

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JANUARY 1987

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Gregory J. Kirchner, telephone number (815)357-6761, extension 704.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

January 1-31, 1987

January 1, 0000 Hours.	Reactor at 1010 MWe, Turbine Generator on line.
January 9, 1155 Hours.	Commence load drop to 925 MWe to observe TCV oscillations.
January 9, 1615 Hours.	Reduce power by 200 MWe/hr. to maximum valve position on FCV (620 MWe) to fix CV limit switch in steam chest area.
January 9, 2015 Hours.	Ramp up 150 MWe/hr. to 850 MWe, then 10 MWe/hr. to 1010 MWe.
January 16, 0620 Hours.	Dropped 50 MWe for CRD exercising, then return to 1000 MWe.
January 23, 2200 Hours.	Commence load drop of 225 MWe/hr. to 660 MWe to work on number 3 C.V.
January 24, 0040 Hours.	Inserted groups 9E1 and 9E2 to position no. 40. Started 150 MWe/hr. to 890 MWe, then 10 MWe/hr. to 1000 MWe.
January 26, 1135 Hours.	Reactor scram due to blown generator pot fuse, generator off-line.
January 27, 1600 Hours.	Reactor critical.
January 30, 1245 Hours.	Generator synchronized to grid.
January 31, 1120 Hours.	Reactor scram due to loss of alternator exciter field. Turbine trip at 350 MWe.
January 31, 2400 Hours.	Reactor in hot shutdown.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility License or Technical Specification.

There were no Amendments to the Facility License or Technical Specifications during this reporting period.

2. Facility or Procedure changes requiring NRC approval.

There were no Facility or Procedural changes requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no Tests or Experiments requiring NRC approval during this reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a Summary of Safety-Related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

