

GPU NUCLEAR CORPORATION
OYSTER CREEK NUCLEAR GENERATING STATION

PROVISIONAL OPERATING
LICENSE NO. DPR-16

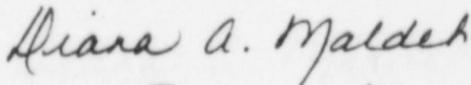
Technical Specification
Change Request No. 140
Docket No. 50-219

Applicant submits, by this Technical Specification Change Request No. 140 to the Oyster Creek Nuclear Generating Station Technical Specifications, a change to Specification 4.4.

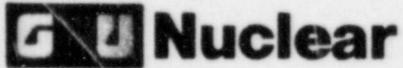
By: _____


Peter B. Fiedler
Vice President and Director
Oyster Creek

Sworn and subscribed to before me this 11th day of September 1986.


Exp. 6/5/91

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September 11, 1986

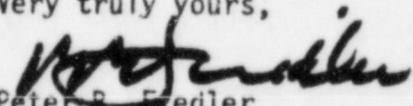
The Honorable Christopher Connors
Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

Dear Mayor Connors:

Enclosed herewith is one copy of the Technical Specification Change Request No. 140 for the Oyster Creek Nuclear Generating Station Operating License.

This document was filed with the United States Nuclear Regulatory Commission on September 11, 1986.

Very truly yours,



Peter B. Fredler
Vice President and Director
Oyster Creek

PBF/pa
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Enclosures

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

IN THE MATTER OF)
GPU NUCLEAR CORPORATION)

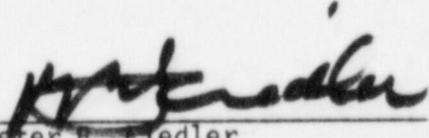
DOCKET NO. 50-219

CERTIFICATE OF SERVICE

This is to certify that a copy of Technical Specification Change Request No. 140 for the Oyster Creek Nuclear Generating Station Technical Specifications, filed with the United States Nuclear Regulatory Commission on September 11, 1986, has this 11th day of September, 1986, been served on the Mayor of Lacey Township, Ocean Country, New Jersey by deposit in the United States mail, addressed as follows:

The Honorable Christopher Connors
Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

By


Peter B. Fiedler

Vice President and Director
Oyster Creek

OYSTER CREEK NUCLEAR GENERATING STATION
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 140

Applicant hereby requests the Commission to change Appendix A of the license. Also, pursuant to 10 CFR 50.91, an analysis concerning significant hazards considerations is presented:

1. Sections to be Changed

4.4-B.1 and the Bases of 4.4

2. Extent of Change

Modify Specification 4.4.B.1 such that the Electromatic Relief Valves (EMRVs) are demonstrated to be operable at system operating pressure before the exceeding 5 percent power. In addition, eliminate the reference to low pressure testing contained in the Bases of Section 4.4

3. Changes Requested

As indicated in the attached revised Technical Specification pages 4.1-1 and 4.4-2.

4. Discussion

Specification 3.4.B.1 currently states that the EMRVs shall be operable when the reactor water temperature is greater than 212°F and pressurized above 110 psig. GPU Nuclear has determined that this specification is too restrictive in that it does not allow for EMRV testing at pressures representative of those at which the EMRVs are anticipated to operate. Testing of the valves at representative operating conditions provides the best assurance that these valves will operate satisfactorily if called upon.

In light of the above, GPU Nuclear intends to change Specification 4.4-B.1 such that the EMRVs may be demonstrated operable at system operating pressures prior to exceeding 5 percent power. In addition, in order to remove a source of confusion, reference to low pressure testing of the EMRV's shall be eliminated from the basis section.

The Automatic Depressurization System (ADS) consists of five automatically activated EMRVs which depressurize the reactor coolant system during a small break LOCA to permit the low pressure core spray system to inject water onto the core, and to provide overpressure protection for anticipated plant transients.

The EMRVs are tested for operability every refueling outage in accordance with Specification 4.4.B when there is essentially no decay heat. In the event of a leak or rupture coincident with the test and the failure of all five EMRVs, the Isolation Condensers can depressurize the reactor coolant system since there would be no stored energy in the fuel and decay heat would be reduced by time following a shutdown from operation. The depressurization capability of the Isolation Condensers is sufficient for testing following a refueling outage and for testing as necessary during the operating cycle.

The ADS is designed to depressurize the reactor coolant system during a small break LOCA to permit the low pressure core spray system to inject water into the core. Testing the EMRV's at system pressures representative of normal operating parameters would not expose the plant to conditions beyond which it is designed to operate.

5. Determination

GPU Nuclear has determined that operation of the Oyster Creek Nuclear Generating Station in accordance with the proposed Technical Specifications does not involve a significant hazard. The change does not:

1. Involve an increase in the probability or consequences of an accident previously evaluated. Testing the EMRV's at low power levels and at pressures representative of operating conditions does not increase the probability of a small break LOCA. In the unlikely event of a small break LOCA during testing of the EMRV's and the coincident failure of all five EMRVs, the Isolation Condensers will be available to depressurize the reactor coolant system and initiate core spray.
2. Create the possibility of a new or different kind of accident from any accident previously evaluated. The Isolation Condensers will be available to depressurize the reactor coolant system and initiate core spray in the unlikely event of a small break LOCA during EMRV testing coincident with the failure of all five EMRV's.
3. Involve a significant reduction in a margin of safety. The EMRV's are tested at low power with alternate depressurization means available in the unlikely event of a small break LOCA coincident with the failure of all five EMRV's.