARY 1997. Pages 1-47

License Renewal

NUREG/CR-6528 ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MASSA-CHUSETTS.

Mill Talling

NUREG-1532 FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-PORATION MOAB MILL Source Material License No. SUA-917.Docket No. 40-3453 (Atlas Corporation)

Modular Construction

NUREG/CR-6466: ASSESSMENT OF MODULAR CONSTRUCTION FOR SAFETY-RELATED STRUCTURES AT ADVANCED NUCLEAR POWER PLANTS

Nondestructive Evaluation

NUREG/CR-6181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION.

Nuclear Component

NUREG/CR-6181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION.

Nuclear Medicine

NUREG/CR-6493: DOSES TO THE HAND DURING THE ADMINISTRA-TION OF RADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M, I-191, AND LU-177

Nuclear Metals, Inc.

NUREG/CR-6528 ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS.INC. CONCORD, MASSA-CHUSETTS

Nuclear Power Plant

NUREG/CR-6295 REASSESSMENT OF SELECTED FACTORS AF-FECTING SITING OF NUCLEAR POWER PLANTS.

Occupational Radiation Exposure

NUREG-0713 V17. OCCUPATIONAL RADIATION EXPOSURE AT COM-MERICAL NUCLEAR POWER REACTORS AND OTHER OTHER FACILITIES, 1995 Twenty-Eighth Annual Report.

Operating Experience

NUREG-1275 V12: OPERATING EXPERIENCE FEEDBACK REPORT Assessment Of Spent Fuel Cooling. NUREG/CR-6400: HUMAN FACTORS ENGINEERING (HFE) INSIGHTS

FOR ADVANCED REACTORS BASED UPON OPERATING EXPERI-ENCE

Operator Licensing NUREG-1021 INT R08: OPERATOR LICENSING EXAMINATION STAND-ARDS FOR POWER REACTORS.

PWR

- NUREG/CR-6456: REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING.
- AND FEEDRING CRACKING AND WALL THINNING. NUREG/CR-8469 EXPERIMENTS TO INVESTIGATE DIRECT CON-TAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT.

Patient Release Criteria

NUREG-1492 REGULATORY ANALYSIS ON CRITERIA FOR THE RE-LEASE OF PATIENTS ADMINISTERED RADIOACTIVE MATERIAL Final Report.

Performance Assessment

NUREG/CR-8513 N01: NRC HIGH-LEVEL RADIOACTIVE WASTE MAN-AGEMENT PROGRAM ANNUAL PROGRESS REPORT: FISCAL YEAR 1996

Petitions For Rulemaking

NUREG-0936 V15 N02: NRC REGULATORY AGENDA Semiannual Report.July-December 1996.

Phenomicna Identification

NUREG/OR-6474: PRELIMINARY PHENOMENA IDENTIFICATION AND RANKING TABLES (PIRT) FOR SEWR STARTUP STABILITY.

- NUREG/OR-6446 FRACTURE TOUGHNESS EVALUATIONS OF TP304
- STAINLESS STEEL PIPES. NUREG/CR-8452: THE SECOND INTERNATIONAL PIPING INTEGRITY RESEARCH GROUP (IPIRG-2) PROGRAM Final Report.

Pipe System

NUREG/CR-6089: IPIRG-2 TASK 1 - PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS.Final Report.September 1991 - November 1995.

Piping

NUREG/CR-6414: PIPING BENCHMARK PROBLEMS FOR THE YES-

TINGHOUSE AP000 STANDARDIZED PLANT. NUREG/CR-8456: REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING, AND FEEDRING CRACKING AND WALL THINNING.

Piping System

NUREG/CR-6181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION.

Plate Material

NUREG/CR-6426 VO2: DUCTILE FRACTURE TOUGHNUSS OF MODI-FIED A 302 GRADE B PLATE MATERIALS.Data Records.

Pregnant Women

NUREG/CR-6397: RADIATION SAFETY CONCERNS FOR PREGNANT OR BREAST-FEEDING PATIENTS The Positions Of The NORP And The ICFIF

Pressure Vessel

NUREG/CR-8379: AN IMPROVED CORRELATION PROCEDURE FOR SUBSIZE AND FULL-SIZE CHARPY IMPACT SPECIMEN DATA.

Pressurized Water Reactor

NUREG/CR-6466: REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING, AND FEEDRING CRACKING AND WALL THINNING. NUREG/CR-8480: EXPERIMENTS TO INVESTIGATE DIRECT CON-

TAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT.

Radiation

NUREG/CR-6397: RADIATION SAFETY CONCERNO FOR PREGNANT OR BREAST-FEEDING PATIENTS. The Positions Of The NCRP And The ICRP

Radiation Dose

NUREG/CR-6493 DOSES TO THE HAND DURING THE ADMINISTRA-TION OF RADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M, I-131, AND LU-177

Radiation Protection

NUREG/CR-4409 V06: DATA BASE ON DOSE REDUCTION PROJECTS FOR NUCLEAR POWER PLANTS.