5. Completed Safety Related Modifications.

The following Table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L53315	VC Filter Bypass Damper OVE08YA	During LIS-VC-01 damper position did not change.	No indication at 731' R/15.	Damper Actuator rebuilt per LEP-EQ-129.
L56332	Outer Trackway Door #20, Handwheel is broken off.	Damaged pin locking sleeves.	Door hard to open.	R/R guide sleeves and adjusted springs.
L62489	LPRM 3C-48-41	Diode CR2 on LPRM 3C-48-41 is defective.	When LPRM card is placed in cal. position, the bypass light comes on.	Replace diode CR2 and returned to service.
L62914	"B" Rx. Recirc. Flow Control Vlv. Hyd. Pwr. Unit Outboard Isol. Vlv. 1B33-F343B	Plunger core of coil is not being pulled up into the coil.	Coil is overheating and fuse interrupts immediately.	Replace control box and coil. Set limits and verify proper operation.
L62967	Drywell Penetration E-14	Loose fitting at pressure gage.	Penetration lost pressure was at 3 PSIG.	Tighten loose pressure gage, charge penetration to 30 PSIG.
L63419	1B RR FCV Hydraulic O.B. Isol. Vlv.	Bad open limit switch.	Valve cycling indicated dual operation for open and closed positions.	Replace limit switch, verified proper cycling operation.
L63979	HCU 22-55	Bad Cartridge Valve (111)	#111 Valve leaks badly when taken off back seat position.	Replaced valve like for like.
L64129	Snubber Collar on "C" RHR Inj. Line	Unknown.	Loose snubber collar to be tightened.	Retighten snubber collar.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L64131	SP DW. Temp Recorder. 1TR-CM037	Paper Tape not advancing.	SP DW temp. recorder not operating.	Replace chart drive motor, verify proper Ops.
L64175	Main Steam Line PRM Drawers 1D18-K610 A/B/C/D	Sporadic, unreliable readings.	Occasionally provided half scrams.	Replaced discrete com- ponents with more reliable microprocessor system.
L64180	Div I, W.R. Level Recorder 1B21-R884A	Level Indicator out of calibration.	Indicated level is between +12" to +15" on Div I, but shows +21" on Div II.	Recalibrate level indicator.
L64182	'1B' VR Exhaust PRM RIY-1D18-K609B	PRM spiked Hi-Hi initiat- ing annunciator 1H13-P601.	Isolated Case.	Checked set point and monitored for 1 day.
L64198	RCIC Steam Line Drain Vlv. 1E51-F026	Spring adjustment nut loose.	Valve does not cycle fully.	Adjusted spring nut to close valve as required.
L64267	A RHR to Rad Waste Valve. 1E12-F049A	Bad torque switch.	Valve will not "seal in" close.	Replace torque switch. Verify proper operation.
L64293	S/P Temp. Recorder 1TR-CM037	Bad Love Controller.	Pts. 14 and 15 are reading 20° F lower than other pts.	Replaced Love Controller 1TY-CM057EA & EB.
L64303	CRD HCU 46-43 111 Valve.	Valve Stem Leaks.	Unable to charge vlv. for leak test.	Replaced valve like for like.
L64316	Div. II Supp. Pool Temp. Recorder. 1TR-CM038	Loose Allen screw on Servo motor.	Recorder jumped from 65° F to 125° F.	Tighten Allen screw verified proper operation.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L64320	RCIC Pump Disch. Pressure Indicator. 1B51-R601	Bad transmitter.	Indicator pegged downscale.	Replaced transmitter.
L64322	Supp. Pool Dry Well Temp. Recorder. 1TR-CM038	Loose set screws in clutch.	Recorder prints same value for all 17 points.	Tighten set screws, verify proper Ops.
L64477	HCU 34-43.	Bad valve leak.	Vlv. leaks when taken off its backseat.	Replace Vlv. like for like.
L64565	Div. II Post-Loca Oxygen Monitor. 1AIR-CM048	Bad Vacuum Pump.	Monitor indicates oxygen concentration to high.	Replace vacuum pump.
L64724	HCU 58-27 CRD Accumulator.	Bad Valve (111).	Valve is leaking when partially open and full closed.	Replace valve.
L64897	HCU 30-27.	Bad Valve (111).	Valve leaks around the stem.	Replace valve.
L57853	Control Room Supply Damper. OVC-24YA	Defective Actuator.	Damper Sticks.	Rebuilt actuator.
L63690	SBGT WRGM. LO1D18000	Defective Pump and Transducer Assemblies.	Intermittent Shutdown of WRGM due to process sample flow loss.	Replaced pump and transducer assemblies.
L64130	Ammonia Detector C.R. HVAC System. OXY-VC165A	Defective Photocell Board.	OXY-VE165A would not trip during test.	Cleaned photocell board and replaced paper drive motor.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	UNIT #1 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L64371	0VE09YB Damper.	Two welds and a bushing were broken on the linkage.	Damper difficult to cycle by hand.	Replaced the welds, in- stalled new bushing.
L64372	0VE07YB Damper.	Three welds on the coupl- ings were broken.	Damper difficult to cycle by hand.	Repaired the welds and lubricated the bearings.

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER: A brief synopsis of incorporated modification objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

The following is a list of the Safety Related Modifications completed from January 1, 1987 through January 31, 1987.

- M01-0-84-060 - This was to modify existing air, water, and electrical services to provide interconnections to the Interim Radwaste Storage Facility. There were no unreviewed safety questions.

- M01-0-85-014 - This modification was to extend service air and clean condensate tie-ins. This will provide extension lines outside the turbine building to the Interim Radwaste Storage Facility. There were no unreviewed safety questions.

- M01-0-85-017 - This modification provided for the actuator on OFZ-VC141 with a new upgraded model. This new model is nuclear grade, has improved viton seals, and an increased torque rating. There were no unreviewed safety questions.

- M01-0-86-001 - Replace refrigeration compressor low oil pressure switch with PENN model P45-NCA-82. There were no unreviewed safety questions.

- M01-0-83-088 - This modification provided the capability to isolate air cylinders during operation of the Control Room Breathing Air System. In addition to this, it also provided high pressure service valves to replace the current valves to eliminate air leakage. There were no unreviewed safety questions.

- M01-1-84-051 - This modification provided for the addition of piping restraints in the low pressure heater bay to prevent the piping from moving excessively. There were no unreviewed safety questions.

- M01-1-85-014 - This modification provided for the installation of a new drywell chiller LVPl45. The associated chilled and service water piping, and electrical equipment was also installed. There were no unreviewed safety questions.

- M01-1-85-036 - This modification updated drawings/references and evaluated approved replacement of any LPRM's with Westinghouse model WL24286 LPRM's with LEMO connector. There were no unreviewed safety questions.

