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Quad Cities Generating Station
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ComEd

LWP-97-082

August 8, 1997

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of July, 1997.

Respectfully,

ComEd
Quad-Cities Nuclear Power Station

L. W. Pearce
Station Manager

LWP/dak

Enclosure

IE24

cc: A. Beach, Regional Administrator
C. Miller, Senior Resident Inspector

STMGR\08297.LWP

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

JULY 1997

COMMONWEALTH EDISON COMPANY

AND

MIDAMERICAN ENERGY COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

TABLE OF CONTENTS

I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors and Steam Turbine/Generators, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Midamerican Energy Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Kristal Sirles and Debra Kelley, telephone number 309-654-2241, extensions 3070 and 2240, respectively.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One was on-line the entire month of July 1997. At 0100 hours on July 1, 1997, a load drop was performed to reverse Condenser flow. A few load drops were performed throughout the day to clear and reset Condenser Vacuum Low Alarms and reverse Condenser flow due to the high river water temperatures. Several other load drops were performed due to Weekly Turbine Testing, high river water temperatures and at the request of Bulk Power Operations. However the average daily power level remained at 80% or greater.

B. Unit Two

Quad Cities Unit Two was on-line the entire month of July 1997. On July 15, 1997 at 2010 hours, a load reduction was initiated in preparation for Fuel Assembly Testing due to increased Off-Gas activities. Test results from Reactor and off-gas samples verified fuel failure. The SJAE monitors indicate that the defect developed on July 13, 1997 at 0430 hours. The one-cycle old GE bundle has been located and is being suppressed. On July 20, 1997 at 2130 hours a load drop was performed for Control Rod Shuffles and Weekly Turbine Test.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements of 10CFR50.73.

UNIT 1

Licensee Event Report Number	Date	Title of occurrence
97-017	6/23/97	Unit 1 entered a 12 Hour Hot Shutdown Limiting Condition for Operation Contrary to the Bases for Technical Specification 3.0.A due to Cognitive Personnel Errors when Scheduling Surveillances.
97-018	7/21/97	Missed Tech Spec Surveillance TIP Squib Valve Continuity 4.7.D.5.a

UNIT 2

Licensee Event Report Number	Date	Title of occurrence
97-006	7/29/97	Cable relied upon for Safe Shutdown is located in fire area which requires use of cable.
97-008	6/29/97	Five Control Rod Drives Did Not Receive Required Scram Insertion Time Testing Prior To 40% Power Due To An Operations Programmatic Deficiency.
97-009	7/14/97	Control Room personnel misread an indication delaying discovery of abnormal Offgas radiation readings which interfered with proper completion of a Technical Specification surveillance due to a cognitive personnel error.

NOTE: Sequence of LER Numbers is out of order due to reuse of numbers.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-254
UNIT One
DATE August 8, 1997
COMPLETED BY Kristal Sirles
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 070197 2400 073197 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
   DESIGN ELECTRICAL RATING (MWe-NET): 789
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	744.00	4468.10	168977.30
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	3421.90
7. HOURS GENERATOR ON LINE	744.00	4379.50	163908.40
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	909.20
9. GROSS THERMAL ENERGY GENERATED (MWH)	1799232.00	10282758.00	357455640.50
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	563400.00	3291976.00	115713406.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	536678.00	3134591.00	109299560.00
12. REACTOR SERVICE FACTOR	100.00	87.83	76.18
13. REACTOR AVAILABILITY FACTOR	100.00	87.83	77.73
14. UNIT SERVICE FACTOR	100.00	86.09	73.90
15. UNIT AVAILABILITY FACTOR	100.00	86.09	74.31
16. UNIT CAPACITY FACTOR (Using MDC)	93.80	80.39	64.08
17. UNIT CAPACITY FACTOR (Using Design MWe)	91.42	78.35	62.46
18. UNIT FORCED OUTAGE RATE	0.00	13.91	7.67

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A
20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: N/A
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): N/A

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-265
UNIT Two
DATE August 8, 1997
COMPLETED BY Kristal Sirles
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 070197 / 2400 073197 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	744.00	2395.80	162982.35
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	2985.80
7. HOURS GENERATOR ON LINE	744.00	2338.60	158587.95
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	702.90
9. GROSS THERMAL ENERGY GENERATED (M WH)	1753272.00	5479746.60	345151835.42
Q0. GROSS ELECTRICAL ENERGY GENERATED (MWH)	550644.00	1735410.00	110752547.00
11. NET ELECTRICAL ENEF GY GENERATED (MWH)	527521.00	1661179.00	104980900.00
12. REACTOR SERVICE FACTOR	100.00	47.10	74.00
13. REACTOR AVAILABILITY FACTOR	100.00	47.10	75.36
14. UNIT SERVICE FACTOR	100.00	45.97	72.01
15. UNIT AVAILABILITY FACTOR	100.00	45.97	72.33
16. UNIT CAPACITY FACTOR (Using MDC)	92.20	42.46	61.98
17. UNIT CAPACITY FACTOR (Using Design MWe)	89.86	41.39	60.41
18. UNIT FORCED OUTAGE RATE	0.00	0.49	11.03

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Q2P01 10/4/97 - 10/24/97

20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: N/A

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): N/A

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE August 8, 1997
COMPLETED BY Kristal Sirles
TELEPHONE (309) 654-2241