Radioactive Material

NUREG-1492 REGULATORY ANALYSIS ON CRITERIA FOR THE RE-LEASE OF PATIENTS ADMINISTERED RADIOACTIVE MATERIAL Final Report

Radiolabeled Antibodies

NUREG/CR-6493: DOSES TO THE HAND DURING THE ADMINISTRA-TION OF FIADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M, I-131, AND LU-177

Reactor Accident

NUREG/OR-6295. REASSESSMENT OF SELECTED FACTORS AF-FECTING SITING OF NUCLEAR POWER PLANTS.

Reactor Component

NUREG/OP-0157 V02 PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETINC Reactor Pres-sure Vessel Embrittlement And Thermal Annealing,Reactor Vessel Lower Head Integrity And Evaluation And Projection of Steam Generator tube.

Subject Index 20

NUREG/CR-6400. HUMAN FACTORS ENGINEERING (HFE) INSIGHTS FOR ADVANCED REACTORS BASED UPON OPERATING EXPERI-ENCE

Reactor Operator

NUREG/OR-6989 INTEGRATED SYSTEM VALIDATION METHODOLO-GY AND REVIEW CRITERIA

Reactor Pressure Vessel NUREG/OR-6426 V01: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS DATA ANALYSIS

Reactor Selety

- NUREG/OP-0167 V01: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING Plenary See
- Bion, High Burnup Fuel, Containment And Structural Aging, NUREG/CP-0157 VO2: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING Reactor Pres-sure Vessel Embrittiement And Thermal Annealing,Reactor Vessel Lower Head Integrity And Evaluation And Projection of Steam Genera-
- And Probabilistic Selamic Hazard Assessment And Selamic Siting Crite-
- NUREG/CR-6295 REASSESSMENT OF SELECTED FACTORS AF FECTING SITING OF NUCLEAR POWER PLANTS.

Reciemation Plan

NUREG-1532 FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-PORATION MOAB MILL Source Material License No. SUA-917.Docket No. 40-3453 (Atlas Corporation)

Regulatory Agenda

NUREG-0936 V15 N02: NRC REGULATORY AGENDA.Semiannual Report.July-December 1996.

Regulatory Analysis

NUREG-1482: REGULATORY ANALYSIS ON CRITERIA SOR THE RE-LEASE OF PATIENTS ADMINISTERED BADIOACTIVE RADIOACTIVE MATERIAL Final Report

Regulatory And Technical Report

NUREG-0304 V21 N03 REGULATORY AND TECHNICAL REPORTS (ABSTRACT INDEX JOURNAL). Compliation For Third Quarter 1996, July September

Repository Design

NUREG/OR-6513 NO1: NRC HIGH-LEVEL RADIOACTIVE WASTE MAN-AGEMENT PROGRAM ANNUAL PROGRESS REPORT FISCAL YEAR 1006

Rules

NUREG-0938 V15 N02 NRC REGULATORY AGENDA.Semiannual Report July-December 1996.

SBWR

NUREG/CR-6474 PRELIMITARY PHENOMENA IDENTIFICATION AND RANKING TABLES (PIRT) FOR SBWR STARTUP STABILITY.

Safety-Related Structure

NUREG/CR-6486: ASSESSMENT OF MODULAR CONSTRUCTION FOR SAFETY-RELATED STRUCTURES AT ADVANCED NUCLEAR POWER PLANTS.

Salamic Event

NUREG/CP-0157 V03. PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING PRA And HRA. And Probabilistic Seismic Hazard Assessment And Seismic Siting Critene.

Salamic Zone

NUREG/CR-6529. VALIDATION OF TECTONIC MODELS FOR AN IN-TRAPLATE SEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA.

Severe Accident

- NUREG/CR-6433 CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCI-DENT CONDITIONS. NUREG/OR-8489 EXPERIMENTS TO INVESTIGATE DIRECT CON-
- TAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT.

Simplified Boiling-Water Reactor

NUREG/CR-6474: PRELIMINARY PHENOMENA IDENTIFICATION AND RANKING TABLES (PIRT) FOR SEWR STARTUP STABILITY

Site Characterization

NUREG/CP-0157 V03: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING PRA And HRA. And Probabilistic Seismic Hazard Assessment And Seismic Siting Criteria.

Site Selection

NUREG/CR-6295 REASSESSMENT OF SELECTED FACTORS AF-FECTING SITING OF NUCLEAR POWER PLANTS.

Solution Mine

NUREG-1508 FINAL ENVIRONMENTAL IMPACT STATEMENT TO CONSTRUCT AND OPERATE THE CROWNPOINT URANIUM SOLU-TION MINING PROJECT, CROWNPOINT, NEW MEXICO.Docket No. 40-8968.(Hydro Resources, Inc.)

Scent Fuel

- NUREG-1275 EXPERIENCE FEEDBACK V12: **OPERATING** REPORT Assessment Of Spent Fuel Cooling, NUREG-1536: STANDARD REVIEW PLAN FOR DRY SPENT FUEL
- STORAGE SYSTEMS. Final Report.

