UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 9, 1999

Mr. S. K. Gambhir
Division Manager - Nuclear Operations
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 399
Hwy. 75 - North of Fort Calhoun
Fort Calhoun, NE 68023-0399

SUBJECT: CLOSURE OF TAC NO. MA0545 - RESPONSE TO THE REQUESTS FOR ADDITIONAL INFORMATION TO GENERIC LETTER 92-01, REVISION 1, SUPPLEMENT 1, "REACTOR VESSEL STRUCTURAL INTEGRITY," FOR THE FORT CALHOUN STATION, UNIT 1

Dear Mr. Gambhir:

On May 19, 1995, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter 92-01, Revision 1, Supplement 1 (GL 92-01, Rev. 1, Supp. 1), "Reactor Vessel Structural Integrity," to holders of nuclear operating licenses. In issuing the GL, the staff required addressees of the GL to:

(1) identify, collect and report any new data pertinent to the analysis of structural integrity of the reactor pressure vessels (RPVs) at their nuclear plants, and

(2) to assess the impact of that data on their RPV integrity analyses relative to the requirements of Sections 50.60 and 50.61 to Part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR 50.60 and 10 CFR 50.61), and to the requirements of Appendices G and H to Part 50 of Title 10 of the *Code of Federal Regulations* (Appendices G and H to 10 CFR Part 50).

On August 17, 1995, you submitted your initial response to GL 92-01, Rev. 1, Supp. 1. and provided the requested information relative to the structural integrity assessment for the Fort Calhoun Station. The staff evaluated your response to GL 92-01, Rev. 1, Supp. 1, and provided its conclusion relative to your response on August 2, 1996. However, since the time of the staff's closure letter, the Combustion Engineering (CE) Owners Group and the Babcock and Wilcox (B&W) Owners Group have each submitted additional data regarding the alloying chemistries of beltline welds in CE and B&W fabricated vessels. The additional alloying data were submitted in Topical Reports CE NPSD-1039, Revision 2, CE NPSD-1119, Revision 1, for CE fabricated RPV welds, and BAW-2325, Revision 1, for B&W fabricated RPV welds. In addition, Chicago Bridge and Iron (CB&I) BWR data were submitted in Topical Report BWRVIP-46. As a result of the efforts by CE and B&W, the staff determined that additional information was necessary relative to the structural integrity assessments for your plants. On July 23, 1998, the staff issued a request for additional information (RAI) in regard to the alloying chemistries of beltline welds, your assessment of surveillance data for your facility, pressure-temperature (P-T) limits, and pressurized thermal shock (PTS) assessment (only applicable to PWRs) for the Fort Calhoun Station. In general, with respect to the contents of the RAI, the

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staff requested that you reassess the alloying chemistries for the beltline welds and RPV surveillance welds relative to the chemistries provided in the applicable topical report, and provide the impact of any changes to the best-estimate chemistries for your beltline RPV welds on the structural integrity assessments for the Fort Calhoun Station relative to the requirements of 10 CFR 50.60, 10 CFR 50.61, and Appendices G and H to 10 CFR Part 50, as applicable to the licensing basis for your plant.

You provided your response to the staff's RAIs for the Fort Calhoun Station on September 28, 1998. As a result of the staff's review of your responses to GL 92-01, Revision 1, GL 92-01, Rev. 1, Supp. 1, and the Supp. 1 RAI, the staff has revised the information in the Reactor Vessel Integrity Database (RVID) and is releasing it as RVID Version 2.

The new database diskettes are posted on the world-wide-web at a location which is linked to the NRC home page (http://www.nrc.gov/NRR/RVID/index.html). We recommend that you review this information. If the staff does not receive comments by September 1, 1999, we will assume that the data entered into the RVID are acceptable for the Fort Calhoun Station. No additional information is necessary with regard to the structural integrity assessments. Future submittals on P-T limits, PTS (only applicable to PWRs), or upper shelf energy (USE) should reference the most current information.

This closes the staff's efforts in regard to TAC No. MA0545. The staff appreciates your efforts in regard to this matter.

Sincerely,

ORIG. SIGNED BY
L. Raynard Wharton, Project Manager, Section 2
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

cc: See next page

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L. Raynard Wharton

L. Raynard Wharton, Project Manager, Section 2
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Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

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