- M01-1-85-044 - This modification was for the design and installation requirements for attaching the new chiller water and service water piping to the existing plant systems. There were no unreviewed safety questions.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, January 1, through January 31, 1987. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
87-001-00	01/09/87	Possible procedure deficiency.
87-002-00	01/15/87	Failure of 'O' Diesel Generator to close onto BUS 241Y.
87-003-00	01/26/87	Reactor Scram - Main Generator lockout and turbine trip.
87-004-00	01/28/87	'A' ammonia detector trip due to jam in cassette tape.
87-005-00	01/31/87	Reactor Scram as a result of generator trip.
87-006-00	01/06/87	Div II DC inoperable due to personnel error.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE February 10, 1987
 COMPLETED BY Gregory Kirchner
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: January, 1987 GROSS HOURS IN REPORTING PERIOD: 744
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 1050
 4. REASONS FOR RESTRICTION (IF ANY): Administrative
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|--|----------------|----------------|-----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>702.9</u> | <u>702.9</u> | <u>15138.5</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>1642.0</u> |
| 7. HOURS GENERATOR ON LINE | <u>634.1</u> | <u>634.1</u> | <u>14608.0</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>1829280</u> | <u>1829280</u> | <u>40584922</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>614078</u> | <u>614078</u> | <u>13304755</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>593383</u> | <u>593383</u> | <u>12615957</u> |
| 12. REACTOR SERVICE FACTOR | <u>94.5%</u> | <u>94.5%</u> | <u>55.9%</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>94.5%</u> | <u>94.5%</u> | <u>61.9%</u> |
| 14. UNIT SERVICE FACTOR | <u>85.2%</u> | <u>85.2%</u> | <u>53.9%</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>85.2%</u> | <u>85.2%</u> | <u>53.9%</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>77.0%</u> | <u>77.0%</u> | <u>45.0%</u> |
| 17. UNIT CAPACITY FACTOR(USING DESIGN MWe) | <u>74.0%</u> | <u>74.0%</u> | <u>43.2%</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>6.1%</u> | <u>6.1%</u> | <u>14.9%</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
 NONE.
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
02/01/87

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: February 10, 1987
COMPLETED BY: Gregory Kirchner
TELEPHONE: (815) 357-6761
MONTH: January, 1987

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	976
2.	971
3.	967
4.	978
5.	977
6.	977
7.	992
8.	995
9.	880
10.	932
11.	977
12.	981
13.	985
14.	986
15.	985
16.	963

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	967
18.	966
19.	964
20.	967
21.	968
22.	968
23.	955
24.	886
25.	966
26.	461
27.	-12
28.	-14
29.	-14
30.	49
31.	113

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE February 10, 1987
COMPLETED BY Gregory Kirchner
TELEPHONE (815)357-6761

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
1	01/26/87	F	97.17	A	3	Blown Generator Pot Fuse.
2	01/31/87	F	12.67	A	3	Alterrex Problems.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no Safety Relief Valves Operated for Unit One during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
01-01-87	Div. 1 Water Leg Pump 1E21-C002	Impeller Adjustment
01-07-87	1E12-F049A	Repair Limitorque
01-14-87	'B' RHR System	Flush 'B' RHR Shutdown Cooling Section
01-17-87	O - Diesel Generator Air Receiver	Maintain receiver pressure with one air compressor

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment Systems during this reporting period.

5. Indications of Failed Fuel Elements

There were no indications of Failed Fuel Elements during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JANUARY, 1987

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Gregory J. Kirchner, telephone number (815)357-6761 extension 704.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

January 1-31, 1987

January 1, 0000 Hours.	Reactor Critical, Generator at 755 MWe.
January 3, 0020 Hours.	Ramping down at 150 MWe/hr. by decreasing core flow, then downshift to 125 MWe preparing for refuel.
January 3, 1100 Hours.	Control Rods inserted.
January 3, 1155 Hours.	Generator taken off-line.
January 3, 1803 Hours.	Scram on Reactor Water Level.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to Facility license or Technical Specification.

There were no Amendments to the Facility License or Technical Specifications during this reporting period.

2. Facility or Procedure changes requiring NRC approval.

There were no Facility or Procedure changes requiring NRC approval during the reporting period.

