

MAY 2 1986

Docket No. 50-440

Mr. Murraray R. Edelman  
Vice President - Nuclear Group  
The Cleveland Electric Illuminating Company  
P.O. Box 5000  
Cleveland, Ohio 44101

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Dear Mr. Edelman:

The Comptroller General in a Report to the Congress dated September 19, 1984, stated "NRC needs better management controls to ensure implementation of generic issue solutions." To correct this situation, the Nuclear Regulatory Commission is working on a long-term program which will improve its documentation on the status of power reactor licensees' implementation of approved regulatory improvements.

To assist me in developing this information for the Perry Nuclear Power Plant, please review the enclosed plant-specific list, add omitted significant actions to the list, and provide the date that all aspects of each identified requirement were completed/implemented or are expected to be completed. For each requirement that has been completed, enter "completed" under the Status column and the date the item was completed. For requirements that have not been completed/implemented, provide the projected date and enter "projected" under the Status column. I do not expect or want a major effort made to obtain precise dates. Rather, I will accept readily-known or best-estimate dates for actions that have been completely implemented. For example, the year only would be acceptable for completions prior to 1984; the month as well as year is needed for requirements completed after that time. For projected completions/implementations in 1986, use month and year. Use year only for projections thereafter. Please provide a response in the format enclosed by May 31, 1986. Should there be any questions or clarification needed, please have your staff get in touch with me. The need for this information request has been discussed with your staff.

In the future, NRR will need estimated dates prior to issuance of the safety evaluation reports so that the estimated schedule may be included in safety evaluation reports and supplements thereto.

This request is covered by OMB Clearance Number 3150-0011, which expires September 30, 1986.

Sincerely,

Original signed by

8605130144 860502  
PDR ADOCK 05000440  
A PDR

John J. Stefano, Project Manager  
BWR Project Directorate No. 4  
Division of BWR Licensing

Enclosure:  
As stated

cc: See next page

PD#4/PM  
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05/02/86

PD#4/D  
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05/12/86



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

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Vice President - Nuclear Group  
The Cleveland Electric Illuminating Company  
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Sincerely,

  
John J. Stefano, Project Manager  
BWR Project Directorate No. 4  
Division of BWR Licensing

Enclosure:  
As stated

cc: See next page

Mr. Murray R. Edelman  
The Cleveland Electric  
Illuminating Company

Perry Nuclear Power Plant  
Units 1 and 2

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Enclosure

<u>Item</u>	<u>Action Completed</u>	<u>Action Implemented</u>	<u>Reference/Status</u>
1. Separately itemize applicable sections of NRC Generic Letters by Generic Letter No. and Section (e.g. GL 83-28, Section 1.1, 1.2, 2.1, 2.2; GL 82-33, etc.)			
2. All applicable TMI Action Plan Items listed in the Perry SER and SER Supplements.			
3. LRG-II related BWR/6 multi-plant generic issue items (cross reference with TMI Action Plan Items, etc. as appropriate.			
4. The following Multi-Plant Activity Items (MPA #) some of which may be covered under 1 thru 3 above:			
°A-01, Inservice Testing Program			
°A-02, 10CFR50, Appendix I			
°A-1A, 10CFR50.55g, ISI			
°A-17, Instru. to follow course of accidents			
°A-18, TS affected by 10CFR50.72 and 50.73			
°A-20, Implementation of ATWS Rule			
°B-17, TS Surveillance for Hydraulic Snubbers			
°B-22, TS Surveillance for Mechanical Snubbers			
°B-24, Venting and Purging at Full Power			
°B-59, Masonry Wall Design			
°B-77, 2.1 Equip. Class/Vendor Interface-RTS			
°B-78, 3.1.1/.2 Post Maint. Test Proced.			
°B-79, 3.1.3 Post Maint.-TS-RTS			
°B-80, 4.1 RTS Reliability-Vendor Mods.			
°B-81, 4.2.1/.2 Preventative Maint.-TS			
°B-83, 0737TS (GL 83-36/-37)			
°B-84, Insp. of BWR Stainless Steel Pipe			
°B-85, 1.2 Post Trip Review-Info Capability			
°B-86, 2.2 Equip. Class/Vendor Interface			
°B-87, 3.2.1/.2 Post Maint. Test Proced.			
°B-88, 3.2.3 Post Maint. Test Proced/TS			
°B-89, 4.2.3/.4 Maint. Reactor Trip Breakers			
°B-92, 4.5.1 RTS Functional Testing			
°B-93, 4.5.2/.3 RTS Funct. Test-Alternatives			
°C-07, Fuel Handling Accident in Containment			
°C-11, RPS Power Supply			
°D-17, Definition of Operable			
°D-19, GL-84-15 Tech Specs.			

- °E-04, BWR Single Loop Operation
- °F-4a, Revised EOP Tech Guidelines
- °F-5, Proced. Generation Packages
- °F-8, TMI I.D.1 (DCRDR)
- °F-9, TMI I.D.2 (SPDS)
- °F-14, TMI II.D.1.2 RV/SV Tests
- °F-19, TMI II.E.4.2 (all sections)
- °F-26, Instru. for Detection of ICC
- °F-55, TMI II.K.3.28
- °F-57, TMI II.K.3.30
- °F-58, TMI II.K.3.31
- °F-63-65, TMI III.A.1.2
- °F-68, TMI III.A.2.2
- °F-71, (same as F-8)