

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit I	DOCKET NUMBER (2) 0 5 0 0 0 1 3 1 8 1 7	PAGE (3) 1 OF 0 1 2
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TITLE (4)
Reactor Water Cleanup System High Flow Isolation Due to Improper Filling/Venting

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																																																															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																																																													
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LICENSEE CONTACT FOR THIS LER (12)

NAME T.S. Ryder - Power Production Engineer	TELEPHONE NUMBER 7 1 7 5 4 2 - 3 2 3 5
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces - i.e., approximately fifteen single-space typewritten lines) (16)

On January 9, 1987 two Reactor Water Cleanup (RWCU) System automatic isolations occurred due to high flow when operations personnel (utility, licensed) failed to ensure RWCU was properly filled and vented prior to placing it in service following maintenance. There were no safety consequences resulting from this event. The RWCU system was subsequently filled and vented and returned to service. The event will be reviewed as part of operator training to prevent recurrence.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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EXT If more space is required, use additional NRC Form 366A's (17)

DESCRIPTION OF EVENT

On January 9, 1987 at 1703 hours with Unit 1 operating at 100% power, an Engineered Safety Feature (ESF) actuation occurred when the Reactor Water Cleanup (RWCU) System (EIIS Code: CE) automatically isolated. Maintenance had just been completed to alleviate leakage problems with the end bell of the Non-Regenerative Heat Exchanger (NRHX) and an attempt was being made to restore the RWCU System. The outboard RWCU containment isolation valve, 1F004, was opened and a RWCU isolation occurred due to high flow. Another attempt was made to open the 1F004 valve and another RWCU isolation was received.

CAUSE OF EVENT

The event was caused by operations personnel error (utility, licensed) due to failure to verify the RWCU System was filled and vented in accordance with the operating procedure prior to opening the 1F004 valve. The system had been isolated and depressurized downstream of the 1F004 valve to perform maintenance on the NRHX. The high flow isolation was determined to have resulted from a sudden flow spike as water rushed into the downstream piping which was partially voided. It was verified that the cause of the high flow isolation was not because of system leakage. Upon investigation it was determined that no water hammer had occurred.

REPORTABILITY

This event was determined reportable per 10CFR50.73 (a) (2) (iv), in that unplanned ESF actuations occurred when the RWCU System automatically isolated. There were no safety consequences resulting from this event. This assessment is based on the fact that the RWCU System performed its intended function of containment isolation upon receipt of the high flow signal. The intended function would have been performed regardless of power level.

CORRECTIVE ACTION

The RWCU System was drained to the condenser, refilled and vented and returned to service. Actions to prevent recurrence consist of reviewing this event as part of operator training with emphasis upon following procedures and the importance of filling and venting systems.

ADDITIONAL INFORMATION

There have been previous ESF actuations involving RWCU isolations initiated from high flow signals; however, none of the previous RWCU isolations were caused by personnel failing to verify the system was filled and vented in accordance with the operating procedure prior to putting it in service.



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

February 6, 1987

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 87-001-00
FILE R41-2
PLAS - 230

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 87-001-00. This event was determined reportable per 10CFR 50.73 (a) (2) (iv), in that an unplanned Engineered Safety Feature (ESF) actuation occurred when the Reactor Water Cleanup System isolated.

R. G. Byram
Superintendent of Plant - Susquehanna

TSR/cmw

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