Standard Review Plan

- NUREG-1536: STANDARD REVIEW PLAN FOR DRY SPENT FUEL
- STORAGE SYSTEMS Final Report. NUREG-1562 DRFT FC: STANDARD REVIEW PLAN FOR APPLICA-TIONS FOR LICENSES TO DISTRIBUTE BYPRODUCT MATERIAL TO PERSONS EXEMPT FROM THE REQUIREMENTS FOR AN NRC ICENSE 10CFR Parts 30.14.30.15, 30.16.30.18,30.19 & 30.20 IREG-1574 DRFT FC STANDARD REVIEW PLA
- NUREG-1574 DRFT FC. STANDARD REVIEW PLAN ON ANTITAUST.Draft Report For Comment NUREG-1577 DRFT FC: STANDARD REVIEW PLAN ON POWER REAC-TOR LICENSEE FINANCIAL QUALIFICATIONS AND DECOMMIS-SIONING FUNDING ASSURANCE.Draft Report For Comment.

Startup Stability

NUREG/CR-6474: PRELIMINARY PHENOMENA IDENTIFICATION AND RANKING TABLES (PIRT) FOR SEWR STARTUP STABILITY

Steam Generator

- NUREG/CP-0154. PROCEEDINGS OF THE CNRA/CSNI WORKSHOP ON STEAM GENERATOR TUBE INTEGRITY IN NUCLEAR POWER PLANTS
- NUREG/CR-4409 VOB: DATA BASE ON DOSE REDUCTION PROJECTS FOR NUCLEAR POWER PLANTS.

Storage Cask

- NUREG-1536: STANDARD REVIEW PLAN FOR DRY SPENT FUEL STORAGE SYSTEMS, Final Report. NUREG-1571: INFORMATION HANDBOOK ON INDEPENDENT SPENT
- FUEL STORAGE INSTALLATIONS.

Stress Corrosion Cracking

NUREG/OP-0154: PROCEEDINGS OF THE CNRA/CSNI WORKSHOP ON STEAM GENERATOR TUBE INTEGRITY IN NUCLEAR POWER PLANTS

Structurel Aging

NUREG/CP-0157 V01: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING. Plenery Session, High Burnup Fuel, Containment And Structural Aging.

Subsize Specimen

NUREG/OR-6379: AN IMPROVED CORRELATION PROCEDURE FOR SUBSIZE AND FULL-SIZE CHARPY IMPACT SPECIMEN DATA.

Suction Strainer

NUREG/CR-6370: BLOCKAGE 2.5 USER'S MANUAL NUREG/CR-6371: BLOCKAGE 2.5 REFERENCE MANUAL

Surface Crack

NUREG/OR-6452 THE SECOND INTERNATIONAL PIPING INTEGRITY RESEARCH GROUP (IPIRG-2) PROGRAM.Final Report.

TLD

NUREG-0837 V16 N03: NRC TLD DIRECT RADIATION MONITORING NETWORK Progress Report July-September 1996. NUREG-0837 V16 N04: NRC TLD DIRECT RADIATION MONITORING NETWORK.Progress Report. October-December 1996.

ä

4

8

TP304 Stainises Steel

NUREG/OR-6446 FRACTURE TOUGHNESS EVALUATIONS OF TP304 STAINLESS STEEL PIPES.

Tectoric Model

NUREG/CR-6529 VALIDATION OF TECTONIC MODELS FOR AN IN-TRAPLATE BEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA

Therapeutic Administration

NUREG-1492 REGULATORY ANALYSIS ON CRITERIA FOR THE RE-LEASE OF PATIENTS ADMINISTERED RADIOACTIVE MATERIAL Final Report

Thermoluminescent Dosimeter NUREG-0637 V16 N03: NRC TLD DIRECT RADIATION MONITORING NETWORK Progress Report July-September 1996. NUREG-0837 V16 N04: NRC TLD DIRECT RADIATION MONITORING NETWORK Progress Report. October-December 1996

THis List

- NUREG-0540 V18 N11: TITLE LIST OF DOCUMENTS MADE PUBLICLY AVAILABLE November 1-30, 1996. NUREG-0540 V18 N12: TITLE LIST OF DOCUMENTS MADE PUBLICLY
- AVAILABLE December 1-31, 1996. NUREG-0540 V18 NO1: TITLE LIST OF DOCUMENTS MADE PUBLICLY
- AVAILABLE, January 1-31, 1997.

Tube Integrity NUREG/CP-0154 PROCEEDINGS OF THE ONRA/CSNI WORKSHOP ON STEAM GENERATOR TUBE INTEGRITY IN NUCLEAR POWER PLANTS.

Uranium

NUREG-1532 FINAL TECHNICAL EVALUATION REPORT FOR THE PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS COR-

PORATION MOAB MILL Source Materiel License No. SUA-917.Docket No. 40-3453 (Atlas Corporation)

NUREG/OR-6528 ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, ASSA-CHUSETTS.

