



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

REGION IV  
611 RYAN PLAZA DRIVE, SUITE 400  
ARLINGTON, TEXAS 76011-8064

AUG 27 1997

Charles M. Dugger, Vice President  
Operations - Waterford 3  
Entergy Operations, Inc.  
P.O. Box 3  
Killona, Louisiana 70066

SUBJECT: NRC INSPECTION REPORT 50-382/97-14 AND NOTICE OF VIOLATION

Dear Mr. Dugger:

An NRC inspection was conducted July 7-25, 1997, at your Waterford Steam Electric Station, Unit 3 reactor facility. Following the exit meeting the results of this inspection were discussed with Messrs. A. Wrape and E. Ewing of your staff on August 27, 1997. The enclosed report presents the scope and results of the inspection.

This inspection involved followup of nine open items including three violations and two unresolved items. Your staff had taken adequate corrective action for two of the violations. However, we are concerned that your staff did not carefully review your existing license basis before reperforming the safety evaluation for changing your commitment to ANSI N195, "Fuel Oil Systems for Emergency Diesel Generators."

Both unresolved items were determined to be violations of NRC requirements. The circumstances surrounding both violations are described in detail in the enclosed report. The first violation involved failure to meet the load shed and sequence verification requirements of Technical Specification 4.8.1.1.2.e during Refueling Outage 7 for loads supplied via electrical contactors. In NRC Inspection Report 50-382/97-10, you were previously cited for a violation of Technical Specification 4.8.1.1.2.e. While we noted that you had not yet responded to this violation at the time of the exit, your staff had already taken corrective actions to address the violation, including this additional example, during Refueling Outage 8. Further, your subsequent response to the violation identified in NRC Inspection Report 50-382/97-10 described the corrective actions applicable to this additional violation example. Therefore, we have concluded that a further citation is not necessary. The second violation involved three failures to update your license basis as required by 10 CFR 50.71(e). We noted that your staff promptly initiated license document change requests to correct two of the examples and that you had previously docketed your corrective action commitments for the third example. While this violation is cited in the enclosed Notice of Violation (Notice), we have determined that these actions satisfy the intent of 10 CFR 2.201 and therefore, no further response is required. //

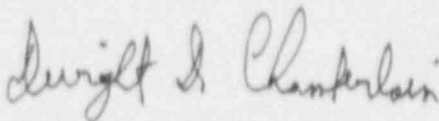
In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, and its enclosure will be placed in the NRC Public Document Room (PDR).

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Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Sincerely,

  
for Arthur T. Howell III, Director  
Division of Reactor Safety

Docket No.: 50-382  
License No.: NPF-38

Enclosures:

1. Notice of Violation
2. NRC Inspection Report  
50-382/97-14

cc w/enclosures:

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Washington, D.C. 20005-3502

E-Mail report to T. Freye (TJF)  
E-Mail report to NRR Event Tracking System (IPAS)  
E-Mail report to Document Control Desk (DOCDESK)

bcc to DCD (IE01)

bcc distrib. by RIV:

Regional Administrator	Resident Inspector
DRP Director	DRS-PSB
Branch Chief (DRP/D)	MIS System
Project Engineer (DRP/D)	RIV File
Branch Chief (DRP/TSS)	

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RIV:RI:EB	C:EB	E	D:DRP	D:DRS		
LJSmith/lmb*	CAVanDenburgh	TPGwynn	ATHowell III			
07/29/97	08/12/97	08/14/97	08/27/97			

\*Previously concurred

OFFICIAL RECORD COPY

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