3. Tests and Experiments requiring NRC approval.

There were no Tests or Experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of Safety-Related Maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

5. Completed Safety Related Modifications.

The following table (Table 2) presents a list of completed Modifications during this reporting period. Each entry will have a short synopsis explaining details involved with each modification.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L62766	CRD Accumulator 38-59 Valve (lll)	Bad Cartridge (lll) Valve	Vlv. leaks nitrogen in every position.	Replace with a new valve
L63738	HCU 22-55 Instr. Block Stop	Bad Cartridge (lll) Valve	Vlv. leaks nitrogen.	Replaced with a new valve.
L63777	HCU 22-59 Instr. Block Stop	Bad Cartridge (lll) Valve	Vlv. leaks through stem	Replaced with a new valve.
L63838	HCU 10-47 Instr. Block Stop	Bad Cartridge (lll)	Vlv. leaks in every position.	Replaced with a new valve.
L63946	2VO06C Exhaust Damper.	Blade axle does not move.	Exhaust damper stays closed when HPCS Battery Room fan on.	Lubricated axle and adjusted center weight.
L64020	S/P Temp. Indicator/ Transmitter 2TY-CM057JB	Defective love controller.	Local indicator reading erratic.	Replaced defective love controller/calibrated.
L64069	HCU 50-51	Defective (lll) Cartridge	Valve is leaking.	Replaced with a new valve.
L64075	HCU 34-43	Defective (lll) Cartridge	Valve leaks	Replaced with a new valve.
L64138	Post LOCA Drywell Suction - 2CM021B	Defective open limit switch.	Shows dual indication for closed position.	Replaced defective limit switch.
L64214	HCU Accumulator 46-23 (lll) Valve.	Defective (lll) Cartridge	Valve leaks around packing.	Replaced with a new valve.
L64325	HCU 30-39 (lll) Valve.	Defective (lll) Cartridge	Valve leaks when backseated.	Replaced with a new valve.
L64431	C IRM	Defective IRM Drawer and IRM Pre-amp	Spikes causing 1/2 scrams.	Replaced IRM Drawer and Pre-amp and calibrated.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	Unit #2 COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L64507	Thermal O.L. Relay SBGT Discharge	Defective torque switch and thermal overload relay	Thermal overload relay will not trip.	Replaced torque switch and overload relay.
L64654	LPRM SB-24-17	Defective LPRM Card SB-24-17	Card will not reset after LPRM-HI alarm is initiated.	Replaced diode VR-2 and transistor Q5 on LPRM Card.
L64737	IRM "H"	Defective Hi-Voltage Connector	Connection caused spurious half scrams.	Installed new hi-voltage connector on IRM "H" Pre- amp.

TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

MODIFICATION NUMBER: A brief Synopsis of Incorporated Modification Objectives with final design resolution. Also, state reviewed or unreviewed safety questions.

The following list below has all the Safety Regulated Modifications completed from December 1, 1986, through December 31, 1986.

- M01-2-84-185 - This Modification added a relay to interlock the drywell equipment drain sump pump alarms. This will allow the pumps to run in the recirculation mode without causing an alarm. There were no unreviewed safety questions.
- M01-2-86-034 - This Modification will upgrade cable tray hangers to prepare for additional loading in trays. This additional loading is due to new cable being installed for alternate rod insertions. There were no unreviewed safety questions.
- M01-2-86-045 - This Modification will remove the horizontal fuel pool cooling return line spargers, which will allow the future installation of high density spent fuel storage racks. There were no unreviewed safety questions.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, January 1 through January 31, 1987. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
87-001-00	01/07/87	2A Diesel Generator failed to auto start.
87-002-00	01/17/87	Containment leakage limit exceeded.
87-003-00	01/20/87	Auto start of "B" VC EMU train due to contractor bumping breaker.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LaSalle Two
 DATE February 10, 1987
 COMPLETED BY Gregory Kirchner
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: January, 1987 GROSS HOURS IN REPORTING PERIOD: 744
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY
 (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 1050
 4. REASONS FOR RESTRICTION (IF ANY): Administrative