Vendor Inspection

- NUREG-0040 V20 N03: LICENSEE CONTRACTOR AND VENDOR IN-SPECTION STATUS REPORT. Quarterly Report.July-September 1996.(White Book)
- NUREG-0040 V20 N04: LICENSEE CONTRACTOR AND VENDOR IN-SPECTION STATUS REPORT. Quarterly Report.October-December 1996.(White Book)

Wall Thinning

NUREG/CR-6456 REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING, AND FEEDRING CRACKING AND WALL THINNING.

Weld

NUREG/CR-6181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION

Westinghouse AP600

NUREG/CR-6414: PIPING BENCHMARK PROBLEMS FOR THE WES-TINGHOUSE AP600 STANDARDIZED PLANT.

Yucca Mountain

NUREG/OR-6513 NO1: NRC HIGH-LEVEL RADIOACTIVE WASTE MAN-AGEMENT PROGRAM ANNUAL PROGRESS REPORT FISCAL YEAR 1008



NRC Originating Organization Index (Staff Reports)

This index lists those NRC organizations that have published staff reports. The index is arranged alphabetically by major NRC organizations (e.g., program offices) and then by subsections of these (e.g., divisions, branches) where appropriate. Each entry is followed by a NUREG number and title of the report(s). If further information is needed, refer to the main citation by NUREG number.

OFFICE OF EXECUTIVE DIRECTOR FOR OPERATIONS (EDO)

EGION 1 (POST 820201) NUREG-0837 V16 N03: NRC TLD DIRECT RADIATION MONITORING REGION NETWORK.Progress Report. July-September 1996. NUREG-0637 V16 N04: NRC TLD DIRECT RADIATION MONITORING

NETWORK Progress Report. October-December 1996

EDD - OFFICE OF ADMINISTRATION (PRE 870413 & POST 890205) RULES & DIRECTIVES REVIEW BRANCH (POST 920323) NUREG-0936 V15 N02: NRC REGULATORY AGENDA.Somiarinual Report July-December 1996

EDO - OFFICE OF THE CONTROLLER (PRE 820418 & POST 890205) DIVISION OF BUDGET & ANALYSIS (POST 890205) NUREG-1100 V13: BUDGET ESTIMATES.Fiscal Year 1998.

EDO - OFFICE FOR ANALYSIS & EVALUATION OF OPERATIONAL DATA

- DIVISION OF SAFETY PROGRAMS (POST 870413) NUREG-1275 V12: OPERATING EXPERIENCE FEEDBACK REPORT Assessment Of Spent Fuel Cooling.
- EDO OFFICE OF INFORMATION RESOURCES MANAGEMENT & ARM
- (POST 861109) OFFICE OF INFORMATION RESOURCES MANAGEMENT (POST 890205)
 - NUREG-0304 V21 N03: REGULATORY AND TECHNICAL REPORTS (ABSTRACT INDEX JOURNAL). Compilation For Third Quarter 1996, July-September. NUREG-0540 V18 N11: TITLE LIST OF DOCUMENTS MADE PUBLIC-
 - LY AVAILAGLE November 1-30, 1996. NUREG-0540 V18 N12: TITLE LIST OF DOCUMENTS MADE PUBLIC-
 - LY AVAILABLE.December 1-31, 1996. NUREG-0540 V19 N01 TITLE LIST OF DOCUMENTS MADE PUBLIC-
 - LY AVAILABLE, JANUARY 1-31, 1997. NUREG-0750 V44 101: INDEXES TO NUCLEAR REGULATORY COM-
 - MISSION ISSUANCES.July-September 1996. NUREG-0750 V44 N05: NUCLEAR REGULATORY COMMISSION IS-
 - SUANCES FOR NOVEMBER 1996, Pages 229-314, NUREG-0750 V44 N06: NUCLEAR REGULATORY COMERISSION IS-
 - SUANC 35 FOR DECEMBER 1996, Pages 315-432, NUREG-1750 V45 N01: NUCLEAR REGULATORY COMMISSION IS-
 - SUANCES FOR JANUARY 1997. Pages 1-47

ġ,

EDO - OFFICE OF NUCLEAR MATERIAL SAFETY & SAFEGUARDS OFFICE OF NUCLEAR MATERIAL SAFETY & SAFEGUARDS NUREG-1536: STANDARD REVIEW PLAN FOR DRY SPENT FUEL STORAGE SYSTEMS. Final Report.

