

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) OYSTER CREEK, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 2 1 1 9	PAGE (3) 1 OF 0 3
---	--	----------------------

TITLE (4)
CONTROL ROD DRIVE HYDRAULIC CONTROL UNITS NOT INSTALLED PER DESIGN

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 8	2 8	8 6	8 6	0 2	2 0 0	0 9	2 2	8 6			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9) R

POWER LEVEL (10) 0 1 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

20.402(b)	20.406(e)	50.73(a)(2)(iv)	73.71(b)
20.406(a)(1)(i)	50.38(a)(1)	50.73(a)(2)(v)	73.71(e)
20.406(a)(1)(ii)	50.38(a)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
John Galanto, Plant Engineering	6 0 9 9 7 1 - 4 3 4 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A discrepancy was noted between the installation of 80 out of 137 Control Rod Drive (CRD) Hydraulic Control Units (HCU) and the installation drawing. An analysis shows that the units in question do not meet the seismic requirement defined by Regulatory Guide 1.26 or the original design basis of the plant for Seismic Class I equipment. The HCUs direct high pressure water from the CRD pumps for normal CRD movement and act as a high pressure water source for the reactor scram function. If the units were rendered inoperable by a seismic event the Standby Liquid Control system would be available for reactor shutdown. Additionally, if the piping attached to the HCUs were ruptured, a small break loss of coolant accident would result inside the Secondary Containment due to reactor water leaking past the CRD seals and out of the break. The improperly installed HCUs will be modified to meet seismic criteria as defined in Regulatory Guide 1.26. This occurrence is reportable under 10CFR50.73(a)(2)(ii)(B).

8609260068 860922
PDR ADOCK 05000219
S PDR

IEZ

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Oyster Creek, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 1 9 8 6 - 0 2 2 - 0 1 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Date of Occurrence

The improper installation of the Hydraulic Control Units was determined to be reportable on August 22, 1986.

Identification of Occurrence

An analysis was performed on the 80 improperly installed HCUs. It was determined that these HCUs do not meet the original design criteria for Seismic Class I equipment as is stated in the Oyster Creek Final Safety Analysis Report (FSAR). This discrepancy is reportable under 10CFR50.73(a)(2)(ii)(B).

Conditions Prior to Occurrence

At the time of the determination, the plant was shutdown for refueling. The condition of the 80 HCUs has existed since initial plant startup.

Description of Occurrence

A concern was raised in April of 1986 regarding the method used to anchor many of Oyster Creek's Control Rod Drive Hydraulic Control Units. These units supply the Control Rod Drives (CRDs) with high pressure water from the CRD pumps for normal operation and act as a high pressure water source for the reactor scram function. They are classified as Seismic Class I.

The HCUs were not installed as specified by the installation drawing. The drawing specifies that the units are to be bolted in place; however, eighty (80) of the 137 units are held in place by four (4) small steel plates welded at the bolting hole locations and then welded to a large steel base plate. The steel base plate is anchored to the Reactor Building floor.

A design review was conducted on the current configuration. The analysis shows that the existing method of anchoring the 80 affected HCUs does not meet the minimum requirements for a design basis earthquake as is stated in the Oyster Creek FSAR.

Apparent Cause of Occurrence

The cause of this problem is an installation field change that was not incorporated into the design drawings, and not analyzed for seismic qualification during original plant construction.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Oyster Creek, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 1 9	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 6	- 0 2 2	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 368A's) (17)

Analysis of Occurrence and Safety Assessment

The failure of the HCU supports during a seismic event may have rendered the HCUs inoperable and therefore placed the plant in an unanalyzed condition. With all HCUs totally inoperable plant shutdown can be achieved using the Standby Liquid Control system. This system is Seismic Class I and is designed to bring the reactor to a shutdown condition at any time in core life independent of control rods.

Additionally, if the piping attached to the HCUs were ruptured, a small break loss of coolant accident would result inside the Secondary Containment due to reactor water leaking past the CRD seals and out of the break.

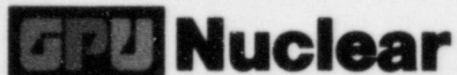
Corrective Action

The Hydraulic Control Unit supports will be upgraded such that a safe shutdown with control rods can be achieved during a design basis earthquake. The upgrade will be performed prior to startup from the current refueling outage.

Failure Data

Not Applicable

(0231A)



GPU Nuclear Corporation
Post Office Box 388
Route 9 South
Forked River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number:

September 22, 1986

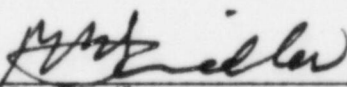
U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report

This letter forwards one (1) copy of Licensee Event Report (LER)
No. 86-022.

Very truly yours,



Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:BP:dsm(0231A)
Enclosures

cc: Dr. Thomas E. Murley, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Mr. Jack N. Donohew, Jr.
U.S. Nuclear Regulatory Commission
7920 Norfolk Avenue, Phillips Bldg.
Bethesda, MD 20014
Mail Stop No. 314

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731

JEZ
11