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>66.05</u>	<u>66.05</u>	<u>12069.5</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>29.83</u>
7. HOURS GENERATOR ON LINE	<u>59.92</u>	<u>59.92</u>	<u>11830.84</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>120864</u>	<u>120864</u>	<u>33635128</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>39232</u>	<u>39232</u>	<u>11100804</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>27226</u>	<u>27226</u>	<u>10567455</u>
12. REACTOR SERVICE FACTOR	<u>8.9%</u>	<u>8.9%</u>	<u>60.1%</u>
13. REACTOR AVAILABILITY FACTOR	<u>8.9%</u>	<u>8.9%</u>	<u>60.3%</u>
14. UNIT SERVICE FACTOR	<u>8.0%</u>	<u>8.0%</u>	<u>58.9%</u>
15. UNIT AVAILABILITY FACTOR	<u>8.0%</u>	<u>8.0%</u>	<u>58.9%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>3.5%</u>	<u>3.5%</u>	<u>50.8%</u>
17. UNIT CAPACITY FACTOR(USING DESIGN MWe)	<u>3.4%</u>	<u>3.4%</u>	<u>48.8%</u>
18. UNIT FORCED OUTAGE RATE	<u>0%</u>	<u>0%</u>	<u>26.4%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A Unit is in shutdown during this period.			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP			<u>5/9/87</u>

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
 UNIT: LASALLE TWO
 DATE: February 10, 1987
 COMPLETED BY: Gregory Kirchner
 TELEPHONE: (815) 357-6761
 MONTH: January, 1987

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

1.	716
2.	715
3.	115
4.	-16
5.	-16
6.	-15
7.	-16
8.	-16
9.	-16
10.	-15
11.	-16
12.	-15
13.	-15
14.	-14
15.	-14
16.	-14

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

17.	-14
18.	-14
19.	-14
20.	-15
21.	-14
22.	-13
23.	-14
24.	-13
25.	-14
26.	-12
27.	-11
28.	-14
29.	-14
30.	-14
31.	-12

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
 UNIT NAME LaSalle Two
 DATE February 10, 1987
 COMPLETED BY G.J. Kirchner
 TELEPHONE (815)357-6761

REPORT MONTH January, 1987

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
1	01/3/87	S	677.95	C	1	Refueling Outage.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
01/04/87	2B21-F013A	2-Manual	Shortly after Shutdown	* Note
01/04/87	2B21-F013B	2-Manual	After S/D	* Note
01/04/87	2B21-F013C	2-Manual	After S/D	* Note
01/04/87	2B21-F013E	2-Manual	After S/D	* Note
01/04/87	2B21-F013F	2-Manual	After S/D	* Note
01/04/87	2B21-F013G	2-Manual	After S/D	* Note
01/04/87	2B21-F013H	2-Manual	After S/D	* Note