NUREG-1571: INFORMATION HANDBOOK ON INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS. ISION OF INDUSTRIAL & MEDICAL NUCLEAR SAFETY (POST DIVISION OF

10

- 870729) NUREG-1562 DRFT FC: STANDARD REVIEW PLAN FOR APPLICA-TIONS FOR LICENSES TO DISTRIBUTE BYPRODUCT MATERIAL TO PERSONS EXEMPT FROM THE REQUIREMENTS FOR AN
- NRC LICENSE 10CFF Parts 30.14,30.15, 30.16,30.19 & 30.20. DIVISION OF WASTE MANAGEMENT (NMSS 940403) NUREG-1508: FINAL ENVIRONMENTAL IMPACT STATEMENT TO CONSTRUCT AND OPERATE THE GROWNPOINT URANIUM SO-
- LUTION MINING PROJECT, CROWNPOINT, NEW MEXICO.Docket No. 40-8968 (Hydro Resources, Inc.) NUREG-1532: FINAL TECHNICAL EVALUATION REPORT FOR THE
- PROPOSED REVISED RECLAMATION PLAN FOR THE ATLAS CORPORATION MOAB MILL Source Material License No. SUA-917 Docket No. 40-3453 (Atlas Corporation)

EDO - OFFICE OF NUCLEAR REGULATORY RESEARCH (POST 820405) DIVISION OF REGULATORY APPLICATIONS (POST 941217) NUREG-0713 V17: OCCUPAT/ONAL RADIATION EXPOSURE AT COMMERICAL NUCLEAR POWER REACTORS AND OTHER FACILITIES, 1995. Twenty-Eighth Annual Report. NUREG-1492: REGULATORY ANALYSIS ON CRITERIA FOR THE RE-

- PATIENTS LEASE OF ADMINISTERED RADIOACTIVE
- MATERIAL Final Report. IVISION OF SYNTEMS TECHNOLOGY (POST 941217) NUREG-1545: EVALUATION CRITERIA FOR COMMUNICATIONS-RE-DIVISION OF LATED CORRECTIVE ACTION PLANS.

EDO - OFFICE OF NUCLEAR REACTOR REGULATION (POST 800428)

- FFICE OF NUCLEAR REACTOR REGULATION (POST 941001) NUREG-0040 V29 N03: LICENSEE CONTRACTOR AND VENDOR IN-SPECTION STATUS REPORT. Quarterly Report, July-September 996. (White Book)
- NUREG-0040 V20 NO4: LICENSEE CONTRACTOR AND VENDOR IN-SPECTION STATUS REPORT. Quarterly Report.October-December
- 1996.(White Book) NUREG-1021 INT ROB: OPERATOR LICENSING EXAMINATION STANDARDS FOR POWER REACTORS. NUREG-1545: EVALUATION CRITERIA FOR COMMUNICATIONS-RE-
- LATED CORRECTIVE ACTION PLANS. NUREG-1574 DRFT FC: STANDARD REVIEW PLAN ON ANTITRUST.Draft Report For Comment. NUREG-1577 DRFT FC: STANDARD REVIEW PLAN ON POWER RE-
- ACTOR LICENSEE FINANCIAL QUALIFICATIONS AND DECOMMIS-SIONING FUNDING ASSURANCE Draft Report For Comment.



NRC Originating Organization Index (International Agreements)

This index lists those NRC organizations that have published international agreement reports. The index is arranged alphabetically by major NRC organizations (e.g., program offices) and then by subsections of these (e.g., divisions, branches) where appropriate. Each entry is followed by a NUREG number and title of the report(s). If further information is needed, refer to the main citation by NUREG number.

There were no NUREG/IA reports published this quarter.



NRC Contract Sponsor Index (Contractor Reports)

This index lists the NRC organizations that sponsored the contractor reports listed in this compilation. It is arranged alphabetically by major NRC organization (e.g., program office) and then by subsections of these (e.g., divisions) where appropriate. The sponsor organization is followed by the NUREG/CR number and title of the report(s) prepared by that organization. If further information is needed, refer to the main citation by the NUREG/CR number.

EDO - OFFICE FOR ANALYSIS & EVALUATION OF OPERATIONAL DATA

DIVISION OF SAFETY PROGRAMS (POST 870413)

- NUREG/CR-6456: REVIEW OF INDUSTRY EFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING, AND FEEDRING CRACKING AND WALL THINNING.
- EDO OFFICE OF NUCLEAR MATERIAL SAFETY & SAFEGUARDS
 - DIVISION OF INDUSTRIAL & MEDICAL NUCLEAR SAFETY (POST 8707201
 - NUREG/CR-6528; ENVIRONMENTAL ASSESSMENT PROPOSED LI-CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MAS-SACHUSETTS
- DIVISION OF WASTE MANAGEMENT (NMSS 940403)
- NUREG/CR-8513 NO1: NRC HIGH-LEVEL RADIOACTIVE WASTE MANAGEMENT PROGRAM ANNUAL PROGRESS REPORT: FISCAL YEAR 1996.
- EDO OFFICE OF NUCLEAR REGULATORY RESEARCH (POST 820408) DIVISION OF ENGINEERING TECHNOLOGY (POST 941217) NUREG/CR-4218 V12 N2 HEAVY-SECTION STEEL TECHNOLOGY
 - PROGRAM.Semiannual Progress Report For April 1995 Through September 1995
 - NUREG/CR-8181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION
 - NUREG/CR-6370 BLOCKAGE 2.5 USER'S MANUAL

 - NUREG/CR-6371 BLOCKAGE 2.5 REFERENCE MANUAL NUREG/CR-6379: AN IMPROVED CORRELATION PROCEDURE FOR
 - SUBSIZE AND FULL-SIZE CHARPY IMPACT SPECIMEN DATA. NUREG/CR-6389: IPIRG-2 TASK 1 PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS Final Report September 1991 - Novembor 1995.
 - NUREG/CR-8428 V01: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS, DATA ANALYSIS.
 - NUREG/CR-8426 V02: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS Data Records.

