

August 13, 1997

Mr. Roger O. Anderson, Director
Licensing and Management Issues
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON THE PRAIRIE ISLAND
NUCLEAR GENERATING PLANT, UNITS 1 AND 2, AMENDMENT REQUEST
ON INCORPORATION OF VOLTAGE-BASED STEAM GENERATOR TUBE
REPAIR CRITERIA (TAC NOS. M98944 AND M98945)

Dear Mr. Anderson:

By letter dated May 15, 1997 the Northern States Power (NSP) Company submitted for staff review a license amendment request to incorporate the voltage-based steam generator tube repair criteria in the technical specifications (TS) for the Prairie Island Nuclear Generating Plant. The proposed changes would be made to TS 3.1.C, "Reactor Coolant System Leakage," and 4.12, "Steam Generator Tube Surveillance." After reviewing the information, the staff requires further information. Our request for additional information (RAI) is enclosed.

In order to continue our review of your submittal, please provide your response to the staff's RAI within 20 days of the receipt of this letter in order to support your requested outage schedule. If you have any questions regarding the content of the RAI, please contact me at (301) 415-2834.

Sincerely,

Orig. signed by
David T. Diec, Acting Project Manager
Project Directorate III-I
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-282, 50-306

Enclosure: As stated

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Mr. Roger O. Anderson, Director
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Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON THE PRAIRIE ISLAND
NUCLEAR GENERATING PLANT, UNITS 1 AND 2, AMENDMENT REQUEST
ON INCORPORATION OF VOLTAGE-BASED STEAM GENERATOR TUBE
REPAIR CRITERIA (TAC NOS. M98945 - M98945)

Dear Mr. Anderson:

By letter dated May 15, 1997, Northern States Power (NSP) Company submitted for staff review a license amendment request to incorporate the voltage-based steam generator tube repair criteria in the technical specifications (TS) for the Prairie Island Nuclear Generating Plant. The proposed changes would be made to TS 3.1.C, "Reactor Coolant System Leakage," and 4.12, "Steam Generator Tube Surveillance." After reviewing the information, the staff requires further information. Our request for additional information (RAI) is enclosed.

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Sincerely,

A handwritten signature in black ink, appearing to read "David T. Diec".

David T. Diec, Acting Project Manager
Project Directorate III-I
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-282, 50-306

Enclosure: As stated

cc w/encl: See next page

Mr. Roger O. Anderson, Director
Northern States Power Company

Prairie Island Nuclear Generating
Plant

cc:

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Plant
Northern States Power Company
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Welch, Minnesota 55089

Plant Manager
Prairie Island Nuclear Generating
Plant
Northern States Power Company
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Welch, Minnesota 55089

Tribal Council
Prairie Island Indian Community
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St. Paul, Minnesota 55101-2145

REQUEST FOR ADDITIONAL INFORMATION
REVIEW OF LICENSE AMENDMENT REGARDING
VOLTAGE-BASED STEAM GENERATOR TUBE REPAIR CRITERIA
AT THE PRAIRIE ISLAND NUCLEAR GENERATING PLANT
(TAC NOS. M98944 AND M98945)

- Q1. Page 2 of Exhibit D. Northern States Power (NSP) states that the latest approved EPRI [Electric Power Research Institute] database (7/8-inch diameter tubing) will be used in performing Generic Letter (GL) 95-05 ("Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes Affected By Outside Diameter Stress Corrosion Cracking") specified calculations. The staff has agreed, in principle, with the Nuclear Energy Institute (NEI) on a protocol by which the industry will periodically update the database used to perform GL 95-05-specified calculations (Reference: Letter from Brian Sheron, NRC, to David Modeen, NEI, dated April 10, 1997). Licensees, including NSP, should follow the guidance of the protocol and provide the staff with the latest database used in performing GL 95-05-specified calculations.
- Q2. Page 2 of Exhibit D. NSP states that tube integrity evaluation will be consistent with Westinghouse WCAP-14277, "SLB Leak Rate and Tube Burst Probability Analysis Methods for ODS/CC at TSP Intersections." However, on page 9 of Exhibit A, NSP references WCAP-14277, Revision 1. Please verify the WCAP revision used to evaluate tube integrity.
- Q3. Address how guidelines in Section 2 of Attachment 1 to GL 95-05 would be implemented. Specifically, (a) discuss the differential pressure (across the tube walls) required for conditional tube burst probability; [The differential pressure is dependent on the operability requirements of the pressurizer power-operated relief valves (PORVs) during power operation.] and (b) verify that Prairie Island satisfies the requirements of GL 90-06, "Resolution of Generic Issue 70, 'Power-Operated Relief Valve and Block Valve Reliability,' and Generic Issue 94, 'Additional Low-temperature Overpressure Protection for Light-water Reactors,' pursuant to 10 CFR 50.54(f)."
- Q4. Page 4 of Exhibit D. It is stated that the industry methodology will be used for new probe variability as approved by the NRC. Discuss which industry methodology will be used and provide reference documents that contain the methodology.
- Q5. Page 4 of Exhibit D. It is not clear to the staff that NSP has committed to the guidelines in Sections 3.b.2, 3.b.3, 3.b.4, 3.c.1, 3.c.2, 3.c.5, 3.c.6, 3.c.7, and 3.c.8 of Attachment 1 to GL 95-05. Please confirm and/or address whether NSP has committed to the above-mentioned guidelines.
- Q6. Section 5.b of Attachment 1 to GL 95-05 recommends that the effectiveness of monitoring procedures for ensuring the timely detection, trending, and response to rapidly increasing leaks should be assessed. In addition, the appropriateness of alarm setpoints on the primary-to-secondary leakage detection instrumentation and the various criteria for operator action in response to detected leakage should also be assessed. Discuss whether the above assessments have been performed to satisfy Section 5.b of Attachment 1 to GL 95-05.

Enclosure