MONTH July 1997

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	631	17.	718
2.	659	18.	709
3.	727	19.	695
4.	734	20.	686
5.	699	21.	707
6.	710	22.	745
7.	749	23.	752
8.	749	24.	747
9.	747	25.	735
10.	716	26.	721
11.	741	27.	677
12.	744	28.	695
13.	751	29.	699
14.	748	30.	748
15.	740	31.	738
16.	730		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE August 8, 1997
COMPLETED BY Kristal Sirles
TELEPHONE (309) 654-2241

MONTH July 1997

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	741	17.	568
2.	739	18.	729
3.	750	19.	699
4.	700	20.	451
5.	758	21.	663
6.	712	22.	725
7.	759	23.	702
8.	760	24.	757
9.	762	25.	749
10.	762	26.	737
11.	757	27.	669
12.	757	28.	744
13.	731	29.	732
14.	742	30.	754
15.	651	31.	755
16.	450		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME One COMPLETED BY Kristal Sirles

DATE August 8, 1997 REPORT MONTH July 1997 TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
97-10	7-1-97	S	0	H	5	--------	---	----	Load Drop to Reverse Condenser Flow

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Two COMPLETED BY Kristal Sirles

DATE August 8, 1997 REPORT MONTH July 1997 TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
97-05	7/15/97	F	0	A	5	----------	---	----	Initiated Load Drop in preparation for Fuel Assembly Testing.
97-06	7/20/97	S	0	B	5	----------	---	----	Load Drop for Control Rod Shuffle and Turbine Testing.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram timing for the reporting period. All scram timing as performed with reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS
PERFORMED ON UNIT 1 & 2 CONTROL
ROD DRIVES, FROM 01/01/97 TO 06/30/97

		AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
DATE	NUMBER OF RODS	5 0.375	20 0.900	50 2.00	90 3.5	7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
2/17/97	14	0.314	0.712	1.528	2.671	3.010 (J-14)	STT for Viton Issue U-1 (Core Wid Ave. Times)
3/29-31/97	21	0.316	0.708	1.513	2.645	3.010 (J-14)	STT for Tech Spec (19) (Core Wide Ave. Times) PMTV (2)
4/25/97	1	0.29	0.64	1.37	2.4	3.010 (J-14)	Post Maintenance Test for Accumulator Replacment Unit 1
5/31/97	14	0.354	0.725	1.497	2.576	2.77 (C-15)	STT for Viton Issue - U1
6/23/97	13	0.297	0.681	1.478	2.615	2.95 (M-12)	Post Outage Scram Test Timing and PMTV - U2
6/29/97	4	0.278	0.630	1.348	2.350	2.46 (P-6)	Post Outage Scram Testing and PMTV - U2
7/16/97	1	0.280	0.630	1.324	2.306	2.306 (N-6)	PMTV - U2
7/20/97	32	0.312	0.701	1.497	2.617	2.67 (G-15)	TS & SSPV Testing U-1

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 0300-S32
Revision 3
April 1997

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q1 Reload: 14 Cycle: 15

2. Scheduled date for next refueling shutdown: 9/05/98

3. Scheduled date for restart following refueling: 11/14/98

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   Yes

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   November, 1997

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   Approx. 216 SPC 9X9IX Fuel Bundles Q1R15 will be loaded.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1933

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2002

QTP 0300-S32
Revision 3
April 1997

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q2 Reload: 14 Cycle: 15

2. Scheduled date for next refueling shutdown: 2/20/99

3. Scheduled date for restart following refueling: 3/27/99

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   Yes

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   August, 1998

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   N/A

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 2943

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2002

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM - Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI - American National Standards Institute
APRM - Average Power Range Monitor
ATWS - Anticipated Transient Without Scram
BWR - Boiling Water Reactor
CRD - Control Rod Drive
EHC - Electro-Hydraulic Control System
EOF - Emergency Operations Facility
GSEP - Generating Stations Emergency Plan
HEPA - High-Efficiency Particulate Filter
HPCI - High Pressure Coolant Injection System
HRSS - High Radiation Sampling System
IPCLRT - Integrated Primary Containment Leak Rate Test
IRM - Intermediate Range Monitor
ISI - Inservice Inspection
LER - Licensee Event Report
LLRT - Local Leak Rate Test
LPCI - Low Pressure Coolant Injection Mode of RHRs
LPRM - Local Power Range Monitor
MAPLHGR - Maximum Average Planar Linear Heat Generation Rate
MCPR - Minimum Critical Power Ratio
MFLCPR - Maximum Fraction Limiting Critical Power Ratio
MPC - Maximum Permissible Concentration
MSIV - Main Steam Isolation Valve
NIOSH - National Institute for Occupational Safety and Health
PCI - Primary Containment Isolation
PCIOMR - Preconditioning Interim Operating Management Recommendations
RBCCW - Reactor Building Closed Cooling Water System
RBM - Rod Block Monitor
RCIC - Reactor Core Isolation Cooling System
RHRS - Residual Heat Removal System
RPS - Reactor Protection System
RWM - Rod Worth Minimizer
SBGTS - Standby Gas Treatment System
SBLC - Standby Liquid Control
SDC - Shutdown Cooling Mode of RHRS
SDV - Scram Discharge Volume
SRM - Source Range Monitor
TBCCW - Turbine Building Closed Cooling Water System
TIP - Traversing Incore Probe
TSC - Technical Support Center