- NUREG/CR-6433: CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO STVERE ACCI-DENT CONDITIONS. NUREG/CR-6446: FRACTURE TOUGHNESS EVALUATIONS OF
- TP304 STAINLESS STEEL PIPES. NUREG/CR-6452: THE SECOND INTERNATIONAL PIPING INTEGRI-TY RESEARCH GROUP (IPIRG-2) PROGRAM Final Report. NUREG/CR-6466: ASSESSMENT OF MODULAR CONSTRUCTION
- FOR SHFETY-RELATED STRUCTURES AT ADVANCED NUCLEAR POWER PLANTS. NUREG/CR-6529: VALIDATION OF TECTONIC MODELS FOR AN IN-
- NUHEG/CH-6529 VALIDATION OF TECTONIC MODELS FOR AN IN-TRAPLATE SEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA. DIVISION OF REGULATORY APPLICATIONS (POST 941217) NUREG/CR-4409 V06: DATA BASE ON DOSE REDUCTION PROJECTS FOR NUCLEAR POWER PLANTS. NUREG/CR-65397: RADIATION SAFETY CONCERNS FOR PREG-

- NANT OR BREAST-FEEDING PATIENTS The Positions Of The NCRP And The ICRP. NUREG/CR-6493: DOSES TO THE HAND DURING THE ADMINIS-
- TRATION OF RADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M.I-131. AND LU-177. DIVISION OF SYSTEMS TECHNOLOGY (POST 941217) NUREG/CR-6295: REASSESSMENT OF SELECTED FACTORS AF-

- FECTING SITING OF NUCLEAR POWER PLANTS. NUREG/CR-6469: EXPERIMENTS TO INVESTIGATE DIRECT CON-TAINMENT HEATING PHENOMENA WITH SCALED MCGELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT. NUREG/CR-6474: PRELIMINARY PHENOMENA IDENTIFICATION
- AND RANKING TABLES (PIRT) FOR SBWR STARTUP STABILITY.

EDO - OFFICE OF NUCLEAR REACTOR REGULATION (POST 800428)

- FFICE OF NUCLEAR REACTOR "2GULATION (POST 941001) NUREG/CR-8393: INTEGRATED SYSTEM VALIDATION: METHODOL-
- OGY AND REVIEW CRITERIA. NUREG/CR-6400: HUMAN FACTORS ENGINEERING (HFE) IN-SIGHTS FOR ADVANCED REACTORS BASED UPON OPERATING EXPERIENCE.
- NUREG/CR-6414: PIPING BENCHMARK PROBLEMS FOR THE WES-TINGHOUSE AP600 STANDARDIZED PLANT.



Contractor Index

This index lists, in alphabetical order, the contractors that prepared the NUREG/CR reports listed in this compilation. Listed below each contractor are the NUREG/CR numbers and titles of their reports. If further information is needed, refer to the main citation by the NUREG/CR number.

ARGONNE NATIONAL LABORATORY

NUREG/CP-0154: PROCEEDINGS OF THE CNRA/CSNI WORKSHOP ON STEAM GENERATOR TUBE INTEGRITY IN NUCLEAR POWER PLANTS

BATTELLE MEMORIAL INSTITUTE, COLUMBUS LABORATORIES

- NUFIEG/CR-8389: IPIRG-2 TASK 1 PIPE SYSTEM EXPERIMENTS WITH CIRCUMFERENTIAL CRACKS IN STRAIGHT-PIPE LOCATIONS.Final Report.September 1991 - November 1995. NUREG/CR-8446: FRACTURE TOUGHNESS EVALUATIONS OF TP304
- STAINLESS STEEL PIPES. NUREG/CR-6452: THE SECOND INTERNATIONAL PIPING INTEGRITY
- RESEARCH GROUP (IPIRG-2) PROGRAM.Final Report.

