

## UNITED STATES NUCLEAR REGULATORY COMMISSION

REGION III 801 WARRENVILLE ROAD LISLE, ILLINOIS 60532-4351

August 12, 1999

Mr. M. Wadley President, Nuclear Generation Northern States Power Company 414 Nicollet Mall Minneapolis, MN 55401

SUBJECT:

PRAIRIE ISLAND INSPECTION REPORT 50-282/99006(DRP);

50-306/99006(DRP)

Dear Mr. Wadley:

On July 20, 1999, the NRC completed an inspection at your Prairie Island Nuclear Generating Plant. The results of this inspection were discussed with Mr. D. Schuelke and other members of your staff. The enclosed report presents the results of that inspection.

The inspection was an examination by the resident inspectors of activities conducted under your license as they relate to reactor safety, verification of performance indicators, event followup, and to compliance with the Commission's rules and regulations and with the conditions of your license. Within these areas, the inspection consisted of a selective examination of procedures and representative records, observations of activities, and interviews with personnel.

Based on the results of this inspection, one issue related to the potential effect of a high-energy line break on control room habitability caused by a broken door was initially evaluated under the risk significance determination and was found to have potential safety significance. This issue has been entered into your corrective action program. Our final risk estimation will be documented in a future inspection report. In addition, one issue related to improperly secured Unit 1 containment sump hatches was categorized as being of low risk significance, although regulatory requirements were violated. Therefore, one non-cited violation (NCV), consistent with Appendix C of the Enforcement Policy, was identified. This issue has been entered into your corrective action program. These two issues are listed in the Enclosed summary of findings and are discussed in the report.

The NCV discussed above is described in the subject inspection report. If you contest the violation or severity level of the NCV, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001, with copies to the Regional Administrator, Region III, the Director, Office of Enforcement, United States Nuclear Regulatory Commission, Washington, DC 20555-0001, and the NRC Resident Inspectors at the Prairie Island facility.

In addition to the above, the inspection included a review and evaluation of the first quarter of 1999 Performance Indicators. All Performance Indicators were green with the exception of the Unit 2 Performance Indicator for Safety System Functional Failures which was reported to be white with six failures in the most recent four quarters. The indicator would have been white from the first quarter of 1998 based on the historic data submitted with the first pilot plant Performance Indicator submittal. Of the six failures counted in the first quarter of the 1999 Performance Indicators submittal, five involved 10 CFR Part 50, Appendix R, fire protection issues discovered in the second and third quarters of 1998. Those issues were all extensively evaluated by the NRC in its fire protection functional inspection as discussed in Inspection Report 50-282/93016(DRS); 50-306/98016(DRS). The NRC resolved the enforcement aspects of the issues as discussed in a letter to you from the Regional Administrator, NRC Region III, dated March 30, 1999 (EA 98-526). The NRC assessed the issues during the Plant Performance Review as reported in a letter to you from the Director of Reactor Projects, NRC Region III, dated March 26, 1999. The NRC plans no additional actions or inspections beyond the baseline inspection program because of those issues.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response, if you choose to provide one, will be placed in the NRC Public Document Room.

Sincerely,

/s/ R. D. Lanksbury

Roger Lanksbury, Chief Reactor Projects Branch 5

Docket Nos. 50-282; 50-306 License Nos. DPR-42; DPR-60

Enclosure:

Inspection Report 50-282/99006(DRP);

50-306/90006(DRP)

cc w/encl:

Site General Manager, Prairie Island

Plant Manager, Prairie Island S. Minn, Commissioner, Minnesota Department of Public Service

State Liaison Officer, State of Wisconsin

Tribal Council, Prairie Island Dakota Community

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