* NOTE: The above SRV's were manually opened to reduce reactor pressure and facilitate cooldown following shutdown for refueling.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-02-87	2B Diesel Generator D/G	Local Switch Mod.
2-10-87	2B Diesel Generator	2B D/G Inspection.
2-11-87	2B D/G Diesel Air Compressor	Repair Air Leak.
2-12-87	2B D/G Motor Air Compressor	Adjust Oil Drain.
2-16-87	2B D/G	Calibrations.
2-36-87	2B D/G	Install Prelube Pump.
2-41-87	2E12-F092 B/C	LTS-500-10 Div. 2 ECCS Response Time.
2-57-87	2E21-F051	LES-LP-201 LPCS Relay Logic Test.
2-59-87	RHR "2A" Valves and RCIC Steam to HX	LES-RH-200.
2-83-87	RCIC Steam to Turbine 2E51-F045	Limiterque Surveillance.
2-85-87	RCIC Pump Discharge 2E51-F013	Limiterque Surveillance.
2-87-87	2B21-F017	Monitor MSL Leakage Past MSL Plugs.
2-96-87	RCIC Suction from CY Valve 2E51-F010	Limiterque Surveillance.
2-97-87	2E51-F019	Limiterque Surveillance.
2-98-87	RCIC Suppr. Pool System 2E51-F031	Limiterque Surveillance.
2-99-87	RCIC Steam Isol. 2E51-F063	Limiterque Surveillance.
2-100-87	RCIC Steam Line Warmup 2E51-F076	Limiterque Surveillance.
2-101-87	PCIS Steam Line Isolation 2E51-F008	Limiterque Surveillance.
2-102-87	RHR B Test Check Valve 2E12-F041B	Repair Solenoid.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-101-87	RCIC Steam Line Isolation Valve - 2E51-F008	Limiterorque Surveillance.
2-102-87	RHR B Test Check Valve 2E12-F041B	Repair Solenoid.
2-103-87	RHR B Test Check Valve 2E12-F015	Repair Solenoid.
2-124-87	2/B D/G Disconnect for Motor Starter K24A	Operational Control.
2-133-87	RCIC Steam Supply Bypass Valve to Heat Exchanger 2E51-F091	Limiterorque Surveillance.
2-134-87	RCIC Steam Supply Valve to RH HX - 2E51-F064	Limiterorque Surveillance.
2-135-87	HP Suction from CY Valve 2E22-F015	Limiterorque Surveillance.
2-136-87	HP Test to Supp. Pool Valve 2E12-F023	Limiterorque Surveillance.
2-138-87	HPCS Pump	Pump Protection No Suction.
2-143-87	2E12-F009	Inspection.
2-151-87	LPCS	Repack 2E21-F011 and F012
2-154-87	2E22-F012	Limiterorque Surveillance.
2-161-87	IN Valves	To Prevent Inadvertant SRV. Actuation.
2-167-87	HPCS Pump	Change Oil.
2-182-87	2E12-F026B	Rotate Actuator.
2-183-87	HPCS D/G Output BKR ACB 2433	Check Breaker Adjustments and Relay Connection.
2-184-87	HPCS Pump Breaker	Check Aux. Cab. Adjust.
2-187-87	2E51-F086	Limiterorque Surveillance.
2-188-87	2E51-F080	Limiterorque Surveillance.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-189-87	2E51-F069	Limiterque Surveillance.
2-190-87	2E51-F046	Limiterque Surveillance.
2-191-87	2E51-F068	Limiterque Surveillance.
2-196-87	2A RHR CY Fill/Flush Supply	To Fill and Flush 2A RHR.
2-198-87	2E21-N005	Add Pressure Switch 2E21-N005B.
2-199-87	2E12-N022A	Changeout 'A' RHR Pressure Switch 2E12-N022A.
2-204-87	HPCS	Limiterque Work.
2-205-87	HPCS D/G CW Pump	Repack Strainer B/D Valve.
2-207-87	2E51-F063, 2B33-F067B, 2B33-F023B	LES-AP-201.
2-215-87	RCIC Vacuum Pump	Replace Pump.
2-216-87	RCIC 2E51-F063	Repack.
2-221-87	2B Diesel Generator	Replace Air Start Solenoids, and Lube Oil Pump.
2-233-87	2E22-F004	Limiterque.
2-238-87	2E12-F063A	To Fill and Vent 'A' RHR.
2-239-87	'A' RHR	Prevent Draining Vessel.
2-240-87	2E12-F017A	Limiterque Surveillance.
2-241-87	2E12-F048A	Limiterque Surveillance.
2-242-87	2E12-F027A	Limiterque Surveillance.
2-243-87	2E12-F042A	Limiterque Surveillance.
2-244-87	2E12-F004A	Limiterque Surveillance.
2-245-87	2E12-F049A	Limiterque Surveillance.
2-246-87	2E12-F047A	Limiterque Surveillance.
2-247-87	2E12-F064A	Limiterque Surveillance.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-248-87	2E12-F068A	Limiterque Surveillance.
2-249-87	2E12-F052A	Limiterque Surveillance.
2-250-87	'A' RHR Min. Flow Check Valve - 2E12-F046A	Inspect and Repair.
2-251-87	Aux. Equipment on 2B D/G	Repair Air Compressor "BB" Breaker.
2-260-87	2-B D/G Air Start System	To Assist in Charging Air Receivers.