BATTELLE MEMORIAL INSTITUTE, PACIFIC NORTHWEST LABORATORY

NUREG/CR-6181 R01: A PILOT APPLICATION OF RISK-INFORMED METHODS TO ESTABLISH INSERVICE INSPECTION PRIORITIES FOR NUCLEAR COMPONENTS AT SURRY UNIT 1 NUCLEAR POWER STATION

BROOKHAVEN NATIONAL LABORATORY

- NUREG/CP-0157 V01: PROCEEDINGS OF THE TWENTY-FOURTH WATER REACTOR SAFETY INFORMATION MEETING. Plenary Session, High Burnup Fuel, Containment And Structural Aging. NUREG/CP-0157 V02: PROCEEDINGS OF THE TWENTY-FOURTH
- WATER REACTOR SAFETY INFORMATION MEETING Reactor Pressure Vessel Embrittlement And Thermal Annealing,Reactor Vessel Lower Head Integrity And Evaluation And Projection of Steam Genera-
- WATER REACTOR SAFETY INFORMATION MEETING. PRA And HRA. And Probabilistic Seismic Hazard Assessment And Seismic Siting Crite-
- NUREG/CR-4409 V06: DATA BASE ON DOSE REDUCTION PROJECTS FOR NUCLEAR POWER PLANTS. NUREG/CR-8295 R ASSESSMENT OF SELECTED FACTORS AF-
- FECTING SITING OF NUCLEAR POWER PLANTS. NUREG/CR-8393: INTEGRATED SYSTEM VALIDATION: METHODOLO
- GY AND REVIEW CRITERIA
- NUREG/CR-8307: RADIATION SAFETY CONCERNS FOR PREGNANT OR BREAST-FEEDING PATIENTS The Positions Of The NCRP And The KORP
- NUREG/CR-8400: HUMAN FACTORS ENGINEERING (HFE) INSIGHTS FOR ADVANCED REACTORS BASED UPON OPERATING EXPERI-ENCE
- NUREG/CR-8414: PIPING BENCHMARK PROBLEMS FOR THE WES-TINGHOUSE AP600 STANDARDIZED PLANT
- NUREG/CR-6474: PRELIMINARY PHENOMENA IDENTIFICATION AND RANKING TABLES (PIRT) FOR SEWER STARTUP STABILITY. NUREG/CR-6486: ASSESSMENT OF MODULAR CONSTRUCTION FOR
- SAFETY-RELATED STRUCTURES AT ADVANCED NUCLEAR POWER PLANTS
- NUREG/CR-6493. DOSES TO THE HAND DURING THE ADMINISTRA-TION OF RADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M, I-131, AND LU-177

CENTER FOR NUCLEAR WASTE REGULATORY ANALYSES

NUREG/CR-6513 NO1: NRC HIGH-LEVEL RADIOACTIVE WASTE MAN-AGEMENT PROGRAM ANNUAL PROGRESS REPORT: FISCAL YEAR 1996

EQE ENGINEERING CONSULTANTS (FORMERLY EQE ENGINEERING, INC.)

NUREG/CR-6433 CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS EUBJECTED TO SEVERE ACCI-DENT CONDITIONS.

IDAHO NATIONAL ENGINEERING LABORA) ORY

NUREG/CR-6456: REVIEW OF INDUSTRY FFFORTS TO MANAGE PRESSURIZED WATER REACTOR FEEDWATER NOZZLE, PIPING, AND FEEDRING CRACKING AND WALL THINNING.

MINNESOTA, UNIV. OF, MINNEAPOLIS, MN

NUREG/CR-8493: DOSES TO THE HAND DURING THE ADMINISTRA-TION OF RADIOLABELED ANTIBODIES CONTAINING Y-90, TC-99M, I-131. AND LU-177

OAK RIDGE NATIONAL LABORATORY

- NUREG/CR-4219 V12 N2: HEAVY-SECTION STEEL TECHNOLOGY PROGRAM.Semiannual Fougress Report For April 1995 Through September 1995
- NUREG/CR-6379: AN IMPROVED CORRELATION PROCEDURE FOR
- SUBSIZE AND FULL-SIZE CHARPY IMPACT SPECIMEN DATA. NUREG/GR-6426 V01: DUCTILE FRACTURE TOUGHNESS OF MODI-FIED A 302 GRADE B PLATE MATERIALS.DATA ANALYSIS. NUREG/CR-6426 V02: DUCTILE FRACTURE TOUGHNESS OF MODI-
- FIED A 302 GRADE B PLATE MATERIALS. Data Records NUREG/CR-8528: ENVIRONMENTAL ASSESSMENT PROPOSED LI-
- CENSE RENEWAL OF NUCLEAR METALS, INC. CONCORD, MASSA-CHUSETTS

- SANDIA NATIONAL LABORATORIES NUREG/CR-6433: CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCI-DENT CONDITIONS.
- NUREG/CR-5469: EXPERIMENTS TO INVESTIGATE DIRECT CON-TAINMENT HEATING PHENOMENA WITH SCALED MODELS OF THE CALVERT CLIFFS NUCLEAR POWER PLANT.

SARGENT & LUNDY, INC.

NUREG/CR-6433: CONTAINMENT PERFORMANCE OF PROTOTYPI-CAL REACTOR CONTAINMENTS SUBJECTED TO SEVERE ACCI-DENT CONDITIONS.

SCIENCE & ENGINEERING ASSOCIATES, INC.

NUREG/CR-8370: BLOCKAGE 2.5 USER'S MANUAL NUREG/CR-8371: BLOCKAGE 2.5 REFERENCE MANUAL

SCIENCE APPLICATIONS INTERNATIONA., CORP. (FORMERLY

SCIENCE APPLICATIONS.

NUREG-0713 V17: OCCUPATIONAL RADIATION EXPOSURE AT COM-NUCLEAR POWER REACTORS MERICAL AND OTHER FACILITIES, 1995. Twenty-Eighth Annual Report.

SOFTWARE EDGE, INC.,

NUREG/CR-6370: BLOCKAGE 2.5 USER'S MANUAL. NUREG/CR-6371: BLOCKAGE 2.5 REFEFENCE MANUAL.

SOUTH CAROLINA, UNIV. OF, COLUMBIA, SC

NUREG/CR-6529: VALIDATION OF TECTONIC MODELS FOR AN IN-TRAPLATE SEISMIC ZONE, CHARLESTON, SOUTH CAROLINA WITH GPS GEODETIC DATA



International Organization Index

This index lists, in alphabetical order, the countries and performing organizations that prepared the NUREG/IA reports listed in this compilation. Listed below each country and performing organization are the NUREG/IA numbers and titles of their reports. If further information is needed, refer to the main citation by the NUREG/IA number.

There were no NUREG/IA reports published this quarter.

Licensed Facility Index

This index lists the facilities that were the subject of NRC staff or contractor reports. The facility names are arranged in alphabetical order. They are preceded by their Docket number and followed by the report number. If further information is needed, refer to the main citation by the NUREG number.

52-003	AP600 Standard Plant Design, Westinghouse	NUREG/CR-6414	50-018	Calvert Cliffs Nuclear Power Plant, Unit 2,	NUREG/CR-6469
40-3453 50-317	Atlas Corp., Denver, CO, Calvert Catts Nuclear Power Plant, Unit 1, Baltimore Gas & Electric	NUREG-1532 NUREG/CR-6469	40-8968 40-8966 40-0672 50-280	Hydro Resources, Inc., Dailas, TX, Nuclear Metals, Inc., Concord, MA, Nuclear Metals, Inc., Concord, MA, Surry Power Station, Unit 1, Virginia Electric &	NUREG-1508 NUREG/CR-8528 NUREG/CR-8528 NUREG/CR-6181 R01

NRC: FORM 335 U.S. NUCLEAR REGULATORY COMMISSION (2-86) NRCM 1102, 3202 BIBLIOGRAPHIC DATA SHEET (See instructions on the reverse)	1 REPORT NUMBER (Assigned by NRC, Add Vol. Supp. Rev. and Addendum Numbers, if any.) NUREG-0304 Vol. 22, No. 1
E TITLE AND SUBTITLE	3. DATE REPORT PUBLISHED
Compilation for First Quarter 1997 January – March	MONTH YEAR June 1997 4. FIN OR GRANT NUMBER
5 AUTHOR(6)	6 TYPE OF REPORT
	7. PERIOD COVERED (Indusive Dates)
	January - March 1997
Publications Branch Office of Information Resources Management U.S. Nuclear Regulatory Commission Washington, DC 20555-0001 SPONSORING ORGANIZATION - NAME AND ADDRESS (If NRC, type "Same as above", if contractor, provide NRC Division arit making address)	Office or Region, U.S. Nuclear Regulatory Commission.
Same as 8, above.	
o supplementary notes M. A. Sheehan, Project Manager	
This journal includes all formal reports in the NUREG series prepared by the lings of conferences and workshops; as well as international agreement reports, indexed for access by title and abstract, secondary report number, personal aut staff and international agreements, contractor, international organization, and	NRC staff and contractors; proceed- The entries in this compilation are hor, subject, NRC organization for licensed facility.
2. KEY WORDS/DESCRIPTORS. (List words or phrases that will assist researchers in locating the report.)	13 AVAILABILITY STATEMENT Unlimited
compilation abstract index	14 SECURITY CLASSIFICATION (This Page) Unclassified (This Report)
	Unclassified
	16. PRICE



12

]

Federal Recycling Program

Main Citations and Abstracts



Secondary Report Number Index



3

Personal Author Index



Subject Index



NRC Originating Organization Index (Staff Reports)



NRC Originating Organization Index (International Agreements)



NRC Contractor Sponsor Index



Contractor Index



10

International Organization Index

Licensed Facility Index

PENALTY FOR PRIVATE USE, \$300 OFFICIAL BUSINESS

LOOSSSI39531 US NAC-01AM BDURLICATIONS BR

INGTON

DC

20555

NUCLEAR REGULATORY COMMISSION WASHINGTON, DC 20555-0001

UNITED STATES

1 JANJACIASICVI ANCH

FIRST CLASS MAIL POSTAGE AND FEES PAID USNRC PERMIT NO. G-67 NUREG-0304, Vol. 22, No. 1

REGULATORY AND TECHNICAL REPORTS COMPILATION FOR FIRST QUARTER 1997