2-263-87	S/D Cooling Return Valve 2E12-F053A	Limiterque Inspection.
2-264-87	Steam Line Isolation Valve 2E12-F087A	Limiterque Inspection.
2-265-87	Heat Exchanger Outlet Valve 2E12-F003A	Limiterque Inspection.
2-271-87	Common D/G	LES-DG-100.
2-272-87	Common D/G Output BKR 2413	Check Wiring Term on 0DG11JB
2-273-87	Common D/G Output BKR 1413	Check Wiring Term on 0DG11JA
2-274-87	Common D/G	Refuel Inspection.
2-278-87	2E12-N022A	Install New Pressure Switch 2E12-N512A
2-279-87	2E12-N022B	Install New Pressure Switch 2E12-N512B
2-280-87	2B RHR	Prevent Inadvertant Drain of Reactor.
2-281-87	LPCS Pump Motor	Change Oil.
2-284-87	2E21-F005	Limiterque Surveillance.
2-285-87	2E21-F001	Limiterque Surveillance.
2-286-87	2E21-F011	Limiterque Surveillance.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-287-87	2E21-F012	Limiterque Surveillance.
2-288-87	2DG035	Limiterque Surveillance.
2-289-87	Division III BUS 243	Calibrate UV Relays.
2-290-87	LPCS System	Prevent Draining Reactor. Protect Pump During Limiterque Surveillance.
2-302-87	'A' RHR CY Flush Line	To facilitate flushing, and Filling.
2-309-87	2B D/G	LES-DG-202
2-310-87	2B D/G	Fix Oil Leaks.
2-311-87	2E12-F099A	Limiterque Surveillance.
2-312-87	2E12-F011A	Limiterque Surveillance.
2-317-87	2E12-F006A	Limiterque Surveillance.
2-326-87	2E12-F063B	Fill 'B' RHR.
2-330-87	'A' RHR	Install 2 New Vents.
2-331-87	Shutdown Cooling Suction	Install Thermowell.
2-334-87	2B/C RHR Discharge Flow Alarm	Split Hi/Lo Flow Switches into 2 Switches.
2-335-87	HPCS	Relief Valve Testing.
2-341-87	2A RHR 2E12-F031A	Repair Check Valve 2E12-F031A
2-379-87	2E12-F024A	Limiterque Surveillance.
2-380-87	2E12-F053B	Limiterque Surveillance.
2-381-87	2E12-F042B	Limiterque Surveillance.
2-382-87	2E12-F016B	Limiterque Surveillance.
2-383-87	2B RHR	Vent Modification.
2-384-87	2DG036	Trouble Shoot and Repair Check Valve.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-385-87	2B D/G	Replace Cooling Water Temperature Switch.
2-379-87	2E12-F024A	Limiterque Surveillance.
2-380-87	2E12-F053B	Limiterque Surveillance.
2-381-87	2E12-F042B	Limiterque Surveillance.
2-382-87	2E12-F016B	Limiterque Surveillance.
2-383-87	2B RHR	Vent Modification.
2-384-87	2DG036	Trouble Shoot and Repair Check Valve.
2-385-87	2B D/G	Replace Cooling Water Temperature Switch.
2-389-87	2E12-F074B	Limiterque Surveillance.
2-390-87	2E12-F040A	Limiterque Surveillance.
2-391-87	2E12-F099B	Limiterque Surveillance.
2-392-87	2E12-F003B	Limiterque Surveillance.
2-393-87	2E12-F047B	Limiterque Surveillance.
2-394-87	2E12-F048B	Limiterque Surveillance.
2-395-87	2E12-F068B	Limiterque Surveillance.
2-399-87	LPCS	Check Out Ammeter.
2-401-87	2E12-F064B	Limiterque Inspection.
2-402-87	2E12-F011B	Limiterque Surveillance.
2-403-87	2E12-F004B	Limiterque Surveillance.
2-404-87	2E12-F027B	Limiterque Surveillance.
2-423-87	2B D/G	Repair/Calibrate Oil Temperature System.
2-535-87	2B D/G	Rework Termination at K24A Starter Cabinet.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-538-87	2E12-F068A	Repack.
2-539-87	2E51-F064	Repack.
2-540-87	2A/D/G	Prelube Modification.
2-541-87	2A/D/G	18 Month LES's.
2-542-87	2A/D/G	18 Month M.M. Inspections.
2-544-87	2B/D/G	Flush D/G Cooler.
2-546-87	2E12-F409B	Replace Valve.
2-549-87	2E12-F026A	Limiterque Surveillance.
2-550-87	2E12-C300D	Inspect Motor Winding.
2-551-87	2/C RHR	Prevent Draining Reactor Vessel During Limitorque LES

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment Systems during this reporting period.

5. Indications of Failed Fuel Elements.

There were no indications of failed fuel elements during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

February 6, 1987

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for the period January 1, 1987 through
January 31, 1987.

Very truly yours,

for G. J. Diederich
Station Manager
LaSalle County Station

GJD/GJK/jdp

Enclosure

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